Public Service Electric and Gas Company

Corbin A. McNeill, Jr. Vice President -Nuclear Public Service Electric and Gas Company P.O. Box 236, Hancocks Bridge, NJ 08038 609 339-4800

August 29, 1986

NLR-N86110 Ref: LCR 86-01

Director of Nuclear Reactor Regulation United States Nuclear Regulatory Commission Washington, DC 20555

Attention: Mr. Steven A. Varga, Director Project Directorate #3 Division of PWR Licensing

Gentlemen:

REQUEST FOR AMENDMENT FACILITY OPERATING LICENSES DPR-70 AND DPR-75 UNIT NOS. 1 AND 2 SALEM GENERATING STATION DOCKET NOS. 50-272 AND 50-311

In accordance with the Atomic Energy Act of 1954, as amended and the regulations thereunder, we hereby transmit copies of our request for amendment and our analyses of the changes to Facility Operating Licenses DPR-70 and DPR-75 for Salem Generating Station, Unit Nos. 1 and 2.

This amendment request consists of minor typographical changes to the Technical Specifications.

In accordance with the fee requirements of 10 CFR 170.21, a check in the amount of \$150.00 is enclosed.

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Mr. Steven A. Varga

Pursuant to the requirements of 10 CFR 50.91, a copy of this request for amendment has been sent to the State of New Jersey as indicated below.

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This submittal includes three (3) signed originals and forty (40) copies.

Sincerely,

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Enclosure

C Mr. Donald C. Fischer Licensing Project Manager

> Mr. Thomas J. Kenny Senior Resident Inspector

Mr. Samuel J. Collins, Chief Projects Branch No. 2, DPRP Region 1

Mr. Frank Cosolito, Acting Chief Bureau of Radiation Protection Department of Environmental Protection 380 Scotch Road Trenton, NJ 08628

Honorable Charles M. Oberly, III Attorney General of the State of Delaware Department of Justice 820 North French Street Wilmington, Delaware 19801

# Ref: LCR 86-01

STATE OF NEW JERSEY ) ) SS. COUNTY OF SALEM )

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Corbin A. McNeill, Jr., being duly sworn according to law deposes and says:

I am Vice President of Public Service Electric and Gas Company, and as such, I find the matters set forth in our letter dated Aug. 29, 1986, concerning our Request for Amendment to Facility Operating Licenses DPR-70 and DPR-75, are true to the best of my knowledge, information and belief.

l'audr

Subscribed and Sworn to before me this day of august , 1986 Notary Public of New Jersey LARAINE Y. BEARD

My Commission expires on

Notory Public of New Jersey My Commission Expires May 1, 1991 PROPOSED CHANGE TO TECHNICAL SPECIFICATIONS SALEM UNIT NOS. 1 AND 2

## DESCRIPTION OF CHANGE

Revise Technical Specifications enumerated below to add clarification, to achieve consistency, or to correct typographical errors. The revisions are identified on the attached changed pages.

- Revise Unit 1 and 2 Technical Specification Table 3.3-1, Functional Units 16 and 17 (Reactor Coolant Pumps Undervoltage and Underfrequency). The minimum channels operable is being changed from four to three.
- Revise Technical Specification Tables 3.3-3 and 4.3-2, Functional Unit 8f Emergency Trip of Steam Generator Feedwater Pumps - Start Motor-Driven (Auxiliary Feedwater Pumps). This change adds Mode 2 to APPLICABLE MODES for Unit 1 and corrects the number of channels and ACTION 22 for Unit 2.
- Revise Action Statement 20 in Unit 2 Table 3.3-3, to allow one channel of functional Units 4b and 8a (Steam Line Isolation and Auxiliary Feedwater Automatic Actuation Logic) to be bypassed for 2 hours for surveillance testing.
- 4. Change table notation in Unit 2 Technical Specification Table 3.3-6, for Action Statement number 24 to correct the number of the referenced specification.
- 5. Revise Unit 2 surveillance requirements for turbine overspeed protection, Specification 4.3.4.2, and delete Specification 4.3.4.3.
- 6. Revise Unit 1 Steam Generator surveillance requirement Specifications 4.4.5.2 through 4.4.5.5.
- Revise numbering for Unit 2 Surveillance Requirements
  4.4.7.2 on reactor coolant system operational leakage.
- 8. Revise descriptions of SJ 144 valves in Unit 2 Specification Table 3.4-1.
- Revise Unit 1 Technical Specification 3.5.5, LCO (a) and (b), to correct typographical errors.

## REASON FOR CHANGE

 This change corrects typographical errors in the Technical Specifications. For four-loop plants three channels are required to be operable. The change brings the Specifications into conformance with Westinghouse Standard Technical Specifications (NUREG-0452, Revision 4). 2. The APPLICABILITY requirement was inadvertently listed as MODE 1 for Unit 1. The Unit 2 specification was inadvertently taken directly from the Standard Technical Specifications. This specification is not applicable since it does not reflect the Salem design.

This change is being made to make the Technical Specifications for both units agree and to respond to a specific NRC request for the correction to Unit 1 in Enclosure 1 to Generic Letter 82-16, Status Summary of NUREG-0737 Technical Specifications for Salem 1 and 2. This change supercedes LCR 84-15, dated March 15, 1986.

- 3. This change corrects a typographical error in the action statement, which currently allows only one hour for surveillance testing. The proposed change to two hours makes the requirement the same as that for Unit 1 and brings the specification into conformance with the Westinghouse Standard Technical Specifications.
- 4. Action Statement 24 for inoperable radiation monitoring system channel currently refers to Specification 3.4.6.1, the Specification for steam generators. It is being corrected to refer to Specification 3.4.7.1 for reactor coolant system leak detection.
- 5. The present Specifications 4.3.4.2 and 4.3.4.3 are applicable for turbines with sequential arc admission, whereas Unit 2 turbines have full arc admission. The proposed change to Specification 4.3.4.2 brings the Unit 2 specification into conformance with Westinghouse Standard Technical Specifications and makes Specification 4.3.4.3 superfluous.
- 6. The proposed change makes Unit 1 surveillance requirements the same as those for Unit 2, and brings these requirements into conformance with Westinghouse Standard Technical Specifications.
- 7. Currently, these are two specifications numbered 4.4.7.2. They are being renumbered to 4.4.7.2.1 and 4.4.7.2.2.
- 8. The values are described in the table as being in high-pressure safety injection lines to the RCS hot legs. They are actually in the lines to the cold legs.
- 9. The present Specification states that the volume in the RWST is "between 364,000 and 400,000 of borated water." The lower limit is being raised to 364,500; additionally, the word "gallons" is being added.

The LCO on boron concentration is being corrected from "between 2000 and 2000 ppm" to "between 2000 and 2200 ppm.".

## SIGNIFICANT HAZARDS CONSIDERATION

These changes are administrative in nature in that they are being made to provide clarification, achieve consistency, or correct errors. There are no modifications to the plant or to plant operating procedures. Therefore the proposed amendment does not constitute a significant hazard in that:

- It does not involve a significant increase in the probability or consequences of any previously evaluated accident;
- It does not create the possibility of a new or different kind of accident from any accident previously evaluated;
- It does not involve significant reduction in a margin of safety.

The commission has provided guidance concerning the application of the standards for a No Significant Hazards determination by providing examples of actions not likely to involve a Significant Hazards Consideration in the Federal Register (48FR14870). This proposed amendment corresponds to example (i) for amendments that are not likely to involve significant hazards considerations.

# REVISED PAGES - UNIT NO. 1