

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311  
Unit Name Salem # 2  
Date 12-2-88  
Telephone 609-935-6000  
Extension 4451

Completed by Art Orticelle

Month NOVEMBER 1988

Day Average Daily Power Level  
(MWe-NET)

Day Average Daily Power Level  
(MWe-NET)

1 0  
2 0  
3 0  
4 0  
5 0  
6 0  
7 0  
8 0  
9 0  
10 0  
11 0  
12 0  
13 0  
14 0  
15 0  
16 0

17 0  
18 0  
19 0  
20 0  
21 0  
22 0  
23 0  
24 0  
25 0  
26 0  
27 0  
28 0  
29 0  
30 0

8812190256 881130  
PDR ADDCK 05000311  
R PDC

*IEC*  
1/1

OPERATING DATA REPORT

Docket No. 50-311  
 Date 12-2-88  
 Telephone 935-6000  
 Extension 4451

Completed by Pell White

Operating Status

- |       |   |                      |              |
|-------|---|----------------------|--------------|
| 1.    | Unit Name   | <u>Salem No. 2</u>   | <u>Notes</u> |
| 2.    | Reporting Period  | <u>November 1988</u> |              |
| 3.    | Licensed Thermal Power (MWt)  | <u>3411</u>          |              |
| 4.    | Nameplate Rating (Gross MWe)  | <u>1170</u>          |              |
| 5.    | Design Electrical Rating (Net MWe)  | <u>1115</u>          |              |
| 6.    | Maximum Dependable Capacity (Gross MWe)   | <u>1149</u>          |              |
| 7.    | Maximum Dependable Capacity (Net MWe)   | <u>1106</u>          |              |
| 8.    | If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason |                      | <u>N/A</u>   |
| <hr/> |   |                      |              |
| 9.    | Power Level to Which Restricted, if any (Net MWe)                                       |                      | <u>N/A</u>   |
| <hr/> |   |                      |              |
| 10.   | Reasons for Restrictions, if any  |                      | <u>N/A</u>   |

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>720</u>	<u>8040</u>	<u>69865</u>
12. No. of Hrs. Reactor was Critical	<u>90.97</u>	<u>5779.6</u>	<u>38142.5</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>5633.7</u>	<u>36915.5</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>10756.8</u>	<u>18475493</u>	<u>64126014</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>6085020</u>	<u>37714490</u>
18. Net Elec. Energy Generated (MWH)	<u>0</u>	<u>5800419</u>	<u>35819797</u>
19. Unit Service Factor	<u>0</u>	<u>70.1</u>	<u>52.8</u>
20. Unit Availability Factor	<u>0</u>	<u>70.1</u>	<u>52.8</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>65.2</u>	<u>46.4</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>64.7</u>	<u>46.0</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>4.6</u>	<u>27.9</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each)			

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:

Unit returned to Service December 1, 1988 at 0122

UNIT SHUTDOWN AND POWER REDUCTIONS  
REPORT MONTH NOVEMBER 1988

Docket No. 50-311  
Unit Name Salem No.2  
Date 12-2-88  
Telephone 609-935-6000  
Extension 4451

Completed by Art Orticelle

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
0223	11-01-88	S	720	C	4	---	RC	VESSEL	Normal Nuclear Refueling
0223	11-01-88	S	720	C	4	---	RC	VESSEL	Nuclear Reactor Overhaul
0223	11-01-88	S	720	C	4	---	HA	GENERA	Generator Major Overhaul
0223	11-01-88	S	720	C	4	---	HA	TURBIN	Major Turbine Overhaul
0223	11-01-88	S	720	C	4	---	HC	HTEXCH	Misc. Major Condenser Overhaul

1  
F: Forced  
S: Scheduled

2 Reason  
A-Equipment Failure-explain  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & Licensing Exam  
F-Administrative  
G-Operational Error-explain  
H-Other-explain

3 Method  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation of Previous Outage  
5-Load Reduction  
9-Other

4 Exhibit G  
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1  
Salem as Source

PSE&G SALEM GENERATING STATION  
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

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WO NO	UNIT	EQUIPMENT IDENTIFICATION
861001016	2	2SV759-PZR FAILURE DESCRIPTION: 2SV759-PZR VENT LINE 2PR1/PM/REPLACE AIR REG.
871026146	2	22GB923 FAILURE DESCRIPTION: IS RUSTED IN THE OPEN POSITION REWORK THE BONNET.
871026149	2	24GB923 FAILURE DESCRIPTION: RUSTED IN THE OPEN POSITION REWORK D-8 BONNET.
880301220	2	21SW77 FAILURE DESCRIPTION: VALVE FAILS TO CLOSE/REPAIR/A-114.
880510199	2	ELIMINATE RTD BYPASS PIPING PER 2EC-2230 PKG #2 FAILURE DESCRIPTION: INSTALLATION OF NEW THERMOWELLS/RTDS ON HOT AND COLD LEG.

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SALEM UNIT 2

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WO NO	UNIT	EQUIPMENT IDENTIFICATION
880518010	2	"A" REACTOR TRIP BYPASS BREAKER FAILURE DESCRIPTION: REPLACE UV TRIP DEVICE.
880518011	2	"B" REACTOR TRIP BREAKER FAILURE DESCRIPTION: REPLACE UV TRIP DEVICE.
880518012	2	"B" REACTOR TRIP BREAKER FAILURE DESCRIPTION: REPLACE UV TRIP DEVICE.
880518013	2	"A" REACTOR TRIP BREAKER FAILURE DESCRIPTION: REPLACE UV TRIP DEVICE.
880608116	2	REACTOR HEAD THERMOCOUPLE COLUMN DEMOLITION. FAILURE DESCRIPTION: CUT AND WELD CAPS ON FIVE (5) CONOSEALS ON THE REACTOR HEAD.

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## SALEM UNIT 2

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WO. NO	UNIT	EQUIPMENT IDENTIFICATION
880620115	2	21 STM. GEN. "J" NOZZLE FAILURE DESCRIPTION: REPLACEMENT DCR. 2-SM-0567.
880620128	2	22 STM. GEN. "J" NOZZLE FAILURE DESCRIPTION: REPLACEMENT/DCR 2-SM-0567.
880621083	2	23 STM. GEN. "J" NOZZLE FAILURE DESCRIPTION: REPLACEMENT/DCR 2-SM-0567.
880621085	2	24 STM. GEN. "J" NOZZLE FAILURE DESCRIPTION: REPLACEMENT/2-SM-0567.
880906130	2	2C DIESEL GEN SUMP PUMP FAILURE DESCRIPTION: DOES NOT RUN/TROUBLESHOOT.
880915099	2	M.T.L.O. LEVEL ALARM FAILURE DESCRIPTION: CAN NOT BE ACKNOWLEDGED/REWORK.

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## SALEM UNIT 2

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WO NO	UNIT	EQUIPMENT IDENTIFICATION
881021093	2	22 AUX FEED PUMP
		FAILURE DESCRIPTION: HAS EXCESSIVE PACKING LEAKAGE .
881029051	2	21 CFCU
		FAILURE DESCRIPTION: LEAK ON MOTOR COOLER/REPLACE.
881029060	2	2A DIESEL
		FAILURE DESCRIPTION: WAS SYNCHRONIZED OUT OF PHASE PLEASE INSPECT FOR DAMAGE.
881031132	2	2A DIESEL GENERATOR
		FAILURE DESCRIPTION: REMOVE 2A 4KV GENERATOR/SHIP TO CMS FOR REPAIR/REINSTALL.
881101223	2	2EP PZR HTR BUS
		FAILURE DESCRIPTION: WIRING DEGRADED/REWORK.
881101242	2	2C 125 VDC BTRY RM EXH FAN MTR
		FAILURE DESCRIPTION: INOPERABLE/REWORK.

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## SALEM UNIT 2

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WO NO	UNIT	EQUIPMENT IDENTIFICATION
881111047	2	2LT461 PZR LVL
		FAILURE DESCRIPTION: TRACKING 5% LOW/TROUBLESHOOT.
881112046	2	2LT935B-22
		FAILURE DESCRIPTION: 2LT935B-22 ACCUM LVL CHAN "A"-NO ALARM WITH UNSAT LEVEL.
881114078	2	2PT403-POPS CHANNEL
		FAILURE DESCRIPTION: NO ALARM WITH HI PRESS-INVESTIGATE & FIX.
881114082	2	2PT-405-POPS CHANNEL
		FAILURE DESCRIPTION: NO ALARM WITH HI PRESS-INVESTIGATE & FIX.
881116089	2	22 RCP
		FAILURE DESCRIPTION: ADD BALANCE WEIGHTS AS DIRECTED BY MAPLEWOOD LAB.
920301006	2	23 REACTOR COOLANT PUMP SEALS
		FAILURE DESCRIPTION: INSPECT AND OR REPLACE 23 REACTOR COOLANT PUMP SEAL PACKAGE.

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MAJOR PLANT MODIFICATIONS  
REPORT MONTH NOVEMBER 1988

DOCKET NO: 50-311  
UNIT NAME: Salem 2  
DATE: December 10, 1988  
COMPLETED BY: P. White  
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPAL SYSTEM	DESCRIPTION
2EC-1808/A	Circulating Water	This design change upgraded the materials of construction of the strainer shells. The backwash was also revised to be continuous at a reduced flow.
2EC-2085	Fire Protection	This design change installed five 3 hour rated fire dampers in the Auxiliary Building stairwells to prevent the spread of fire between floor elevations via the stairwells.
2SC-0382	Circulating Water	This design change upgraded the circulating water travelling screen contactor panels and bearing lubrication strainer timing panels from carbon steel to fiberglass NEMA 4 enclosures. The internal components were also upgraded.

\* Design Change Request

MAJOR PLANT MODIFICATIONS  
REPORT MONTH - NOVEMBER 1988

DOCKET NO: 50-272  
UNIT NAME: Salem 2  
DATE: December 10, 1988  
COMPLETED BY: P. White  
TELEPHONE: (609) 339-4455

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\*DCR SAFETY EVALUATION 10 CFR 50.59

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2EC-1808/A

This design change upgraded the materials of construction as the previous materials were not suitable for the service water environment and experienced excessive corrosion. The modification to the backwash operation was to reduce strainer clogging and the frequency of manual cleaning. There was no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

2EC-2085

The installation of the fire dampers in the Auxiliary Building stairwells will prevent fire spread between floors via the stairwells. This modification did not alter any plant process or discharge or affect the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

2SC-0382

This design change will eliminate the severe corrosion problems experienced with the control panel enclosures and associated electrical components. There was no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

SALEM GENERATING STATION  
MONTHLY OPERATING SUMMARY - UNIT NO. 2  
NOVEMBER 1988

SALEM UNIT NO. 2

The Unit began the period shutdown for a refueling/maintenance outage. On November 28, 1988, following completion of the outage activities, a unit startup was performed and a power escalation commenced. Later, the same day, the Unit tripped on a lo-lo level condition in No. 22 Steam Generator. The trip was caused by problems with No. 23 feedwater regulating valve. The feedwater regulating valves were adjusted and the process loop tuned and a startup was again commenced. The Unit completed the period operating at reduced power to heat soak the turbine prior to performance of the overspeed trip test.

REFUELING INFORMATION

COMPLETED BY: P. White

DOCKET NO.: 50-311  
UNIT NAME: Salem 2  
DATE: December 10, 1988  
TELEPHONE: 609/935-6000  
EXTENSION 4455

Month November 1988

- 1. Refueling information has changed from last month:  
YES \_\_\_\_\_ NO X
- 2. Scheduled date for next refueling: January 6, 1990
- 3. Scheduled date for restart following refueling: February 20, 1990
- 4. A) Will Technical Specification changes or other license amendments be required?  
YES \_\_\_\_\_ NO \_\_\_\_\_  
NOT DETERMINED TO DATE X
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?  
YES \_\_\_\_\_ NO X

If no, when is it scheduled? November 1989

5. Scheduled date(s) for submitting proposed licensing action:

6. Important licensing considerations associated with refueling:  
NONE

7. Number of Fuel Assemblies:

A) Incore	<u>193</u>
B) In Spent Fuel Storage	<u>308</u>

8. Present licensed spent fuel storage capacity: 1170

Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2003