

## Vogtle PEmails

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**From:** Comar, Manny  
**Sent:** Monday, April 02, 2018 10:13 AM  
**To:** Vogtle PEmails  
**Subject:** DRAFT REQUEST FOR ADDITIONAL INFORMATION RELATED TO LAR 18-004 for Vogtle Units 3 and 4

### Vogtle LAR-18-004 Request for Additional Information

The following is the regulatory basis as well as the draft Request for additional Information question related to LAR 18-004 for Vogtle Units 3 and 4.

The NRC regulations in 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants," General Design Criterion (GDC) 14, "Reactor coolant pressure boundary," state that the reactor coolant pressure boundary (RCPB) shall be designed, fabricated, erected, and tested so as to have an extremely low probability of abnormal leakage, of rapidly propagating failure, and of gross rupture. In addition, GDC 30, "Quality of reactor coolant pressure boundary," in 10 CFR Part 50, Appendix A, states, in part, that components which are part of the RCPB shall be designed, fabricated, erected, and tested to the highest quality standards practical.

In License Amendment Request (LAR) 18-004 (dated January 31, 2018), Southern Nuclear Operating Company (SNC) requested an amendment to the Combined Licenses (COLs) for Vogtle Electric Generating Plant (VEGP) Units 3 and 4 to modify the VEGP Units 3 and 4 COL Appendix A, Technical Specifications (TS). Among the proposed changes in LAR-18-004, SNC requests changes to TS 3.4.6, Pressurizer Safety Valve, Applicability, to require the pressurizer safety valves (PSVs) to be operable when the TS 3.4.14, Low Temperature Overpressure Protection, is not required to be operable. The NRC staff requests that SNC indicate whether any changes in the performance requirements or operating conditions are necessary for the PSVs or any other valves for the implementation of LAR-18-004. If so, the staff requests that SNC describe the plans to incorporate those performance requirements and operating conditions as part of the qualification of those valves in accordance with American Society of Mechanical Engineers (ASME) Standard QME-1-2007, "Qualification of Active Mechanical Equipment Used in Nuclear Power Plants," as accepted in NRC Regulatory Guide 1.100 (Revision 3), "Seismic Qualification of Electrical and Active Mechanical Equipment and Functional Qualification of Active Mechanical Equipment for Nuclear Power Plants."

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18-004 for Vogtle Units 3 and 4  
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**From:** Comar, Manny

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