

Public Service  
Electric and Gas  
Company

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Vice President - Nuclear Operations

APR 21 1989

NLR-N89085

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

REQUEST FOR AMENDMENT  
FACILITY OPERATING LICENSE DPR-75  
SALEM GENERATING STATION  
UNIT NO. 2  
DOCKET NO. 50-311

In accordance with the Atomic Energy Act of 1954, as amended, and the regulations thereunder, Public Service Electric and Gas Company (PSE&G) hereby transmits a request for amendment of Facility Operating License DPR-75 for Salem Generating Station (SGS), Unit No. 2. Pursuant to the requirements of 10CFR50.90(b)(1), a copy of this request has been sent to the State of New Jersey as indicated below.

The proposed change request revision of the Unit 2 Technical Specification Tables 4.3-1 Functional Unit 20 to agree with the Unit 1 Technical Specifications and the Westinghouse Standard Technical Specifications. This change is being submitted on an emergency basis because recent conversations with Mr. J. Stone, NRC Licensing Project Manager disclosed a discrepancy exists between the distribution copy of the Salem Unit 2 Technical Specifications and the NRC record copy of the Salem Unit 2 Technical Specifications. If the requirements of the NRC record copy are applied, a required surveillance would not have been performed in the required time. This would make the Reactor Coolant Pump (RCP) Breaker Position Trip units inoperable. With all 4 RCP breaker position units inoperable, the plant is in a one hour shutdown action statement. The required surveillance requirements cannot be performed with the plant at power.

Therefore please grant a waiver of compliance to the required actions pending approval of this Emergency License Change Request.

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This submittal includes one (1) signed original, including affidavit, and thirty-seven (37) copies pursuant to 10CFR50.4(b)(2)(ii).

Should you have any questions on the subject transmittal, please do not hesitate to contact us.

Sincerely,

*Stanley LaBruna /RAB*

Attachment

C Mr. J. C. Stone  
Licensing Project Manager

Ms. K. Halvey Gibson  
Senior Resident Inspector

Mr. W. T. Russell, Administrator  
Region I

Mr. Kent Tosch, Chief  
New Jersey Department of Environmental Protection  
Division of Environmental Quality  
Bureau of Nuclear Engineering  
CN 415  
Trenton, NJ 08625

## ATTACHMENT 1

### I. Description of Change

Revise Salem Unit 2 Technical Specification Table 4.3-1, Functional Unit 20, Reactor Coolant Pump Breaker Trip Position, on page 3/4 3-12 as follows:

Change the Channel Calibration interval to NA,  
Change the Channel Functional Test to R, and  
Change the Applicable Mode to NA.

### II. Reason for Change

The proposed change is requested in order to make the Unit 2 Technical Specifications consistent with the Unit 1 Technical Specifications. The surveillance interval is also consistent with that specified in the Westinghouse Standard Technical Specifications (WSTS).

### II. Justification for Change

The proposed change is requested to achieve consistency between the two Salem units. This change relieves the station of unnecessary surveillance of the Reactor Coolant Pump Trip Breakers prior to any unit startup. Industry practice shows that a functional test of these channels on a once per refueling outage basis is adequate to ensure proper functioning of the breakers.

### III. Significant Hazards Evaluation

PSE&G has reviewed the proposed change and has determined that it does not involve a significant hazards consideration as discussed below.

1. The proposed change does not increase the probability or consequence of an accident previously evaluated.

The proposed change is consistent with accepted industry practices for surveillance of the Reactor Coolant Pump Breaker Position Trip equipment. The change from once per startup to once per refueling reduces the number of unnecessary equipment

challenges and therefore, increases the probability that the equipment will function as intended. The reactor trip associated with the reactor coolant pump breaker position is not taken credit for in the Salem accident analysis.

2. The proposed change does not create the possibility of a new or different kind of accident.

Since the proposed change does not change any plant hardware nor the manner in which the equipment functions, a new or different kind of accident is not created.

3. The proposed change does not reduce the margin of safety.

Because the proposed change makes Salem Unit 2 consistent with accepted industry practice and since the reactor trip associated with the reactor coolant pump breaker position is not taken credit for in the Salem accident analysis, the margin of safety is not reduced.

#### IV. Conclusion

Based on the above, PSE&G concludes that the proposed change is does not involve a significant hazards consideration.