

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Salem Generating Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 2 1	PAGE (3) OF 0 3
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TITLE (4)
ESF Actuation, Cont. Purge/Pressure - Vac. Relief Isolation, Due To An Equip. Problem

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)											
0	1	0	5	8	9	8	9	0	0	0	2	0	0	0	0	0	0	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 1	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
	20.406(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME M. J. Pollack - LER Coordinator	TELEPHONE NUMBER AREA CODE: 6 0 9 3 3 9 - 4 0 2 2
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS										
B	I	L	D	E	T	V	1	1	5	Yes									

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 5, 1989 at 0419 hours, the 1R41C (Plant Vent Noble Gas Monitor) Radiation Monitoring System (RMS) channel began spiking high (offscale). This resulted in an actuation signal for Containment Ventilation Isolation as well as automatic closure of the Waste Gas Decay Tank Vent Control Valve, 1WG41. Technical Specification Table 3.3-13 Action Statement 31 was subsequently entered. The root cause of this event has been attributed to an equipment problem. The channel back plane was found corroded; the gold contacts were discolored. The back plane contacts on all the modules in both the local and remote 1R41C channel monitors were cleaned. The channel was subsequently returned to service, but not declared operable for two days while close observation of the channel functioning was conducted. Technical Specification Action Statement 31 was exited on January 7, 1989 at 2200 hours when the channel was declared operable. Engineering had initiated, prior to this event, an investigation to provide a permanent fix to the problem(s) associated with the Victoreen model back plane. This investigation is continuing.

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PLANT AND SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as {xx}

IDENTIFICATION OF OCCURRENCE:

Actuation of an Engineered Safety Feature (ESF) signal, Containment Purge Pressure-Vacuum Relief isolation, due to an equipment problem

Event Date: 1/05/89

Report Date: 2/01/89

This report was initiated by Incident Report No. 89-007.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 Reactor Power 100% - Unit Load 1155 MWe

DESCRIPTION OF OCCURRENCE:

On January 5, 1989 at 0419 hours, the 1R41C (Plant Vent Noble Gas Monitor) Radiation Monitoring System (RMS) {IL} channel began spiking high (offscale). This resulted in an actuation signal for Containment Purge/Pressure Vacuum Relief System {BF} isolation as well as automatic closure of the Waste Gas Decay Tank Vent Control Valve, 1WG41. Technical Specification Table 3.3-13 Action Statement 31 was subsequently entered.

Technical Specification Table 3.3-13 Action Statement 31 states:

"With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment provided that prior to initiating the release:

- a. At least two independent samples of the tank's contents are analyzed, and
- b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valving lineup;

Otherwise, suspend release of radioactive effluents via this pathway."

APPARENT CAUSE OF OCCURRENCE:

The root cause of this event has been attributed to an equipment problem. The channel back plane was found corroded; the gold contacts were discolored.

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ANALYSIS OF OCCURRENCE:

Isolation of the Containment Purge/Pressure - Vacuum Relief System is part of the design Engineered Safety Features (ESFs). It mitigates the release of excessive quantities of radioactive material to the environment after a design base accident.

The 1R41C channel monitors the plant vent effluent releases for radioactive noble gases via representative sampling. The ESF actuation feature of Containment Purge/Pressure Vacuum Relief System isolation is redundant to the 1R12A channel which monitors Containment noble gas activity. In addition to the ESF function, the channel will cause the isolation of the 1WG41 valve.

As indicated in the Apparent Cause of Occurrence section, the isolation signal was the result of an equipment problem resulting in a high channel spike. It was not the result of high plant vent activity. Therefore, this event did not affect the health or safety of the public. However, this event is reportable in accordance with the Code of Federal Regulations 10CFR 50.73(a)(2)(iv).

CORRECTIVE ACTION:

The back plane contacts on all the modules in both the local and remote 1R41C channel monitors were cleaned. The channel was subsequently returned to service, but not declared operable for two days while close observation of the channel functioning was conducted. Technical Specification Action Statement 31 was exited on January 7, 1989 at 2200 hours when the channel was declared operable.

The back plane of the Victoreen model detector has proven to be a continuing problem. This back plane is used in most of the Unit 2 RMS detectors. It has periodically caused detector inoperability. Engineering had initiated, prior to this event, an investigation to provide a permanent fix to this problem. This investigation is continuing.



General Manager -
Salem Operations

MJP:pc

SORC Mtg. 89-005



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

February 1, 1989

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

SALEM GENERATING STATION
LICENSE NO. DPR-70
DOCKET NO. 50-272
UNIT NO. 1
LICENSEE EVENT REPORT 89-002-00

This Licensee Event Report is being submitted pursuant to the requirements of the Code of Federal Regulations 10CFR 50.73(a)(2)(iv). This report is required within thirty (30) days of discovery.

Sincerely yours,

L. K. Miller
General Manager-
Salem Operations

MJP:pc
Distribution

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