

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
 Unit Name Salem # 1
 Date November 10, 1987
 Telephone 609-935-6000
 Extension 4451

Completed by Pell White

Month October 1987

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

1	<u>748</u>
2	<u>618</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

P. 8.1-7 R1

8712160040 871031
 PDR ADOCK 05000272
 R DCD

IE2471

OPERATING DATA REPORT

Docket No. .50-272Date: November 10, 1987Telephone: 935-6000Extension: 4451Completed by Pell WhiteOperating Status

	<u>Salem No. 1</u>	<u>Notes</u>
1. Unit Name	<u>October 1987</u>	
2. Reporting Period		
3. Licensed Thermal Power (Mwt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) NA10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>745</u>	<u>7296</u>	<u>90625</u>
12. No. of Hrs. Reactor was Critical	<u>46.3</u>	<u>6412.5</u>	<u>57695.1</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>46.0</u>	<u>6363.2</u>	<u>55791.8</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>107678.4</u>	<u>19612962.8</u>	<u>172269008.8</u>
17. Gross Elec. Energy Generated (MWH)	<u>34790</u>	<u>6496490</u>	<u>57194490</u>
18. Net Elec. Energy Generated (MWH)	<u>27670</u>	<u>6216611</u>	<u>54401379</u>
19. Unit Service Factor	<u>6.2</u>	<u>87.2</u>	<u>61.6</u>
20. Unit Availability Factor	<u>6.2</u>	<u>87.2</u>	<u>61.6</u>
21. Unit Capacity Factor (using MDC Net)	<u>3.4</u>	<u>77.0</u>	<u>54.3</u>
22. Unit Capacity Factor (using DER Net)	<u>3.3</u>	<u>76.4</u>	<u>53.8</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>0.60</u>	<u>24.2</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

CURRENTLY IN A REFUELING OUTAGE25. If shutdown at end of Report Period, Estimated Date of Startup:
12-11-87

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH OCTOBER 1987

Docket No. 50-272
Unit Name Salem No.1
Date NOVEMBER 10, 1987
Telephone 609-935-6000
Extension 4451

Completed by Pell White

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License			Cause and Corrective Action to Prevent Recurrence
						Event Report	System Code 4	Component Code 5	
0285	10-02-87	S	699.0	C	1	---	ZZ	ZZZZZZ	REFUELING OUTAGE

- | | | | | |
|---|--|--|--|--|
| <p>1
F: Forced
S: Scheduled</p> | <p>2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain</p> | <p>3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
 Previous Outage
5-Load Reduction
9-Other</p> | <p>4 Exhibit G
Instructions
for Prepara-
tion of Data
Entry Sheets
for Licensee
Event Report
(LER) File
(NUREG 0161)</p> | <p>5 Exhibit 1
Salem as
Source</p> |
|---|--|--|--|--|

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	UNIT	EQUIPMENT IDENTIFICATION
870327035	1	13 SW STUFFING BOX FAILURE DESCRIPTION: 13 SW STR STUFFING BOX IS BADLY ERODED
870603060	1	1CV361 FAILURE DESCRIPTION: BROKEN VALVE STEM ON 1CB361. OPERATOR BROKE VALVE STEM ATTEMPTING TO OPERATE VALVE.
870630050	1	12SW379 FLANGE FAILURE DESCRIPTION: REPAIR FLANGE LEAK ON 12SW379.
870716091	1	SPRINKLER HEADS FAILURE DESCRIPTION: SPRINKLER HEADS WERE BLOKED AND WOULD NOT FLOW WATER DURING TESTING
870806131	1	12RH55 FAILURE DESCRIPTION: 12RH55, REMOVE VALVE AND REPLACE.

SALEM UNIT 1

WO NO	UNIT	EQUIPMENT IDENTIFICATION
870914007	1	1A,1B,1C EMERG DIESEL GEN FAILURE DESCRIPTION: 1A,1B AND 1C EMERGENCY DIESEL GENERATORS, CLEAN FILTERS WITH ACETONE
870916109	1	13 CFCU FAILURE DESCRIPTION: 13 CFCU HAS NO STOP INDICATION WHEN TAKEN OUT OF SERVICE. TROUBLESHOOT AND REPAIR
870918127	1	1SW191 FAILURE DESCRIPTION: 1SW191 OPENS HALF WAY, STROKE AND REPAIR
870922175	1	1WL23 X-RAY FAILURE DESCRIPTION: PERFORM RADIOGRAPHY ON 1WL23 TO VERIFY POSITION.

SALEM UNIT 1

WO NO	UNIT	EQUIPMENT IDENTIFICATION
870821050	1	15 SW STRAINER
		FAILURE DESCRIPTION: 15 SERVICE WATER STRAINER HAS A BAD PACKING LEAK. REPLACE GLAND AND REPACK.
870824182	1	#11 CHILLER
		FAILURE DESCRIPTION: #11 CHILLER UNIT TRIPS ON MOTOR OVERLOAD. TROUBLESHOOT
870828058	1	1RA4334 RAD MON CABLE
		FAILURE DESCRIPTION: DETECTOR CABLE IS TOO SHORT, CAUSING STRESS ON CABLE. ATTEMPT TO PULL SLACK
870831097	1	CIRC WATER SLUICE GATE
		FAILURE DESCRIPTION: THE ROLL PIN IS BROKEN FOR THE REMOTE OPERATOR.
870911054	1	CONTROL POINT PAGE
		FAILURE DESCRIPTION: THE CONTROL POINT PAGE AND ASSOCIATED SPEAKERS ARE NOT WORKING. REPAIR
870911105	1	1FW64
		FAILURE DESCRIPTION: REPLACE VALVE

MAJOR PLANT MODIFICATIONS
REPORT MONTH OCTOBER 1987

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: November 10, 1987
COMPLETED BY: L. Miller
TELEPHONE: 609/339-4497

<u>*DCR NO.</u>	<u>PRINCIPAL SYSTEM</u>	<u>SUBJECT</u>
1EC-1790	RMS System	Modify circuitry to O-H window "containment ventilation isolation reset with actuation alarm" to include R41 channels.

* DCR - Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH OCTOBER 1987

DOCKET NO. 50-272
UNIT NAME: Salem 1
DATE: November 10, 1987
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR

SAFETY EVALUATION 10 CFR 50.59

1EC-1790

Implementation of this DCR tied the containment ventilation isolation signal from the R41 channels to the overhead annunciator window, "Cont. Vent. Isol. Reset W/Act. Signal" so that the operator will realize that the isolation signal still exists upon reset of the containment ventilation isolation. This eliminates the possibility of an uncontrolled release. This change does not affect any equipment required to shut down the Reactor. This modification does not alter any plant process or discharge and does not affect the existing plant impact. No unreviewed safety or environmental questions are involved.

* DCR - Design Change Request

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy DOCKET NO.: 50-272
 UNIT NAME: Salem 1
 DATE: November 10, 1987
 TELEPHONE: 609/935-6000
 EXTENSION: 4497

Month OCTOBER 1987

1. Refueling information has changed from last month:
 YES X NO _____
2. Scheduled date for next refueling: March 4, 1989
3. Scheduled date for restart following refueling: April 18, 1989
4. A) Will Technical Specification changes or other license amendments be required?
 YES _____ NO _____
 NOT DETERMINED TO DATE X
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
 YES _____ NO X
 If no, when is it scheduled? January 1989
5. Scheduled date(s) for submitting proposed licensing action:
January if required
6. Important licensing considerations associated with refueling:
NONE

7. Number of Fuel Assemblies:
 A) Incore 0
 B) In Spent Fuel Storage 657
8. Present licensed spent fuel storage capacity: 1170
 Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
OCTOBER 1987

The Unit began the period coasting down into a refueling and ISI outage. On October 2, 1987, the Unit was shutdown for refueling. Reactor disassembly, removal of the RTD bypass manifold piping, and installation of bottom mounted thermocouples were subsequently commenced. Major work accomplished during this period includes removal of the reactor head and upper internals, defueling the reactor, removal of the reactor flux thimbles, removal of the reactor lower internals, a secondary hydro of No. 13 SG, and removal of the thermocouple columns.

The tube leak in No. 13 SG has been identified as a leaking plug in a previously plugged tube located near the tube lane blocking device. As part of the normal eddy current examination required by Technical Specifications, two tubes with greater than forty percent tube degradation have been identified and will be plugged. Because of this discovery, the scope of the inspection has been increased an additional 12%.

The electric generator has been lifted three feet for regrouting to improve vibration conditions. The high pressure turbine rotor has been shipped to the Central Maintenance Facility for inspection.

Steam Generator "J" tube wall thinning has been identified during inspection of Nos. 11 & 14 SG's. It has been decided to replace all of the Steam Generator "J" tubes (35 per Steam Generator) with ones made of an upgraded inconel type material.



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

November 10, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of October 1987 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
Major Plant Modification
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

RH:sl

cc: Mr. William T. Russell
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Enclosures
8-1-7.R4

DESIGNATED ORIGINAL

Certified By *J. A. Schwartz*

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