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1984 INSERVICE EXAMINATION OF SELECTED CLASS 1 AND CLASS 2 COMPONENTS AND SYSTEMS OF SALEM GENERATING STATION, UNIT 1

VOLUME I
FINAL REPORT
WITH APPENDICES
SwRI Project 4166

Prepared for
Public Service Electric and Gas Company
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September 1984

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ABSTRACT

An inservice examination (ISI) of selected Class 1 and Class 2 components of Public Service Electric and Gas Company's (PSE&G) Salem Generating Station, Unit 1, was performed during the summer 1984 refueling outage. These examinations constituted the sixth ISI performed at Salem, Unit 1, and the first ISI of the third 40-month period of commercial operation. The components were examined in accordance with the "Examination Plan for the 1984 Inservice Examination of Salem Generating Station, Unit 1," including any changes made during the ISI as approved by PSE&G.

The nondestructive examinations were performed using visual (VT), liquid penetrant (PT), magnetic particle (MT), and manual ultrasonic (UT) techniques. These examination techniques revealed indications which were reported to and dispositioned by PSE&G personnel using Customer Notification Forms. The UT examinations also revealed geometric reflectors.

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* Not included in submittal - available on site for review along with Volume II.

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I. INTRODUCTION

During the May through July 1984 refueling outage, Southwest Research Institute (SwRI) personnel performed nondestructive examinations (NDE) of selected components for Public Service Electric and Gas Company (PSE&G) at Salem Generating Station, Unit 1. The examinations constituted the sixth such inservice examination (ISI) performed at Salem Unit 1 and the first ISI of the third 40-month period of commercial operation.

A. Examination Areas

The ISI was performed in accordance with the following documents:

- Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, "Rules for Inservice Inspection of Nuclear Power Plant Components," 1974 Edition with Addenda through Summer 1975.
- SwRI Document 84-PEG-SGS-1/2-1-0: "Project Plan for the Nondestructive Examination Activities to be Performed at Salem Generating Station, Units 1 and 2," dated May 1984.
- SwRI "Examination Plan for the 1984 Inservice Examination of Salem Generating Station, Unit 1," dated April 1984.
- SwRI Nuclear Quality Assurance Program Manual (NQAPM), with all applicable changes.
- U. S. NRC Regulatory Guide 1.14, "Reactor Coolant Pump Flywheel Integrity," Revision 1, August 1975.
- U. S. NRC Regulatory Guide 1.65, "Materials and Inspections for Reactor Vessel Closure Studs," October 1973.
- U. S. NRC Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examination," Revision 1, February 1983.

Representative samples of the following components and areas were examined with NDE techniques.

Class 1

Reactor Pressure Vessel
Reactor Pressure Vessel Closure Head
Pressurizer
Steam Generator No. 13
Steam Generator No. 14
Chemical and Volume Control System
Pressure Relief System
Pressurizing System

Reactor Coolant System
Residual Heat Removal System
Safety Injection System
Reactor Coolant Pumps
Valves

Class 2

Feedwater System
Chemical and Volume Control System
Safety Injection System

B. Summary of Examination Results

The NDE activities were performed using visual (VT), liquid penetrant (PT), magnetic particle (MT), and manual ultrasonic (UT) techniques. Each examination was conducted in accordance with standard SwRI nondestructive testing (NDT) procedures which were written to comply with the requirements of the pertinent sections of the ASME Boiler and Pressure Vessel Code and the SwRI NQAPM. A copy of each applicable SwRI NDT procedure is included in Appendix D.

SwRI operating procedures for weld joint identification marking on nuclear power plant piping, recording data during VT, PT, and MT examinations, and recording search unit location and maximum signal amplitude during UT examinations are included in Appendix C.

During the VT examinations, severe pitting and corrosion was observed on reactor pressure vessel studs numbered 23 through 27 and reported to PSE&G on Customer Notification Form (CNF) 84-1-015. The studs were subsequently rejected by PSE&G and replaced with studs numbered S6 through S10. Studs 37, 44, and 53 were also rejected by PSE&G after failing a tolerance check and were replaced with studs S2, S3, and S4. Baseline MT and UT examinations were performed on the replacement studs.

During the PT piping examinations, indications were observed on nine welds: 2-SJ-1118-17, 1-1/2-SJ-1132-23, 1-1/2-SJ-1122-3, 1-1/2-SJ-1112-15, 2-SJ-1128-41, 2-SJ-1118-29, 2-SJ-1129-25, 2-RC-1114-8, and 2-SJ-1128-74. These indications were reported to PSE&G on CNFs 84-1-001 through 84-1-009, respectively. The indications in each examination area were dispositioned by PSE&G, and reexamination by SwRI personnel revealed no recordable indications.

A 10-year examination of reactor coolant pump (RCP) flywheels, as specified in U.S. NRC Regulatory Guide 1.14, Article C, was performed on RCP flywheels 11, 13, and 14. The elements comprising this 10-year examination included a PT examination on the inner bore and the keyways of the flywheel and MT and UT examinations on the top surface, bottom surface, and outer perimeter of the flywheel. During the PT examinations, indications were recorded on the inner bores of No. 11 and No. 14 flywheels and reported to PSE&G on CNFs 84-1-012 and 84-1-011. After the spot grinding of a one-square-inch section of each of the flywheels, the indications were

resolved as superficial chatter marks and accepted after evaluation by PSE&G. MT indications were observed on the bottom surface of RCP flywheel No. 13 and the top surface of RCP flywheel Nos. 11, 13, and 14. These were reported to PSE&G on CNFs 84-1-010, -013, and -014 (includes top surface of No. 13 and No. 14), respectively. The indications were dispositioned by PSE&G, and reexamination by SwRI personnel revealed no recordable indications. No reportable indications were observed during the UT examination of the flywheels.

During the UT examinations, one reportable indication was observed on 14-BF-2131-23. The indication was resolved by SwRI personnel as a Code-acceptable inclusion and was accepted after evaluation by PSE&G. The remaining UT examinations revealed numerous insignificant and geometric indications; however, no additional reportable UT indications were observed.

No other reportable indications were observed during the examinations.

In addition to the examinations performed by SwRI personnel, VT examinations of selected Class 1, Class 2, and Class 3 hangers were performed and reported separately by PSE&G.

The SwRI Quality Assurance (QA) Section was represented on site by the SwRI QA Representative who was present for approximately 50 percent of the examination period. The QA Representative performed the appropriate duties outlined in Paragraph 3.3.1(4), "Onsite SwRI Quality Assurance Activities," of the Project Plan.

The SwRI examiners were qualified and certified in accordance with SwRI Nuclear Quality Assurance Procedures 11-1 and 11-2 which incorporate the guidelines of SNT-TC-1A of the American Society for Nondestructive Testing. A copy of each individual's certifications is included in Appendix F. Included in Appendix G are copies of the certifications for the ultrasonic instruments, transducers, couplant, and marking pencils used during the examinations.

Drawings of the basic calibration blocks used during the ultrasonic system calibrations are located in Appendix E.

C. Radiation Exposure

Radiation exposure encountered during the ISI was of fundamental concern to the SwRI personnel involved in the daily examination activities. SwRI personnel took the necessary precautions to minimize overall exposure and consequently received the minimum dosage practicable while performing the selected examinations. The following listing details the approximate radiation exposure levels associated with various examination areas.

Area	Exposure Level
CONTAINMENT	
Annulus Area	0-50 mR/hr
Inside Bioshield	20-2000 mR/hr
Pressurizer Enclosure	50-1500 mR/hr
RPV Closure Head	
Inside	3000-5000 mR/hr
Outside	10-50 mR/hr
Refueling Deck	0-20 mR/hr
Regenerative Heat Exchanger Room	100-1000 mR/hr
Steam Generator	10-100 mR/hr
AUXILIARY BUILDING	
Boron Injection Tank Room	10-50 mR/hr
Mechanical Penetration Area	0-10 mR/hr
Charging Safety Injection Pump Rooms	0-20 mR/hr

D. Equipment

1. Sonic FTS MK I (see Figures 1 and 2)

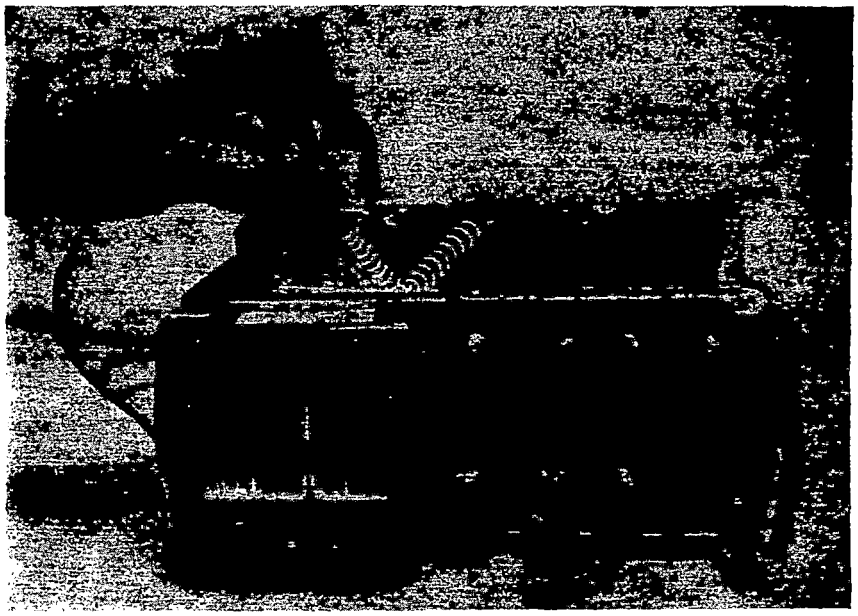
Sonic FTS Mark I UT instruments were used to perform the manual ultrasonic examination of selected components. They also served to determine the characteristics and properties of the components by measuring the transmission and attenuation of ultrasound. The Sonic FTS Mark I instrument is a lightweight portable unit powered by a rechargeable battery and utilizes transistorized circuits and a cathode-ray tube to give a video display representation of the test data.

Instrument calibration certifications for the Sonic FTS MK I are located in Appendix G.

2. Transducers

Various brands, sizes, types, and frequencies of ultrasonic transducers (search units) were used to perform the examinations. Transducer frequencies of 1.5, 2.25, and 5.0 MHz were used. The 2.25- and 5.0-MHz transducers were used on carbon steels, and the 1.5- and 2.25-MHz transducers were used on stainless steels, as specified in the procedure. For information on the actual transducer used for any specific examination, consult the data sheets and referenced calibration sheets in the field data volumes.

Figure 1. Sonic FTS Mark I and pictorial representation of crown indication



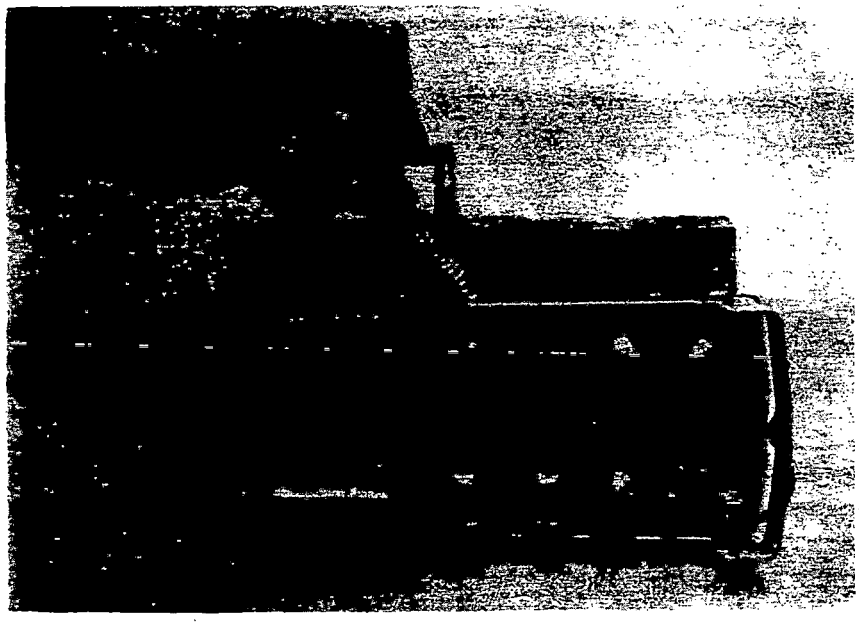


Figure 2. Sonic FTS Mark I and pictorial representation of root indication

II. SUMMARY OF THE INSERVICE EXAMINATION

This section of the report provides a discussion of the field data records and a summary of the NDE activities performed.

A. Explanation of Field Data Records

The results of the NDE activities and calibrations performed by SwRI personnel were recorded on standard SwRI forms. These completed documents constitute a portion of the Inservice Examination Report. The original records are retained in the SwRI Data Storage Facility, and copies are provided herein for completeness.

The field data records for each weld or area are assembled into a package preceded by a summary sheet. The examination areas and summary sheet numbers correspond to those listed in the Summary Table. A general explanation of the individual field data forms follows.

- The instruments used in performing UT examinations were calibrated prior to use, then verified again at specified intervals during the examinations and upon completion of the examinations. The calibration parameters were recorded on the appropriate calibration record sheet as specified in the applicable NDT procedure. The documented calibration and calibration verification provide immediate assurance that the examinations were performed using properly calibrated instruments.

- The results of UT examinations were recorded on the applicable data record sheets as specified in the appropriate NDT procedure. The information documented on these forms describes the parameters associated with those indications which were greater than the recording levels specified in the applicable NDT procedures.

- When required, the size, location, and nature of reflectors were determined by analyzing the indication parameters recorded on the forms described above. The analysis is documented on SwRI Indication Resolution Record Sheets, which are included as a part of each data package.

- VT, PT, and MT Examination Record Sheets were used to record the results of those examinations. Both remote and direct VT examination techniques utilize the same basic format for documenting indications. The equipment and/or materials used in VT, PT, and MT examinations are also identified on the record sheets.

B. Summary of Nondestructive Examinations

The following section of this report is the Summary of Nondestructive Examinations Table (Summary Table). The Summary Table provides information

and results for the NDE activities performed during this ISI. See Figure 3 for an explanation of the Summary Table format. Weld identification figures for Class 1 and Class 2 systems are located in Appendices A and B, respectively.

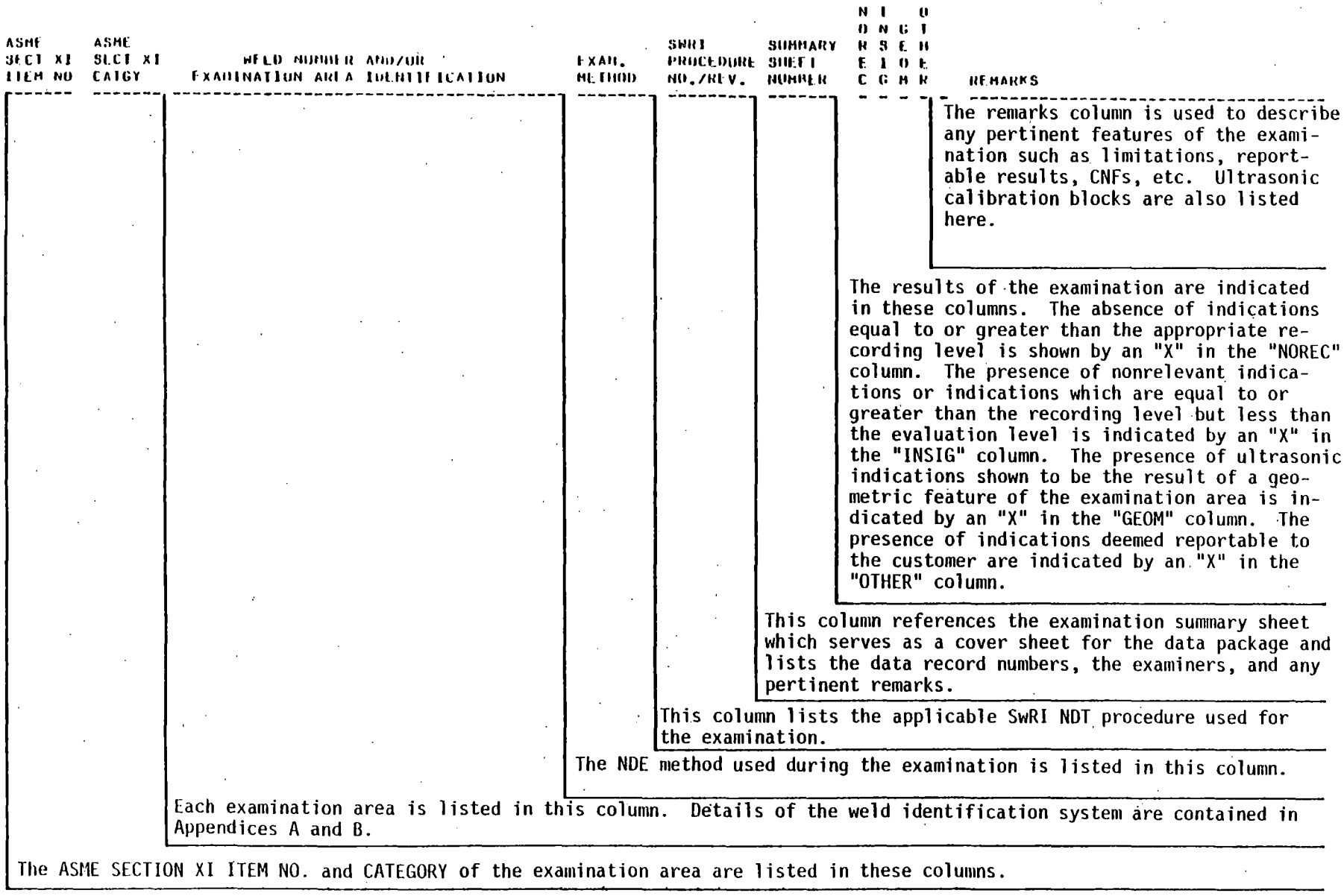


Figure 3. Explanation of Summary Table Format

CLASS 1 COMPONENTS

REACTOR PRESSURE VESSEL (SEE FIGURE A-1)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INDICATIONS GEO	OTHER	REMARKS
		LIGAMENTS BETWEEN THREADED STUD HOLES							
B1.9	B-G-1	1-RPV-LIG 37 THRU 1	VT UTO	900-1/50 600-5/35 DEV. 5	003700	X -	- - - X - -		UT EXAMINATION WAS PERFORMED ON LIGAMENT AREA BETWEEN THE CENTERLINES OF STUD HOLE NO. 37 AND STUD HOLE NO. 1. AT THE REQUEST OF PSE&G VT AND UT EXAMINATIONS WERE PERFORMED BETWEEN THE CENTERLINES OF STUD HOLE NO. 22 AND STUD HOLE NO. 28. ***BASIC CALIBRATION BLOCK*** 7-1.125-8-CS-60-SAM

REACTOR PRESSURE VESSEL CLOSURE HEAD (SEE FIGURE A-2)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSG	OTHER	REMARKS
		MERIDIONAL WELDS IN CLOSURE HEAD							
B1.2	B-B	1-RPV-1046C MERIDIONAL WELD AT 60 DEG.	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/54 DEV. 3	004700	X X X X X X	- - - - - -	- - - - - -	10% (6 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 7-CSCL-50-SAM
B1.2	B-B	1-RPV-1046F MERIDIONAL WELD AT 240 DEG	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/54 DEV. 3	005000	X X X X X X	- - - - - -	- - - - - -	10% (6 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 7-CSCL-50-SAM
		CIRCUMFERENTIAL WELDS							
B1.3	B-C	1-RPV-6046A HEAD TO FLANGE	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/54 DEV. 3	005100	X X X X X X	- - - - - -	- - - - - -	33-1/3% (185.5 INCHES) OF WELD LENGTH WAS EXAMINED FROM THE CENTERLINE OF STUD HOLE NO. 37 TO THE CENTERLINE OF STUD HOLE NO. 1. LIMITED EXAMINATION FROM THE HEAD SIDE DUE TO THE PROXIMITY OF A LIFTING LUG. NO EXAMINATION FROM THE FLANGE SIDE DUE TO THE FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 7-CSCL-50-SAM

REACTOR PRESSURE VESSEL CLOSURE HEAD (SEE FIGURE A-2)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INSIG NIFICANT	OTHER	REMARKS
		CLOSURE STUDS, NUTS, AND WASHERS							
B1.8	B-G-1	1-RPV-STUDS CLOSURE STUDS	VT	900-1/50	005200	-	-	X	SEE CNF 84-1-015. VT INDICATIONS ON STUDS 23-27. STUDS 23-27, 37, 44 AND 53 WERE REJECTED BY PSERG DUE TO SEVERE PITTING AND FAILURE TO PASS A TOLERANCE CHECK AND WERE REPLACED WITH STUDS S2-S4 AND S6-S10. VT EXAMINATIONS WERE PERFORMED ON STUDS 23-27, 38-54, S2-S4, AND S6. BASELINE UT AND MT EXAMINATIONS WERE PERFORMED ON REPLACEMENT STUDS S2-S4 AND S6-S10. ***BASIC CALIBRATION BLOCK*** 7-1.125-8-CS-60-SAM
			MT	300-2/35 DEVS. 4,5		X	-	-	
			UT60	600-18/35 DEV. 6		X	-	-	
B1.8	B-G-1	1-RPV-NUTS CLOSURE NUTS	MT	300-2/35 DEVS. 4,5	005310	X	-	-	NUTS 38-54 WERE EXAMINED.
			UT0	600-19/31		X	-	-	
			UT42	DEV. 4		X	-	-	***BASIC CALIBRATION BLOCK*** 9.563-7-8-CS-59-SAM
B1.10	B-G-1	1-RPV-WASHERS CLOSURE WASHERS	VT	900-1/50	005400	X	-	-	WASHERS 38-54 WERE EXAMINED.
		CLADDING							
B1.13	B-I-1	1-RPVCH-PATCH 5 6 INCH BY 6 INCH PATCH	VT	900-1/50	005900	X	-	-	CENTER OF 6 INCH SQUARE PATCH IS 23 INCHES ABOVE THE BOTTOM EDGE OF THE FLANGE AT THE 225-DEGREE AZIMUTH OF THE HEAD.
			PT	200-1/56 DEV. 2		X	-	-	

REACTOR PRESSURE VESSEL CLOSURE HEAD (SEE FIGURE A-2)

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG EIO MR	OTHER	REMARKS
		CLADDING							
		(CONTD)							
B1.13	B-1-1	1-RPVCH-PATCH 6 6 INCH BY 6 INCH PATCH	VT PT	900-1/50 200-1/56 DEV. 2	006000	X X	- - - - - -	- -	CENTER OF 6 INCH SQUARE PATCH IS 23 INCHES ABOVE THE BOTTOM EDGE OF THE FLANGE AT THE 270-DEGREE AZIMUTH OF THE HEAD.

PRESSURIZER (SEE FIGURE A-3)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INS IGS	GEO MET	OTH ER	REMARKS
		LONGITUDINAL WELDS								
B2.1	B-B	1-PZR-2 LONGITUDINAL WELD SHELL A	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006100	X X X X X X	-	-	-	10% (5.5 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
B2.1	B-B	1-PZR-4 LONGITUDINAL WELD SHELL B	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006125	X X X X X X	-	-	-	10% (5.5 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
B2.1	B-B	1-PZR-8 LONGITUDINAL WELD SHELL D	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006175	X X X X X X	-	-	-	10% (5.5 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
		CIRCUMFERENTIAL WELDS								
B2.1	B-B	1-PZR-1 LOWER HEAD TO SHELL A	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006600	X X X X X X	-	-	-	5% (15 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM

PRESSURIZER (SEE FIGURE A-3)

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REJECTED	INDICATIONS GEOMETRY	OTHER	REMARKS
		CIRCUMFERENTIAL WELDS							
		(CONTD)							
B2.1	B-B	1-PZR-3 SHELL A TO SHELL B	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006625	X X X X X X	- - - - - -	- - - - - -	5% (15 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
B2.1	B-B	1-PZR-7 SHELL C TO SHELL D	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	006675	X X X X X X	- - - - - -	- - - - - -	5% (15 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
		NOZZLE INSIDE RADIUSED SECTION							
B2.2	B-D	4-PRN-1100-IRS RELIEF NOZZLE	UT53	800-55/12	007000	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM

PRESSURIZER (SEE FIGURE A-3)

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSG	OTHER	REMARKS
		NOZZLE INSIDE RADIUSED SECTION							
		(CONTD)							
B2.2	B-D	6-PRN-1103-IRS RELIEF NOZZLE	UT53	800-55/12	007100	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM
B2.2	B-D	6-PRN-1104-IRS RELIEF NOZZLE	UT53	800-55/12	007200	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM
B2.2	B-D	6-PRN-1105-IRS RELIEF NOZZLE	UT53	800-55/12	007300	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM
B2.2	B-D	4-PSN-1131-IRS SPRAY NOZZLE	UT60	800-55/12 DEV. 2	007400	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM
		BOLTING							
B2.11	B-G-2	PZR BOLTING - MANWAY	VT	900-1/50	007820	X	-	-	

STEAM GENERATOR 13 (SEE FIGURE A-6)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	OTHER	REMARKS
		CIRCUMFERENTIAL WELDS							
B3.1	B-B	13-STG-11 LOWER HEAD TO TUBE SHEET	UT0L UT0W UT45 UT45T UT60 UT60T	600-15/57 DEV. B	009700	X X X X X X	-	-	5% (22 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM
		NOZZLE INSIDE RADIUSED SECTION							
B3.2	B-D	31-RCN-1130-IRS	UT30	800-55/12	009800	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM
B3.2	B-D	29-RCN-1130-IRS	UT34	800-55/12	009900	X	-	-	***BASIC CALIBRATION BLOCK*** IR-CSCL-84-SAM

STEAM GENERATOR 14 (SEE FIGURE A-7)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOR EC	INSP GEO	OTHER	REMARKS
		CIRCUMFERENTIAL WELDS							
B3.1	B-B	14-STG-11 LOWER HEAD TO TUBE SHEET	UTOL UTOW UT45 UT45T UT60 UT60T	600-15/57 DEV. 8	010600	X X X X X X	- - - - - - - - - - - - - - - - - -		5% (22 INCHES) OF WELD LENGTH WAS EXAMINED. ***BASIC CALIBRATION BLOCK*** S-CSCL-42-SAM

CHEMICAL AND VOLUME CONTROL SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INGS	OTHER	REMARKS
		3-CV-1143 (SEE FIGURE A-8)							
B4.5	B-J	3-CV-1143-8 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	012900	X - X -	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
B4.5	B-J	3-CV-1143-21 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	014200	X - X -	- X - X	- X - -	***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
		3-CV-1141 (SEE FIGURE A-9)							
B4.5	B-J	3-CV-1141-8 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	016100	X X X -	- - - X	- - - -	***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
B4.5	B-J	3-CV-1141-15 ELBOW TO BRANCH CONNECTION	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	016800	X X - X -	- - X - X	- - - - -	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO BRANCH CONNECTION CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM

CHEMICAL AND VOLUME CONTROL SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSG	EIOM	OTHER	REMARKS
		3-CV-1133 (SEE FIGURE A-10)								
B4.5	B-J	3-CV-1133-9 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	017700	X X X -	- - - X	- - - -	- - - -	***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-50-SAM
		2-CV-1175 (SEE FIGURES A-11, A-12)								
B4.8	B-J	2-CV-1175-12 PIPE TO ELBOW	PT	200-1/56 DEV. 2	019700	X	-	-	-	
B4.8	B-J	2-CV-1175-13 ELBOW TO PIPE	PT	200-1/56 DEV. 2	019800	X	-	-	-	
B4.8	B-J	2-CV-1175-27 COUPLING TO PIPE	PT	200-1/56 DEV. 2	021200	X	-	-	-	
B4.8	B-J	2-CV-1175-29 ELBOW TO PIPE	PT	200-1/56 DEV. 2	021400	X	-	-	-	

PRESSURE RELIEF SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	DEFECT	OTHER	REMARKS
		6-PR-1105 (SEE FIGURE A-13)								
B4.5	B-J	6-PR-1105-6 ELBOW TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	022700	X X X -	- - - X	- - - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-PR-1104 (SEE FIGURE A-14)								
B4.1	B-F	6-PR-1104-1 NOZZLE TO SAFE-END	PT UTOW UT45T	200-1/56 DEV. 2 600-3/61 DEV. 12	023400	X X X	- - -	- - -	- - -	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDES DUE TO THE NOZZLE AND SAFE-END CONFIGURATIONS. ***BASIC CALIBRATION BLOCK*** 6-SS-XX-1.5-64-SAM
B4.5	B-J	6-PR-1104-7 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	024000	X - X -	- - - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-PR-1103 (SEE FIGURE A-15)								
B4.1	B-F	6-PR-1103-1 NOZZLE TO SAFE-END	PT UTOW UT45T	200-1/56 DEV. 2 600-3/61 DEV. 12	024600	X X X	- - -	- - -	- - -	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDES DUE TO THE NOZZLE AND SAFE-END CONFIGURATIONS. ***BASIC CALIBRATION BLOCK*** 6-SS-XX-1.5-64-SAM

PRESSURE RELIEF SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INSIG 1	INSIG 2	INSIG 3	OTHER	REMARKS
		6-PR-1103 (SEE FIGURE A-15)									
		(CONTD)									
B4.5	B-J	6-PR-1103-8 PIPE TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	025300	X - X -	- - - X	- X - -	- - - -		***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
B4.12	B-G-2	6-PR-1103-12FB FLANGE BOLTING	VT	900-1/50	025800	X	-	-	-		
		4-PR-1100 (SEE FIGURE A-16)									
B4.5	B-J	4-PR-1100-4 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	026200	X X X X	- - - -	- - - -	- - - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
		3-PR-1107 (SEE FIGURE A-16)									
B4.5	B-J	3-PR-1107-2 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	027000	X - X -	- X - X	- - - -	- - - -		***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM

PRESSURE RELIEF SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INSIG R	EXAM I	OTHER E	REMARKS
		3-PR-1107 (SEE FIGURE A-16)								
		(CONTD)								
B4.5	B-J	3-PR-1107-10 PIPE TO VALVE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	027800	X X X X	 X 	 	 	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO VALVE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
		3-PR-1106 (SEE FIGURE A-17)								
B4.5	B-J	3-PR-1106-7 ELBOW TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	028900	X X X 	 X	 	 	***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM

PRESSURIZING SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	ING SE	OTHER	REMARKS
		4-PS-1131 (SEE FIGURES A-19, A-20, A-21)							
B4.5	B-J	4-PS-1131-1 BRANCH CONNECTION TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	030100	X X X X	- - - - - - - - - - - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-PS-1131-12 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	031700	X X X -	- - - - - - - - - X - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-PS-1131-13 ELBOW TO VALVE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	031900	X X X X X	- - - - - - - - - - - - - - -		NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO VALVE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-PS-1131-24 TEE TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	033000	X X X X X	- - - - - - - - - - - - - - -		NO EXAMINATION FROM THE UP-STREAM SIDE DUE TO TEE CONFIG-URATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.1	B-F	4-PS-1131-29 SAFE-END TO NOZZLE	PT UTOW	200-1/56 DEV. 2 600-3/61 DEV. 12	033500	X X	- - - - - -		NO EXAMINATION FROM THE UP-STREAM OR DOWNSTREAM SIDES DUE TO THE NOZZLE AND SAFE-END CONFIGURATIONS. NO UT45T SCAN DUE TO WELD CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13

PRESSURIZING SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG MAJ	EXTENT OF EXAM	REMARKS
		4-PS-1131 (SEE FIGURES A-19, A-20, A-21)							
		(CONTD)							
B4.9	B-K-1	4-PS-1111-14PL PIPE LUG	UTOL UTOW UT45 UT45T	600-3/61 DEV. 12	035600	X X - X	- - X -	- - - -	NO UT45T SCAN ON WELD DUE TO WELD CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
		4-PS-1111 (SEE FIGURE A-22)							
B4.5	B-J	4-PS-1111-18 PIPE TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	036000	X X X X X	- - - - -	- - - - -	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE INNER RADIUS OF THE BENT PIPE. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-PS-1111-23 VALVE TO TEE	PT UTOW UT45T	200-1/56 DEV. 2 600-3/61 DEV. 12	036500	X X X	- - -	- - -	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDE DUE TO VALVE AND TEE CONFIGURATIONS. PT WAS PERFORMED AS A SUPPLEMENTAL EXAMINATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13

REACTOR COOLANT SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SNRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	EOMR	HER	REMARKS
		31-RC-1130 (SEE FIGURE A-24)								
B4.1	B-F	31-RC-1130-2 NOZZLE TO ELBOW	PT	200-1/56 DEV. 2	039200	X	-	-	-	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDE DUE TO ELBOW ACOUSTICAL PROPERTIES AND PUMP CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 2.312-SS-37-SAM
			UTOW	600-3/61 DEV. 12		X	-	-	-	
			UT45T			X	-	-	-	
		31-RC-1120 (SEE FIGURE A-25)								
B4.5	B-J	31-RC-1120-5 ELBOW TO PIPE	UTOL	600-3/61 DEV. 12	043650	X	-	-	-	LIMITED EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO AN ADJACENT BRANCH CONNECTION. NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE ACOUSTICAL PROPERTIES OF THE ELBOW. ***BASIC CALIBRATION BLOCK*** 2.312-SS-37-SAM
			UTOW			X	-	-	-	
			UT45			X	-	-	-	
			UT45T			X	-	-	-	
			UT60			X	-	-	-	
		29-RC-1140 (SEE FIGURE A-27)								
B4.5	B-J	29-RC-1140-3 PIPE TO PIPE	UTOL	600-3/61 DEV. 12	047600	X	-	-	-	***BASIC CALIBRATION BLOCK*** 2.312-SS-37-SAM
			UT45			-	X	-	-	
			UT45T			X	-	-	-	
			UT60			-	X	-	-	
B4.7	B-J	29-RC-1140-3/6-SJ-1141 6 IN. BRANCH CONNECTION	PT	200-1/56 DEV. 2	047800	X	-	-	-	

REACTOR COOLANT SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INSG	EIOM	OTHER	REMARKS
		29-RC-1130 (SEE FIGURE A-28)								
B4.1	B-F	29-RC-1130-5 ELBOW TO NOZZLE	PT	200-1/56 DEV. 2	050210	X	-	-	-	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDE DUE TO ELBOW ACOUSTICAL PROPERTIES AND NOZZLE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 2.312-SS-37-SAM
			UTOW	600-3/61		X	-	-	-	
			UT45T	DEV. 12		X	-	-	-	
		27-1/2-RC-1130 (SEE FIGURE A-32)								
B4.5	B-J	27.5-RC-1130-2 PIPE TO PIPE	UTOL	600-3/61	056600	X	-	-	-	***BASIC CALIBRATION BLOCK*** 2.312-SS-37-SAM
			UT45	DEV. 12		-	X	-	-	
			UT45T			X	-	-	-	
			UT60			-	X	-	-	
		27-1/2-RC-1110 (SEE FIGURE A-34)								
B4.7	B-J	27.5-RC-1110-1/4-PS-1111 4 IN. BRANCH CONNECTION	PT	200-1/56 DEV. 2	059100	X	-	-	-	
		3-RC-1143 (SEE FIGURE A-35)								
B4.5	B-J	3-RC-1143-17 ELBOW TO VALVE	UTOL	600-3/61	061800	X	-	-	-	NO EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO VALVE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
			UTOW	DEV. 12		X	-	-	-	
			UT45			-	X	X	-	
			UT45T			X	-	-	-	
			UT60			-	X	-	-	

REACTOR COOLANT SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	NTIG	GSEM	OTHER	REMARKS
		3-RC-1133 (SEE FIGURE A-36)								
B4.5	B-J	3-RC-1133-6 ELBOW TO PIPE	UTOL	600-3/61	062500	X	-	-	-	
			UT45	DEV. 12		-	X	-	-	
			UT45T			X	-	-	-	
			UT60			X	-	-	-	
										BASIC CALIBRATION BLOCK 3-SS-160-.451-30-SAM
B4.5	B-J	3-RC-1133-18 VALVE TO BRANCH CONNECTION	PT	200-1/56 DEV. 2	063800	X	-	-	-	NO UT FROM THE UPSTREAM OR DOWNSTREAM SIDE DUE TO VALVE AND BRANCH CONNECTION CONFIGURATIONS. PT WAS PERFORMED AS A SUPPLEMENTAL EXAMINATION, ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
			UTOW	600-3/61		X	-	-	-	
			UT45T	DEV. 12		X	-	-	-	
		3-RC-1123 (SEE FIGURE A-37)								
B4.5	B-J	3-RC-1123-14 PIPE TO FLANGE	UTOL	600-3/61	065200	X	-	-	-	NO EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
			UTOW	DEV. 12		X	-	-	-	
			UT45			X	-	-	-	
			UT45T			X	-	-	-	
			UT60			-	X	-	-	
B4.12	B-G-2	3-RC-1123-14FB FLANGE BOLTING	VT	900-1/50	065300	X	-	-	-	

REACTOR COOLANT SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG SEIG	OTHER	REMARKS
		3-RC-1123 (SEE FIGURE A-37)							
		(CONTD)							
B4.5	B-J	3-RC-1123-15 FLANGE TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	065400	X - - - X - - - X - - - X - - - X - - -			NO EXAMINATION FROM THE UP-STREAM SIDE DUE TO FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
		3-RC-1113 (SEE FIGURE A-38)							
B4.5	B-J	3-RC-1113-14 PIPE TO FLANGE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	067100	X - - - X - - - - X - - X - - - - X - -			NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
B4.12	B-G-2	3-RC-1113-14FB FLANGE BOLTING	VT	900-1/50	067200	X - - -			
		2-RC-1144 (SEE FIGURE A-39)							
B4.8	B-J	2-RC-1144-13 PIPE TO TEE	PT	200-1/56 DEV. 2	068900	X - - -			

REACTOR COOLANT SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	OTHER	REMARKS
		2-RC-1142 (SEE FIGURE A-40)							
B4.8	B-J	2-RC-1142-20 PIPE TO ELBOW	PT	200-1/56 DEV. 2	071500	X	-	-	
B4.8	B-J	2-RC-1142-21 ELBOW TO PIPE	PT	200-1/56 DEV. 2	071600	X	-	-	
		2-RC-1141 (SEE FIGURE A-47)							
B4.5	B-J	2-RC-1141-1 TEE TO PIPE	PT UTOL UT45 UT45T	200-1/56 600-39/2 DEV. 4	071800	X X X X	- - - -	- - - -	LIMITED UT FROM THE DOWNSTREAM SIDE DUE TO THE PROXIMITY OF WELD 2-RC-1141-2. NO UT FROM THE UPSTREAM SIDE DUE TO TEE CONFIGURATION. PT WAS PERFORMED AS A SUPPLEMENTAL EXAMINATION. ***BASIC CALIBRATION BLOCK*** 2-SS-160-.330-39-SAM
		2-RC-1134 (SEE FIGURE A-41)							
B4.8	B-J	2-RC-1134-10 VALVE TO PIPE	PT	200-1/56 DEV. 2	073100	X	-	-	
		2-RC-1132 (SEE FIGURE A-42)							
B4.8	B-J	2-RC-1132-6 PIPE TO VALVE	PT	200-1/56 DEV. 2	074500	X	-	-	

REACTOR COOLANT SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	OTHER	REMARKS
		2-RC-1132 (SEE FIGURE A-42)							
		(CONTD)							
B4.8	B-J	2-RC-1132-15 TEE TO PIPE	PT	200-1/56 DEV. 2	075400	X	-	-	
B4.5	B-J	2-RC-1132-17N FLANGE TO PIPE	UTOL UT45 UT45T	600-39/2 DEV. 4	075605	X X X	- - -	- - -	BASELINE EXAMINATION. NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 2-SS-160-.330-39-SAM
		2-RC-1131 (SEE FIGURE A-47)							
B4.8	B-J	2-RC-1131-5 TEE TO REDUCER	PT	200-1/56 DEV. 2	076700	X	-	-	
		2-RC-1124 (SEE FIGURE A-43)							
B4.8	B-J	2-RC-1124-11 PIPE TO VALVE	PT	200-1/56 DEV. 2	078000	X	-	-	
		2-RC-1122 (SEE FIGURE A-44)							
B4.8	B-J	2-RC-1122-4 PIPE TO ELBOW	PT	200-1/56 DEV. 2	079100	X	-	-	

REACTOR COOLANT SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE FIG MOR CAT	REMARKS
		2-RC-1122 (SEE FIGURE A-44)					
		(CONTD)					
B4.12	B-G-2	2-RC-1122-16FB FLANGE BOLTING	VT	900-1/50	080400	X - - -	
B4.5	B-J	2-RC-1122-17N FLANGE TO PIPE	UTOL UT45T	600-39/2 DEV. 4	080405	X - - - X - - -	BASELINE EXAMINATION. LIMITED UTOL AND UTOW SCANS AND NO UT45 SCAN FROM THE DOWNSTREAM SIDE DUE TO THE PROXIMITY OF WELD 2-RC-1122-18. NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 2-SS-160-.330-39-SAM
		2-RC-1114 (SEE FIGURE A-45)					
B4.8	B-J	2-RC-1114-8 TEE TO PIPE	PT PT	200-1/56 DEV. 2	082200	- - - X X - - -	SEE CNF 84-1-008. ONE LINEAR INDICATION. RE-EXAMINATION AFTER COSMETIC BUFFING BY PSE&G REVEALED NO RECORDABLE INDICATIONS.
B4.8	B-J	2-RC-1114-9 PIPE TO VALVE	PT	200-1/56 DEV. 2	082300	X - - -	

REACTOR COOLANT SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	EOMI	OTHER	REMARKS
		2-RC-1112 (SEE FIGURE A-46)								
B4.8	B-J	2-RC-1112-1 BRANCH CONNECTION TO PIPE	PT	200-1/56 DEV. 2	083300	X	-	-	-	
B4.8	B-J	2-RC-1112-4 PIPE TO ELBOW	PT	200-1/56 DEV. 2	083600	X	-	-	-	
B4.12	B-G-2	2-RC-1112-16FB FLANGE BOLTING	VT	900-1/50	084900	X	-	-	-	
B4.5	B-J	2-RC-1112-17N FLANGE TO PIPE	UTOL UT45 UT45T	600-39/2 DEV. 4	085005	X X X	- - -	- - -	- - -	BASELINE EXAMINATION. NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO FLANGE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 2-SS-160-.330-39-SAM

RESIDUAL HEAT REMOVAL SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG JOM	OTHER	REMARKS
		14-RH-1111 (SEE FIGURE A-52)							
B4.7	B-J	14-RH-1111-1/6-SJ-1111 6 IN. BRANCH CONNECTION	PT	200-1/56 DEV. 2	092300	X	-	-	
B4.5	B-J	14-RH-1111-4 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	092600	X - X -	- X - X	- X - -	***BASIC CALIBRATION BLOCK*** 14-SS-160-1.400-78-SAM
B4.5	B-J	14-RH-1111-7 ELBOW TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	092900	X X - X -	- - X - X	- - - - -	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO A WELDED METAL RING. ***BASIC CALIBRATION BLOCK*** 14-SS-160-1.400-78-SAM
B4.5	B-J	14-RH-1111-14 VALVE TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	093600	X - X -	- X - X	- - - -	NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO VALVE CONFIGURATION. NO UTOW OR UT45T SCAN ON WELD DUE TO WELD CONFIGURATION. EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 14-SS-160-1.400-78-SAM
B4.5	B-J	14-RH-1111-15 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	093700	X - X -	- X - X	- - - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 14-SS-160-1.400-78-SAM

RESIDUAL HEAT REMOVAL SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOZZLE	INSIDE SEAM	OUTSIDE GEOMETRY	OUTSIDE HEAT AFFECTED ZONE	REMARKS
		14-RH-1111 (SEE FIGURE A-52)								
		(CONTD)								
B4.5	B-J	14-RH-1111-16 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	093800	X - X -	- X - X	- - - X	- - - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 14-SS-160-1.400-78-SAM

SAFETY INJECTION SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE (REC)	INSIG (RSIG)	OTHER (EIOMR)	REMARKS
		10-SJ-1141 (SEE FIGURE A-54)							
B4.5	B-J	10-SJ-1141-9 PIPE TO ELBOW	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	096200	X X - X -	- - X - X	- - X - X	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE PROXIMITY OF AN ADJACENT WELDED RING. ***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1141-10 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	096300	X - X -	- X - X	- X - X	***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
		10-SJ-1131 (SEE FIGURE A-55)							
B4.9	B-K-1	10-SJ-1131-5PS PENETRATION TO PIPE	UTOL UTOW UT45 UT45T	600-3/61 DEV. 12	098000	X X X X	- - - -	- - - -	NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO PENETRATION CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1131-6 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	098100	X - X -	- X - -	- X - X	***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INSIG S	EXTENT O	OTHER R	REMARKS
		10-SJ-1131 (SEE FIGURE A-55)								
		(CONTD)								
B4.5	B-J	10-SJ-1131-8 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	098300	X - X -	- X - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1131-9 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	098400	X - X -	- X - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
		10-SJ-1121 (SEE FIGURE A-56)								
B4.5	B-J	10-SJ-1121-3 PIPE TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	099600	X - X -	- X - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1121-8 ELBOW TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	100200	X X - X X	- - X - -	- - X - -	- - - - -	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO THE PROX-IMITY OF THE PENETRATION SLEEVE. ***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INGS R	IEG I	OTHE M	OTHE R	REMARKS
		10-SJ-1121 (SEE FIGURE A-56)									
		(CONTD)									
B4.5	B-J	10-SJ-1121-13 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	100800	X - X -	- X - X	- - - X	- - - -		***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
		10-SJ-1111 (SEE FIGURE A-57)									
B4.5	B-J	10-SJ-1111-3 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	102100	X - X -	- X - X	- - - X	- - - -		***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1111-10 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	102900	X - X -	- X - X	- X - X	- - - -		***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM
B4.5	B-J	10-SJ-1111-18 VALVE TO ELBOW	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	103700	X X - X -	- - - - X	- - X - -	- - - - -		NO EXAMINATION FROM THE UP-STREAM SIDE DUE TO VALVE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 10-SS-160-1.119-22-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INSIG E	OTHER	REMARKS
		8-SJ-1162 (SEE FIGURES A-58, A-59)							
B4.5	B-J	8-SJ-1162-1 VALVE TO PIPE	UTOL	600-3/61	104200	X	-	-	NO EXAMINATION FROM THE UP-
			UTOW	DEV. 12		X	-	-	STREAM SIDE DUE TO VALVE
			UT45			-	X	-	CONFIGURATION.
			UT45T			X	-	-	
			UT60			-	X	-	
									BASIC CALIBRATION BLOCK 8-SS-XX-.860-23-SAM
B4.5	B-J	8-SJ-1162-13 ELBOW TO PIPE	UTOL	600-3/61	105800	X	-	-	LIMITED EXAMINATION FROM THE
			UTOW	DEV. 12		X	-	-	DOWNSTREAM SIDE DUE TO AN
			UT45			-	X	-	ADJACENT BRANCH CONNECTION.
			UT45T			X	-	-	
			UT60			-	X	-	
									BASIC CALIBRATION BLOCK 8-SS-XX-.860-23-SAM
B4.5	B-J	8-SJ-1162-23 PIPE TO PIPE	UTOL	600-3/61	106900	X	-	-	
			UT45	DEV. 12		-	X	-	
			UT45T			X	-	-	
			UT60			-	X	-	
									BASIC CALIBRATION BLOCK 8-SS-XX-.860-23-SAM
		8-SJ-1152 (SEE FIGURES A-60, A-61)							
B4.9	B-K-1	8-SJ-1152-1PL-3 PIPE LUG	PT	200-1/56	108700	X	-	-	LIMITED UT DUE TO WELDED
				DEV. 2					HANGER BRACKET. NO UTOW DUE
			UTOL	600-3/61		X	-	-	TO WELD AREA CONFIGURATION.
			UT45	DEV. 12		X	-	-	PT WAS PERFORMED AS A SUPPLE-
			UT45T			X	-	-	MENTAL EXAMINATION.
									BASIC CALIBRATION BLOCK 8-SS-XX-.860-23-SAM

SAFETY INJECTION SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	EMOT	OTHER	REMARKS
		8-SJ-1152 (SEE FIGURES A-60, A-61)								
		(CONTD)								
B4.5	B-J	8-SJ-1152-3 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	109200	X - X -	- X - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 8-SS-XX-.860-23-SAM
B4.5	B-J	8-SJ-1152-10 PIPE TO ELBOW	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	110300	X X - - -	- - X X X	- - X X X	- - - - -	LIMITED EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO THE PROXIMITY OF A PIPE SUPPORT AND WELD 8-SJ-1152-11. ***BASIC CALIBRATION BLOCK*** 8-SS-XX-.860-23-SAM
B4.5	B-J	8-SJ-1152-19 PIPE TO TEE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	111300	X X - X -	- - X - X	- - X - X	- - - - -	LIMITED EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO THE TEE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 8-SS-XX-.860-23-SAM
		8-SJ-1145 (SEE FIGURE A-62)								
B4.5	B-J	8-SJ-1145-8 PIPE TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	112400	X - X -	- X - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 8-SS-XX-.860-23-SAM

SAFETY INJECTION SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG	EVAL	OTHER	REMARKS
		8-SJ-1135 (SEE FIGURE A-63)								
B4.5	B-J	8-SJ-1135-6 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	113600	X - X -	- - - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 8-SS-XX-.860-23-SAM
		6-SJ-1142 (SEE FIGURE A-64)								
B4.5	B-J	6-SJ-1142-10 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	114800	X - X -	- X - X	- - - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
B4.5	B-J	6-SJ-1142-11 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	114900	X - X -	- - - X	- X - X	- - - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-SJ-1141 (SEE FIGURE A-65)								
B4.5	B-J	6-SJ-1141-6 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	115800	X - X -	- X - X	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	EMOT	HER	REMARKS
		6-SJ-1132 (SEE FIGURE A-66)								
B4.7	B-J	6-SJ-1132-3/2-SJ-1139 2 IN. BRANCH CONNECTION	PT	200-1/56 DEV. 2	117400	X	-	-	-	
B4.5	B-J	6-SJ-1132-6 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	117700	X - X -	- X - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
B4.5	B-J	6-SJ-1132-11 ELBOW TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	118400	X X - X -	- - X - -	- - X - X	- - - - -	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE PROXIMITY OF WELD 6-SJ-1132-10 ***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-SJ-1131 (SEE FIGURE A-67)								
B4.5	B-J	6-SJ-1131-2 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	118900	X - X -	- X - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-SJ-1122 (SEE FIGURE A-68)								
B4.5	B-J	6-SJ-1122-3 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	121200	X - X -	- X - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE C	INGS I	IGEM U	IOTH R	REMARKS
		6-SJ-1121 (SEE FIGURE A-69)								
B4.5	B-J	6-SJ-1121-4 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	122500	X - X -	- X - X	- - - X	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		6-SJ-1112 (SEE FIGURE A-70)								
B4.5	B-J	6-SJ-1112-3 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	123000	X - X -	- X - X	- X - -	- - - -	***BASIC CALIBRATION BLOCK*** 6-SS-160-.764-25-SAM
		4-SJ-1194 (SEE FIGURE A-73)								
B4.5	B-J	4-SJ-1194-6 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	125200	X X X X	- - - -	- - - -	- - - -	***BASIC CALIBRATION BLOCK*** 4-SS-160-.556-13
B4.9	B-K-1	4-SJ-1194-7PS-2 PENETRATION TO PIPE	UTOL UT45 UT45T	600-3/61 DEV. 12	125500	X - X	- X -	- - -	- - -	NO EXAMINATION FROM THE UP-STREAM SIDE DUE TO PENETRATION CONFIGURATION. NO UTOW SCAN DUE TO THE WELD CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 4-SS-XX-.689-27-SAM

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INGSEIOM	OTHER	REMARKS
		4-SJ-1194 (SEE FIGURE A-73)							
		(CONTD)							
B4.5	B-J	4-SJ-1194-8 PIPE TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	125600	X - X -	- X - X	- - - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-SJ-1194-9 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	125700	X X X X	- - - -	- - - -	***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-SJ-1194-21 TEE TO PIPE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	126900	X X - X -	- - X - X	- - - - -	NO EXAMINATION FROM THE UP-STREAM SIDE DUE TO THE TEE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
		4-SJ-1182 (SEE FIGURE A-74)							
B4.5	B-J	4-SJ-1182-4 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	128000	X X X -	- - - X	- - - -	***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG MAJ	OTHER	REMARKS
		4-SJ-1182 (SEE FIGURE A-74)							
		(CONTD)							
B4.5	B-J	4-SJ-1182-10 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	128600	X X X -	- - - - - - - - - X - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-SJ-1182-13 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	128900	X X X -	- - - - - - - - - X - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
B4.5	B-J	4-SJ-1182-22 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	130000	X - X -	- - - X - - - - - X - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
		4-SJ-1172 (SEE FIGURE A-75)							
B4.5	B-J	4-SJ-1172-9 ELBOW TO PIPE	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	131600	X X X -	- - - - - - - - - X - -		***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REF	INSP CG	OTHER	REMARKS
		4-SJ-1172 (SEE FIGURE A-75)							
		(CONTD)							
B4.5	B-J	4-SJ-1172-18 PIPE TO ELBOW	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	132500	X - X -	- X - X	- - - -	***BASIC CALIBRATION BLOCK*** 4-SS-160-.536-13
		3-SJ-1192 (SEE FIGURE A-72)							
B4.5	B-J	3-SJ-1192-12 PIPE TO VALVE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	135300	X X - - -	- - X X X	- - - - -	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO VALVE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 3-SS-160-.451-30-SAM
		2-SJ-1149 (SEE FIGURES A-76, A-77)							
B4.8	B-J	2-SJ-1149-5 VALVE TO PIPE	PT	200-1/56 DEV. 2	136400	X	-	-	
B4.8	B-J	2-SJ-1149-17 PIPE TO ELBOW	PT	200-1/56 DEV. 2	137600	X	-	-	

SAFETY INJECTION SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INDICATIONS SURF	INDICATIONS GEO	OTHER	REMARKS
		2-SJ-1149 (SEE FIGURES A-76, A-77)								
		(CONTD)								
B4.8	B-J	2-SJ-1149-37 PIPE TO ELBOW	PT	200-1/56 DEV. 2	139600	X	-	-	-	
		2-SJ-1147 (SEE FIGURE A-78)								
B4.8	B-J	2-SJ-1147-1 BRANCH CONNECTION TO PIPE	PT	200-1/56 DEV. 2	140700	X	-	-	-	
B4.8	B-J	2-SJ-1147-8 PIPE TO FLANGE	PT	200-1/56 DEV. 2	141400	X	-	-	-	
B4.8	B-J	2-SJ-1147-13 VALVE TO PIPE	PT	200-1/56 DEV. 2	142000	X	-	-	-	
		2-SJ-1139 (SEE FIGURE A-79)								
B4.8	B-J	2-SJ-1139-8 PIPE TO ELBOW	PT	200-1/56 DEV. 2	144300	X	-	-	-	
B4.8	B-J	2-SJ-1139-33 ELBOW TO BRANCH CONNECTION	PT	200-1/56 DEV. 2	146800	X	-	-	-	

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE (REC)	INSIG (NSIG)	OTHER	REMARKS
		2-SJ-1137 (SEE FIGURE A-80)							
B4.8	B-J	2-SJ-1137-7 ELBOW TO PIPE	PT	200-1/56 DEV. 2	147500	X	-	-	
B4.8	B-J	2-SJ-1137-8 PIPE TO ELBOW	PT	200-1/56 DEV. 2	147600	X	-	-	
B4.8	B-J	2-SJ-1137-20 ELBOW TO PIPE	PT	200-1/56 DEV. 2	148900	X	-	-	
B4.8	B-J	2-SJ-1137-32 COUPLING TO BRANCH CONNCTN	PT	200-1/56 DEV. 2	150100	X	-	-	
		2-SJ-1129 (SEE FIGURES A-81, A-82)							
B4.8	B-J	2-SJ-1129-13 TEE TO REDUCER	PT	200-1/56 DEV. 2	151500	X	-	-	
B4.8	B-J	2-SJ-1129-25 PIPE TO ELBOW	PT PT	200-1/56 DEV. 2	152700	- X	- -	X -	SEE CNF 84-1-007. ONE LINEAR AND ONE ROUND INDICATION. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE INDICATIONS WERE OBSERVED.
B4.8	B-J	2-SJ-1129-37 PIPE TO COUPLING	PT	200-1/56 DEV. 2	153900	X	-	-	

SAFETY INJECTION SYSTEM
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG EIG	OTHER	REMARKS
		2-SJ-1129 (SEE FIGURES A-81, A-82)							
		(CONTD)							
B4.8	B-J	2-SJ-1129-48 COUPLING TO BRANCH CONNCTN	PT	200-1/56 DEV. 2	155000	X	-	-	
		2-SJ-1128 (SEE FIGURES A-83, A-84, A-85)							
B4.8	B-J	2-SJ-1128-4 PIPE TO VALVE	PT	200-1/56 DEV. 2	155700	X	-	-	
B4.8	B-J	2-SJ-1128-18 ELBOW TO PIPE	PT	200-1/56 DEV. 2	157100	X	-	-	
B4.8	B-J	2-SJ-1128-30 PIPE TO TEE	PT	200-1/56 DEV. 2	158300	X	-	-	
B4.8	B-J	2-SJ-1128-41 COUPLING TO PIPE	PT PT	200-1/56 DEV. 2	159500	- X	-	X -	SEE CNF 84-1-005. SIX LINEAR INDICATIONS. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE INDICATIONS WERE OBSERVED.
B4.8	B-J	2-SJ-1128-53 ELBOW TO PIPE	PT	200-1/56 DEV. 2	160700	X	-	-	

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE RECORDABLE INDICATIONS	INSIGNIFICANT INDICATIONS	GEOMETRIC	UNACCEPTABLE INDICATIONS	REMARKS
		2-SJ-1128 (SEE FIGURES A-83, A-84, A-85)								
		(CONTD)								
B4.5	B-J	2-SJ-1128-74 PIPE TO REDUCER	PT PT UTOL UT45 UT45T	200-1/56 DEV. 2 600-39/2 DEV. 4	162800	- X X X X	- - - - -	- - - - -	X - - - -	SEE CNF 84-1-009. ONE LINEAR INDICATION. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE INDICATIONS WERE OBSERVED. NO UT FROM THE DOWNSTREAM SIDE DUE TO THE REDUCER CONFIGURATION. PT WAS PERFORMED AS A SUPPLEMENTAL EXAMINATION. ***BASIC CALIBRATION BLOCK*** 2-SS-160-.330-39-SAM
		2-SJ-1119 (SEE FIGURE A-86)								
B4.12	B-G-2	2-SJ-1119-2FB FLANGE BOLTING	VT	900-1/50	163100	X	-	-	-	
B4.8	B-J	2-SJ-1119-5 VALVE TO PIPE	PT	200-1/56 DEV. 2	163400	X	-	-	-	
B4.8	B-J	2-SJ-1119-17 PIPE TO ELBOW	PT	200-1/56 DEV. 2	164600	X	-	-	-	
B4.8	B-J	2-SJ-1119-23 PIPE TO ELBOW	PT	200-1/56 DEV. 2	165200	X	-	-	-	

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG	OTHER	REMARKS
		2-SJ-1118 (SEE FIGURES A-87, A-88)							
B4.12	B-G-2	2-SJ-1118-4FB FLANGE BOLTING	VT	900-1/50	165800	X	-	-	
B4.8	B-J	2-SJ-1118-5 FLANGE TO PIPE	PT	200-1/56 DEV. 2	165900	X	-	-	
B4.8	B-J	2-SJ-1118-17 TEE TO PIPE	PT PT	200-1/56 DEV. 2	167100	- X	- -	X -	SEE CNF 84-1-001. ONE LINEAR AND ONE ROUND INDICATION. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE INDICATIONS WERE OBSERVED.
B4.8	B-J	2-SJ-1118-29 ELBOW TO PIPE	PT PT	200-1/56 DEV. 2	168300	- X	- -	X -	SEE CNF 84-1-006. ONE LINEAR INDICATION. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE INDICATIONS WERE OBSERVED.
		1-1/2-SJ-1142(SEE FIGURES A-89,A-90,A-91)							
B4.12	B-G-2	1-1/2-SJ-1142-7FB FLANGE BOLTING	VT	900-1/50	170700	X	-	-	
B4.8	B-J	1-1/2-SJ-1142-9 PIPE TO VALVE	PT	200-1/56 DEV. 2	170900	X	-	-	

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOREC	INSIG	OTHER	REMARKS
		1-1/2-SJ-1142(SEE FIGURES A-89,A-90,A-91)							
		(CONTD)							
B4.8	B-J	1-1/2-SJ-1142-21 ELBOW TO PIPE	PT	200-1/56 DEV. 2	172100	X	-	-	
B4.8	B-J	1-1/2-SJ-1142-33 PIPE TO ELBOW	PT	200-1/56 DEV. 2	173300	X	-	-	
B4.8	B-J	1-1/2-SJ-1142-45 ELBOW TO PIPE	PT	200-1/56 DEV. 2	174500	X	-	-	
		1-1/2-SJ-1132 (SEE FIGURES A-92, A-93)							
B4.8	B-J	1-1/2-SJ-1132-1 BRANCH CONNCTN TO COUPLING	PT	200-1/56 DEV. 2	175400	X	-	-	
B4.8	B-J	1-1/2-SJ-1132-13 PIPE TO ELBOW	PT	200-1/56 DEV. 2	176600	X	-	-	
B4.8	B-J	1-1/2-SJ-1132-23 ELBOW TO PIPE	PT PT	200-1/56 DEV. 2	177900	- X	- -	X -	SEE CNF 84-1-002. TWO LINEAR INDICATIONS. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE WERE OBSERVED.

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE REC	INSIG	OTHER	REMARKS
		1-1/2-SJ-1132 (SEE FIGURES A-92, A-93)							
		(CONTD)							
B4.8	B-J	1-1/2-SJ-1132-35 ELBOW TO PIPE	PT	200-1/56 DEV. 2	179100	X	-	-	
		1-1/2-SJ-1122 (SEE FIGURES A-94, A-95)							
B4.8	B-J	1-1/2-SJ-1122-3 PIPE TO ELBOW	PT PT	200-1/56 DEV. 2	180500	- X	- -	X -	SEE CNF 84-1-003. TWO LINEAR INDICATIONS. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE WERE OBSERVED.
B4.8	B-J	1-1/2-SJ-1122-15 VALVE TO PIPE	PT	200-1/56 DEV. 2	181800	X	-	-	
B4.9	B-K-1	1-1/2-SJ-1122-15PS-1 & 2 PIPE SUPPORT	PT	200-1/56 DEV. 2	182000	X	-	-	
B4.8	B-J	1-1/2-SJ-1122-17 ELBOW TO PIPE	PT	200-1/56 DEV. 2	182200	X	-	-	
B4.8	B-J	1-1/2-SJ-1122-37 PIPE TO ELBOW	PT	200-1/56 DEV. 2	184200	X	-	-	

SAFETY INJECTION SYSTEM

(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INS IG	EM	OTHER	REMARKS
		1-1/2-SJ-1112 (SEE FIGURES A-96, A-97)								
B4.8	B-J	1-1/2-SJ-1112-3 PIPE TO ELBOW	PT	200-1/56 DEV. 2	185400	X	-	-	-	
B4.8	B-J	1-1/2-SJ-1112-15 PIPE TO ELBOW	PT PT	200-1/56 DEV. 2	186700	- X	-	-	X -	SEE CNF 84-1-004. TWO LINEAR INDICATIONS. AFTER COSMETIC BUFFING BY PSE&G NO RECORDABLE WERE OBSERVED.
B4.8	B-J	1-1/2-SJ-1112-39 ELBOW TO PIPE	PT	200-1/56 DEV. 2	189100	X	-	-	-	
B4.8	B-J	1-1/2-SJ-1112-44 PIPE TO VALVE	PT	200-1/56 DEV. 2	189600	X	-	-	-	

REACTOR COOLANT PUMPS

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	OTHER	REMARKS
		PUMP MOTOR FLYWHEEL							
RG1.14	---	11-PMP-FLW PUMP MOTOR FLYWHEEL	PT	200-1/56	191300			X	SEE CNF 84-1-012. 10 LINEAR
			PT	DEV. 2		X			PT INDICATIONS ON INNER BORE.
			MT	300-1/26				X	AFTER SPOT GRINDING A ONE
			MT	DEV. 1		X			SQUARE INCH AREA THE INDICA-
			UT0	600-6/22		X			TIONS WERE RESOLVED AS NONRE-
			UT45	DEV. 7		X			LEVANT AND ACCEPTED BY PSE&G.
									SEE CNF 84-1-013. 29 MT INDI-
									CATIONS ON TOP SURFACE. AFTER
									REWORK THE REEXAMINATION RE-
									VEALED NO RECORDABLE INDICA-
									TIONS. PT ON INNER BORE AND
									KEYWAYS, AND MT AND UT ON TOP
									AND BOTTOM SURFACES.
									BASIC CALIBRATION BLOCK
									5-CSCL-42-SAM
									7-CSCL-50-SAM
RG1.14	---	13-PMP-FLW PUMP MOTOR FLYWHEEL	PT	200-1/56	192900	X			SEE CNF'S 84-1-010 AND
				DEV. 2					84-1-014. SIX MT INDICATIONS
			MT	300-1/26				X	ON BOTTOM SURFACE AND ONE
			MT	DEV. 1		X			INDICATION ON TOP SURFACE.
			UT0	600-6/22		X			AFTER REWORK THE REEXAMINATION
			UT45	DEV. 7		X			REVEALED NO RECORDABLE INDICA-
									TIONS. PT ON INNER BORE AND
									KEYWAYS, AND MT AND UT ON TOP
									AND BOTTOM SURFACES.
									BASIC CALIBRATION BLOCK
									5-CSCL-42-SAM
									7-CSCL-50-SAM

REACTOR COOLANT PUMPS

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INSIG	OTHER	REMARKS
		PUMP MOTOR FLYWHEEL							
RG1.14	---	14-PMP-FLW PUMP MOTOR FLYWHEEL	PT	200-1/56	193700	-	-	X	SEE CNF 84-1-011. THREE LINEAR PT INDICATIONS ON INNER BORE. AFTER SPOT GRINDING A ONE SQUARE INCH AREA THE INDICATIONS WERE RESOLVED AS NON-RELEVANT AND ACCEPTED BY PSE&G SEE CNF 84-1-014. MT INDICATIONS ON TOP SURFACE. AFTER REWORK THE REEXAMINATION REVEALED NO RECORDABLE INDICATIONS. PT ON INNER BORE AND KEYWAYS, AND MT AND UT ON TOP AND BOTTOM SURFACES. ***BASIC CALIBRATION BLOCK*** 5-CSCL-42-SAM 7-CSCL-50-SAM
			PT	DEV. 2		X	-	-	
			MT	300-1/26		-	-	X	
			MT	DEV. 1		X	-	-	
			UT0	600-6/22		X	-	-	
			UT45	DEV. 7		X	-	-	

VALVES

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	OTHER	REMARKS
		BOLTING							
B6.9	B-G-2	1 CV 77 ON LINE 3-CV-1133	VT	900-1/50	194000	X	-	-	
B6.9	B-G-2	1 CV 78 ON LINE 3-CV-1133	VT	900-1/50	194100	X	-	-	
B6.9	B-G-2	1 CV 79 ON LINE 3-CV-1141	VT	900-1/50	194200	X	-	-	
B6.9	B-G-2	1 CV 80 ON LINE 3-CV-1141	VT	900-1/50	194300	X	-	-	
B6.9	B-G-2	1 CV 275 ON LINE 3-CV-1141	VT	900-1/50	194500	X	-	-	
B6.9	B-G-2	1 PR 3 ON LINE 6-PR-1103	VT	900-1/50	194900	X	-	-	
B6.9	B-G-2	1 PR 5 ON LINE 6-PR-1105	VT	900-1/50	195100	X	-	-	
B6.9	B-G-2	1 PR 7 ON LINE 3-PR-1107	VT	900-1/50	195300	X	-	-	

VALVES
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INSP GEOM	OTHER	REMARKS
		BOLTING (CONTD)							
B6.9	B-G-2	1 PS 1 ON LINE 4-PS-1111	VT	900-1/50	195400	X	-	-	
B6.9	B-G-2	1 PS 24 ON LINE 4-PS-1111	VT	900-1/50	195600	X	-	-	
B6.9	B-G-2	1 PS 25 ON LINE 4-PS-1111	VT	900-1/50	195700	X	-	-	
B6.9	B-G-2	11 RC 20 ON LINE 3-RC-1113	VT	900-1/50	196700	X	-	-	
B6.9	B-G-2	12 RC 20 ON LINE 3-RC-1123	VT	900-1/50	197600	X	-	-	
B6.9	B-G-2	12 RC 23 ON LINE 3-RC-1123	VT	900-1/50	197700	X	-	-	
B6.9	B-G-2	13 RC 20 ON LINE 3-RC-1133	VT	900-1/50	198500	X	-	-	

VALVES
(CONTD)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONREC	INSIG	GSEM	OTHER	REMARKS
		BOLTING (CONTD)								
B6.9	B-G-2	13 RC 23 ON LINE 3-RC-1133	VT	900-1/50	198600	X	-	-	-	
B6.9	B-G-2	14 RC 20 ON LINE 3-RC-1143	VT	900-1/50	199300	X	-	-	-	
B6.9	B-G-2	14 RC 23 ON LINE 3-RC-1143	VT	900-1/50	199400	X	-	-	-	

CLASS 2 COMPONENTS

FEEDWATER SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSERVICE	OTHER	REMARKS
		14-BF-2141 (SEE FIGURE B-10)							
C2.1	C-G	14-BF-2141-16 REDUCER TO NOZZLE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	210600	X X - X -	- - X - X	- - X - X	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE PROXIMITY OF TWO THERMOCOUPLES. LIMITED EXAMINATION ON WELD DUE TO WELD CROWN CONFIGURATION. NO EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO THE NOZZLE CONFIGURATION. WITH IE BULLETIN 79-13. ***BASIC CALIBRATION BLOCK*** 14-CS-80-.760-35-SAM
		14-BF-2131 (SEE FIGURE B-11)							
C2.1	C-G	14-BF-2131-23 ELBOW TO ELBOW	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	213350	X - - X -	- - X - X	- X X - X	SEE CNF 84-1-016. ONE INDICATION WAS OBSERVED DURING THE UTOW SCAN. INDICATION WAS RESOLVED AS A CODE ALLOWABLE INCLUSION BY SWRI PERSONNEL AND WAS ACCEPTED BY PSE&G PERSONNEL AFTER EVALUATION. LIMITED EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO AN ADJACENT THERMOCOUPLE. ***BASIC CALIBRATION BLOCK*** 16-CS-160-1.610-34-SAM 14-CS-80-.760-35-SAM
C2.1	C-G	14-BF-2131-24 ELBOW TO NOZZLE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	213400	X X - X -	- - X - X	- - X - X	LIMITED EXAMINATION FROM THE UPSTREAM SIDE DUE TO A WELD OVERLAY. LIMITED EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO NOZZLE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 16-CS-160-1.610-34-SAM 14-CS-80-.760-35-SAM

FEEDWATER SYSTEM

(CONID)

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NO REC	INSIG	OTHER	REMARKS
		14-BF-2121 (SEE FIGURE B-12)							
C2.1	C-G	14-BF-2121-18 ELBOW TO REDUCER	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	217780	X - X -	- X - X	- X - X	***BASIC CALIBRATION BLOCK*** 14-CS-80-.760-35-SAM
C2.1	C-G	14-BF-2121-19 REDUCER TO NOZZLE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	217790	X X - X -	- - X - X	- - X - -	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO THE NOZZLE CONFIGURATION. LIMITED UT45T SCAN DUE TO THE WELD CROWN CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 14-CS-80-.760-35-SAM
		14-BF-2111 (SEE FIGURE B-13)							
C2.1	C-G	14-BF-2111-17 ELBOW TO REDUCER	UTOL UT45 UT45T UT60	600-3/61 DEV. 12	220570	X - X -	- X - X	- X - X	***BASIC CALIBRATION BLOCK*** 14-CS-80-.760-35-SAM
C2.1	C-G	14-BF-2111-18 REDUCER TO NOZZLE	UTOL UTOW UT45 UT45T UT60	600-3/61 DEV. 12	220572	X X - X -	- - X - X	- - X - X	NO EXAMINATION FROM THE DOWN-STREAM SIDE DUE TO THE NOZZLE CONFIGURATION. LIMITED UT45T SCAN DUE TO THE WELD CROWN CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 14-CS-80-.760-35-SAM

CHEMICAL AND VOLUME CONTROL SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NONE	INSIG	OTHER	REMARKS
		8-CV-2101 (SEE FIGURES B-15, B-16, B-17)							
C2.1	C-F	8-CV-2101-21 TEE TO PIPE	UTOL UT45 UT45T	800-36/28 DEV. 12	220622	X - -	- X X	- - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. NO EXAMINATION FROM THE UPSTREAM SIDE DUE TO THE TEE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 8-SS-10-.140-24-SAM
C2.1	C-F	8-CV-2101-25 ELBOW TO PIPE	UTOL UT45 UT45T	800-36/28 DEV. 12	220630	X - X	- X -	- - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 8-SS-20-.268-33-SAM
C2.1	C-F	8-CV-2101-40 PIPE TO TEE	UTOL UT45 UT45T	800-36/28 DEV. 12	220660	X X X	- - -	- - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. NO EXAMINATION FROM THE DOWNSTREAM SIDE DUE TO THE TEE CONFIGURATION. ***BASIC CALIBRATION BLOCK*** 8-SS-10-.140-24-SAM

SAFETY INJECTION SYSTEM

ASME SECT XI ITEM NO	ASME SECT XI CATGY	WELD NUMBER AND/OR EXAMINATION AREA IDENTIFICATION	EXAM. METHOD	SWRI PROCEDURE NO./REV.	SUMMARY SHEET NUMBER	NOZZLE	INSIDE	GEOMETRY	OTHER	REMARKS
		8-SJ-2152 (SEE FIGURE B-56)								
C2.1	C-F	8-SJ-2152-21 PIPE TO ELBOW	UTOL UT45 UT45T	800-36/28 DEV. 12	264022	X - X	- X -	- - -	- - -	EXAMINATION SCHEDULED IN ACCORDANCE WITH IE BULLETINS 76-06 AND 79-17. ***BASIC CALIBRATION BLOCK*** 8-SS-40-.330-44-SAM

FORM NIS-1 OWNERS'S DATA REPORT FOR INSERVICE INSPECTIONS
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

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2. Plant: Salem Generating Station P.O. Box E, Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner Certificate of Authorization (if required) N/A.
5. Commercial Service Data 7/1/77.
6. National Board Number for Unit N/A.
7. Examinations Dates: 10/16/82 through 5/21/83.
8. This examination report is for the first examination conducted in the third inspection period which ends July 1, 1987. Salem's first inspection interval is from July 1, 1977 to July 1, 1987.
9. Components Inspected.

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Reactor Vessel	Combustion Engineering	66201 Head 66101 Vessel	N/A	20757
#11 Steam Generator	Westinghouse Tampa Div. P.O. Box 19218 Tampa, FL 33616	1003	N/A	68-10
#12 Steam Generator	" "	1022	N/A	68-08
#13 Steam Generator	" "	1023	N/A	68-09
#14 Steam Generator	" "	1203	N/A	68-51
#11 Regen. Ht. Exch.	Sentry Equip. Corp. Oconomowoc, WI	4195-A4 7,8,9	N/A	385, 386, 387

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9. Components Inspected (cont'd).

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Pressurizer	Delta Southern	1011	N/A	68-8
Chemical Volume & Control Piping System	United Engineers & Constructors (UE&C) 30 South 17th St., Phila. PA 19101	N/A	N/A	N/A
Containment Spray Piping System	UE&C	N/A	N/A	N/A
Mainsteam Piping Sys.	UE&C	N/A	N/A	N/A
#1 VCT Tank	Joseph Oats & Sons, Inc.	1781-2A	N/A	374
#1 Excess Letdown Heat Exchanger	Atlas Industries 81 Somerset Place Clifton, NJ	850	N/A	701
#1 Letdown Heat Exchanger	Atlas Industries	N/A	N/A	694
#1 RHR Heat Exchanger	Engineers & Fabricators, Inc.	S15860-C	N/A	1122

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9. Components Inspected (cont'd).

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
Pressurizer Relief Piping System	UE&C	N/A	N/A	N/A
Reactor Coolant Piping Sys.	UE&C	N/A	N/A	N/A
Residual Heat Removed Piping Sys.	UE&C	N/A	N/A	N/A
Steam Gen. Feed Piping System	UE&C	N/A	N/A	N/A
Safety Inject. Piping System	UE&C	N/A	N/A	N/A

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10. Abstracts of Examinations:

NOTE: This report also contains augmented examinations required by Salem Technical Specifications and Regulatory Guides, Circulars, and bulletins issued by the United States Nuclear Regulatory Commission.

Examinations were conducted by PSE&G as well as companies under contract to PSE&G. The following is a brief summary with further details found in the attached report and on file at the Salem Generating Station.

Since nearly all inservice examination requirements can be found in Salem's Technical Specifications, this summary lists these requirements first arranged according to Tech. Spec. paragraph number followed by applicable NRC circulars and bulletins.

1.0 Technical Specification 4.0.5 - ASME XI

These examinations constituted the sixth ISI performed at Salem, Unit 1, and the first ISI of the third 40-month period of commercial operation. The components were examined in accordance with the

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6. National Board Number for Unit N/A.
10. Examination Summary (Cont'd)

"Examination Plan for the 1984 Inservice Examination of Salem Generating Station, Unit 1."

1.1 Summary

Southwest Research Institute (SwRI) - conducted one hundred twenty-one (121) ultrasonic, eighty (80) liquid penetrant, five (5) magnetic particle, and thirty-two (32) visual examinations on the main steam, residual heat removal, reactor coolant, chemical and volume control, steam generator feedwater, safety injection, and containment spray systems.

There were sixteen (16) nonconformances reported by SwRI:

Visual examinations on the reactor vessel studs numbered 23-27 revealed severe pitting and corrosion. It should be noted that these studs were in the vicinity of a reactor vessel head leak. These studs were reported to PSE&G by SwRI on Customer Notification Form (CNF)

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10. Examination Summary (Cont'd)

84-1-15 and were rejected and replaced with studs S6 through S10. There were also three (3) other studs rejected by PSE&G for being out of tolerance. These were replaced with studs S2 through S4.

Baseline UT and MT were performed on the replacement studs and found acceptable.

Liquid penetrant examinations revealed indications on nine (9) small bore fillet welds. These were reported to PSE&G on CNFs 84-1-001 to 84-1-009. All indications were removed by light blending with a flapper wheel. The areas were reexamined by liquid penetrant and ultrasonic thickness examinations to assure minimum wall was not violated. No deficiencies were found on the reexaminations.

During UT examinations one reportable indication was noted on a feedwater weld and reported to PSE&G on CNF 84-1-16. The indication was resolved by SwRI personnel as a Code-acceptable inclusion and was accepted

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6. National Board Number for Unit N/A.
10. Examination Summary (Cont'd)

after evaluation by PSE&G. The remaining UT examinations revealed numerous insignificant and geometric indications; however, no additional reportable UT indications were observed.

1.2 Catalytic, Inc. - Site Ouality Control Group - conducted visual examinations on three hundred eighty-one (381) supports. Six (6) discrepancies were found i.e. rust, loose nuts, etc., all noted conditions were corrected.

1.3 PSE&G Maintenance Department - conducted thirty-three (33) safety relief valve lift set tests. Twenty (20) main steam safeties and thirteen (13) various system relief valves were tested. Of all the valves tested four (4) accumlator relief valves were found out of tolerance, and seven (7) main steam safeties were found to exceed the allowable limits

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10. Examination Summary (Cont'd)

(worst case was 8.3 lbs. high and 13.0 lbs low). In addition, the Maintenance Department conducted lift set tests on three (3) vacuum breakers, (which are outside the scope of Section XI) and were found acceptable.

1.3.1 In addition to the valves tested on-site, the three (3) pressurizer relief valves (1PR3, 4, and 5) were tested by Wyle Laboratories at Huntsville, Alabama.

The "As Received" tests revealed that 1PR3 and 1PR4 wre found satisfactory but 1PR5 lifted 16 lbs. high and was leaking. All three (3) valves had leakage on the post test leakage test.

All valves were then disassembled and inspected by a Crosby Valve representative. No significant problems were found. All valves were cleaned and the seats and discs lapped

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10. Examination Summary (Cont'd)

1.3.1 (Cont'd)

prior to reassembly. After reassembly, all valves were tested and found satisfactory.

1.4 NUS corporation, under the direction of the Salem ISI Group, conducted a reactor vessel internal examination on all accessible areas as required by ASME XI (fuel was removed).

The results of this examination revealed no abnormal conditions within the reactor vessel.

1.5 PSE&G Nuclear Site Maintenance - ISI Group, conducted five, Ten Year Hydrostatic Pressure Test procedures, this outage. The following piping sections were tested: the pressurizer relief valve discharge piping, the pressurizer relief tank, the high pressure portion of component cooling water inside containment, the reactor coolant system, the reactor coolant pump sealwater return and bypass lines, and the changing pump minimum flow

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10. Examination Summary (Cont'd)

1.5 (Cont'd)
recirculating lines. Other than mechanical joint or valve seat leakage (which were corrected), no leakage was identified.

2.0 Technical Specification 4.4.5.0 - Steam Generator Tube Eddy Current Examination

2.1 Summary

Eddy Current examinations were conducted on No. 12 and No. 14 steam generator tubes this outage. The examinations were conducted by Westinghouse Electric Corporation and coordinated by the Salem ISI Group.

The results of the examinations are as follows:

	#12	#14
Full length (Hot or Cold Leg)	3,168	3,159
"U" Bend (From Hot Leg to #7 Support Plate Cold Leg)	178	173
TOTAL	3,346	3,332

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10. Examination Summary (Cont'd)

% of Indications

T.S. Classification	#12	#14
<20% (Imperfection)	9	39
≥20-39% (Degraded)	9	18
≥40% (Defective)	0	0

From the results of the examinations no (0) tubes were plugged in either No. 12 or No. 14 steam generator.

At this time the total number of tubes plugged in the Salem Unit 1 generators are as follows:

Generator #	11	12	13	14
Tubes Plugged	14	18	13	31

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10. Examination Summary (Cont'd)

3.0 Technical Specification 4.4.9.1.2 "Reactor Vessel Irradiated Specimens"

3.1 Summary

Capsule "Y" was removed from Unit 1 reactor vessel this outage and sent to Westinghouse Electric Corporation for analysis.

As of yet, results have not been received from Westinghouse.

4.0 Technical Specification 4.4.10.1.1 (b) "Reactor Coolant Pump Flywheel Examinations"

4.1 Summary

Ten Year Examinations were performed on No. 11, 13 and 14 reactor coolant pump (RCP) flywheels in accordance with U.S. NRC Reg. Guide 1.14, Article C.

The elements comprising this 10-year examination included PT examinations on the

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10. Examination Summary (Cont'd)

4.1 (Cont'd)

top surface, bottom surface, and outer perimeter of the flywheel. During the PT examinations, indications were recorded on the inner bores of No. 11 and No. 14 flywheels and reported to PSE&G on CNFs 84-1-012 and 84-1-11. After the spot grinding of a one-square-inch section of each of the flywheels, the indications were resolved as superficial chatter marks and accepted after evaluation by PSE&G. MT indicatons were observed on the bottom surface of RCP flywheel No. 13 and the top surface of RCP flywheel Nos. 11, 13, 1 . These were reported to PSE&G on CNFs 84-1-010, -013, and -014 (includes top surface of No. 13 and No. 14), respectively. The indications were dispositioned by PSE&G, and reexamination by SwRI personnel revealed no recordable indications. No reportable indications were observed during the UT examination of the flywheels.

5.0 Technical Specification 4.6.1.2 (a) "Reactor Containment Type "A" Test"

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10. Examination Summary (Cont'd)

5.1 Summary

The second of three containment Type A tests was performed during the 24 hour period ending August 13, 1984. The test instrumentation was setup and monitored by the PSE&G Research Corp. and directed by the Salem ISI Group.

After 24 hours of data collection, the calculated leak rate was 0.033% of containment volume per day. With 95% confidence level and other corrections this converts into 0.0411% of containment volume per day or $0.411L_a$. This is within the Technical Specification acceptance criteria of $0.75L_a$.

6.0 Technical Specification 4.6.1.2 (d) "Containment Type "B" (Penetrations) and "C" (Isolation valves) Leak Rate Testing"

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10. Examination Summary (Cont'd)

6.1 Summary

PSE&G Research Corporation under the supervision of the Salem ISI Group conducted sixty three (63) Type B tests and one hundred twenty two (122) Type C tests.

At this time the total leakage of all Type B and C tests is 29,524.3 SCCM out of an allowable 129,750 SCCM.

7.0 Technical Specification 4.6.1.2 (e) "Elevation 100' and 130' Personnel Airlocks."

7.1 Summary

The PSE&G Research Corporation under the supervision of the Salem ISI Group performed leak rate tests on the 100' and 130' airlocks.

Prior to replacement of the chevron seals on the airlock handwheel penetrations, the combined leakage was 15,810 SCCM. After

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10. Examination Summary (Cont'd)

replacement the leakage dropped to 3,815 SCCM which is within acceptable administrative limits.

8.0 Technical Specification 4.6.1.6.1 "Visual Examination of Containment Surfaces"

8.1 Summary

The PSE&G Research Corporation along with the Salem ISI Group conducted an examination of the containment interior and exterior surfaces, both prior to and after the Type A test.

One adverse condition was noted - several old attachment lugs were starting to rust above the spring line. This condition was identified on Deficiency Report No. MT 84-093 and will be corrected during the next refueling outage.

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10. Examination Summary (Cont'd)

9.0 Technical Specification 4.7.9.(a) "Visual Inspection of Hydraulic and Mechanical Snubbers"

9.1 Summary

Catalytic, Inc. QC Department with assistance from the Salem ISI Group conducted visual examinaitons on all mechanical snubbers.

No discrepencies were found that would effect operability. However, several were found with indications such as minor rust and Attachment ends missing washers. Work orders were issued on these, and deficient conditions were corrected.

The PSE&G ISI Group conducted visual examinations on all hydraulic snubbers. Several minor union leaks were found. These discrepancies were corrected by Catalytic, Inc. under the direction of the PSE&G Nuclear Construction Support Group.

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10. Examination Summary (Cont'd)

10.0 Technical Specification 4.7.9 (c) "Functional Testing of Selected Hydraulic and Mechanical Snubbers"

10.1 Summary - Hydraulic Snubbers

Technical specification 4.7.9 requires functional testing on 10% of the snubber total during each plant refueling outage. In addition, all snubbers which failed their previous functional test shall also be functionally tested. Accordingly seventeen (17) snubbers were tested (see below):

- One (1) 200 kip main steam isolation valve (MSIV) hydraulic snubber.
- Sixteen (16) 1000 kip steam generator (s/g) hydraulic snubber.

NOTE: 100% of the 1000 kip snubbers (sixteen, (16 ea.)) failed their previous functional test.

In-place testing was performed by Babcock and Wilcox, using a hydraulic snubber testing

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10. Examination Summary (Cont'd)

10.1 (Cont'd)

machine developed by Leen Company. All failures were sent to Wyle Laboratories in Huntsville, Alabama for retest and refurbishment. The results of these tests revealed the following:

- One (1) 200 kip (MSIV) snubber found acceptable.
- Fourteen (14) 1000 kip (S/G) snubbers found acceptable.
- Two (2) 1000 kip (S/G) snubbers failed, due to snubbers exceeding the "lockup" velocity requirements.

Both failed snubbers were refurbished and retested satisfactorily at Wyle Laboratories, Huntsville, Alabama.

10.2 Summary - Mechanical Snubbers

Technical Specification 4.7.9 requires functional testing on 10% of the snubber total during each Plant Refueling Outage. In

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10. Examination Summary (Cont'd)

10.2 (Cont'd)

additional, for each type of snubber that does not meet the acceptance criteria an additional 10% of that type of snubber shall be functionally tested. Also per the Pacific Scientific Service Report No. 83-01 (Potential Flaws in Mechanical Snubber Capstan Springs), a visual and functional test shall be performed on specific mechanical snubbers identified by serial number.

The first (10%) sample included fourteen (14) snubbers of which two (2) ((1) PSA 1/4, and (1) PSA 3) failed. Due to this failure an additional 10% were sent to Wyle for testing.

The second (10%) sample, a total of ten (10) snubbers, was tested with one (1) failure (PSA-3), which prompted another 10% to be removed and tested.

At this time it was decided to bring Wyle Laboratories on site with their mobile snubber testing trailer for the balance of the samples.

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10. Examination Summary (Cont'd)

10.2 (Cont'd)

The third (10%) sample, totaling twenty-eight (28) snubbers, were tested with sixteen (16) failures (3 ea. PSA 1/4, 8 ea. PSA 1, 4 ea. PSA 3 and 1 ea. PSA 10). The large sample was selected due to the excessive failures in the feed water system PSA 1/4 and PSA 1/2. The fourth sample was selected and removed for testing.

The fourth (10%) sample, totaling thirteen (13) snubbers, were tested with one (1) failure (1 ea. PSA 3). The fifth and final sample encompassing the balance of the mechanical snubbers were removed for testing.

The fifth and final sample, totaling twenty-two (22) snubbers, was tested with two (2) failures (1 ea. PSA 1 and PSA 10). Below is a summary of the results of the 100% functional testing of Unit 1's mechanical snubbers:

- Snubbers Tested = 104 (100%)
- Snubbers Failed = 22

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10. Examination Summary (Cont'd)

10.2 (Cont'd)

- Model #'s of Failures: PSA 1/4 = 4
  PSA 1 = 9
  PSA 3 = 7
  PSA 10 = 2

All nonradioactively contaminated failed snubbers were sent to their manufacturer, Pacific Scientific, Anaheim, California for retest, repair and evaluation. In those cases where the snubber was contaminated, the snubber was sent to Wyle Laboratories for retest, repair and evaluation.

As of this date, PSE&G has not received back the evaluation (failure mechanism) on snubbers that were rejected.

PSE&G Engineering also performed a safety evaluation on the failed snubbers, and it was concluded that the attached piping and components were not impaired due to inoperability of the snubbers.

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10. Examination Summary (Cont'd)

10.3 Summary - Pacific Scientific Service Report 83-01

In addition to the required sample of snubbers to be functionally tested, seventeen (17) PSA 1 and 3's were sent to Wyle Laboratories for capstan spring inspections, all of which were found acceptable.

11.0 NRC Circular 76-06 and NRC bulletion 79-17 "Commitment to examine and/or flush stainless steel lines containing stagnant borated water"

11.1 Summary

As part of the ISI program, SwRI performed ultrasonic examinations on eight (8) piping welds covered by this circular.

Quarterly chemistry samples were taken by the PSE&G Chemistry Department and results transmitted to the Salem ISI Group as required by Maintenance Procedure M17G.

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10. Examination Summary (Cont'd)

11.1 (Cont'd)

There were no adverse findings in the examinations or tests conducted relative to the reporting requirement of this circular.

12.0 NuReg 0578 - TMI Lessons Learned -
"Performing Service Pressure Leak Exams, measuring leakage outside Containment, and take Corrective Actions necessary to reduce leakage as low as possible on systems likely to contain Radioactive liquids in the event of an incident"

12.1 Summary

The Salem ISI Group along with members of the PSE&G Maintenance Department performed Service Pressure Leak Exams on the Safety Injection, Chemical and Volume Control, Residual Heat Removal and Containment Spray Systems.

In addition, PSE&G Research Corp. under the direction of the Salem ISI Group conducted the Waste Gas System Leak Rate Test in accordance with Maintenance Procedure M11C.

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10. Examination Summary (Cont'd)

12.1 (Cont'd)
There were no adverse findings in the examinations or tests conducted relative to this bulletin.

13.0 NRC Bulletin 79-13 Steam Generator Feedwater Nozzle Cracking, Follow-up

13.1 Southwest Research Institute (SwRI) under the direction of the Salem ISI Group performed ultrasonic examinations on all four (4) steam generator feedwater nozzle to piping welds.

The UT results were evaluated for any cracking or any other defective conditions. The results of these examinations were satisfactory.

14.0 NRC Bulletin 82-02 "Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants"

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10. Examination Summary (Cont'd)

14.1 Summary

Due to this bulletin, all steam generator primary manway bolts on No. 12 and No. 14 S/G were magnetic partical (MT) surface examined by Magnaflux Ouality Services, and visually examined by the Salem ISI Group.

Although no bolts were rejected, one was selected for analysis, due to some indications noted on the visual exam - possibly caused by the forming process. It should be noted that this bolt had no indications when MT examined.

In addition, the reactor coolant pump main flange bolts were visually examined in place by the PSE&G Maintenance Department and the ISI Group. No discrepancies were noted.

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We certify that the statements made up in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 1/11/85 Signed PSE&G - Owner By Louis H Lake

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of Province of New Jersey and employed by Lumberman's Mutual Casualty Company, Long Grove, Illinois, have inspected the components described in this Owner's Data Report during the period 2/24/84 to 10/23/84 and state that to the best of my knowledge and belief the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-17-85

James E. Cook
Inspector's Signature

Commissions N.J. 373 "I"
National Board, State, Province and No.