PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Nuclear Department

Ref: LCR 83-04

July 22, 1983

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Steven Varga, Chief
Operations Reactors Branch 1
Division of Licensing

Gentlemen:

REQUEST FOR AMENDMENT
FACILITY OPERATING LICENSES DPR-70
UNIT NO. 1
SALEM GENERATING STATION
DOCKET NOS. 50-272

In accordance with the Atomic Energy Act of 1954, as amended and the regulations thereunder, we hereby transmit copies of our request for amendment and our analyses of the changes to Facility Operating Licenses DPR-70 for Salem Generating Station, Unit No. 1.

This request consists of an extension of the interval of a surveillance requirement for integrated leak rate testing of the containment building.

This change involves a single safety issue and is, therefore, determined to be a Class III Amendment as defined by 10CFR 170.22. A check in the amount of $4,000 is enclosed.

Pursuant to the requirements of 10 CFR50.91(b)(1), a copy of this request for amendment has been sent to the State of New Jersey as indicated below.

A001
1/1

Rec'd w/check
$4,000

This submittal includes three (3) signed originals and forty (40) copies.

Very truly yours,

E. A. Liden
Manager - Nuclear
Licensing and Regulation

Enclosure

CC: Mr. Donald C. Fischer
Licensing Project Manager

Mr. Leif Norrholm
Senior Resident Inspector

Mr. Frank Cosolito, Acting Chief
Bureau of Radiation Protection
Department of Environmental Protection
380 Scotch Road
Trenton, New Jersey 08628

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STATE OF NEW JERSEY)
) SS.
COUNTY OF SALEM)

RICHARD A. UDERITZ, being duly sworn according to law deposes and says:

I am a Vice President of Public Service Electric and Gas Company, and as such, I find the matters set forth in our letter dated July 22, 1983, concerning, Request for Amendment to Facility Operating Licenses DPR-70, are true to the best of my knowledge, information and belief.

RICHARD A. UDERITZ

Subscribed and sworn to before me
this 22 day of July, 1983

Ann McKeever
Notary Public of New Jersey

My Commission expires on January 7, 1985

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PROPOSED CHANGES
TECHNICAL SPECIFICATION
SALEM NO. 1 UNIT

DESCRIPTION OF CHANGE

On a one-time basis, extend the 40 ± 10 month interval of Technical Specification 4.6.1.2a during the first 10 year service period to permit the second inservice integrated leak rate test to be performed during the fifth refueling outage by adding a footnote to the specification that reads:

> "The second inservice Integrated Leak Rate Test shall be performed at the fifth refueling outage."

REASON FOR CHANGE

Paragraph 4.6.1.2 of the Salem Unit 1 Technical Specification defines the surveillance requirements for the overall integrated containment leakage rate, including the schedule for conducting the necessary surveillance tests, to be in conformance with Appendix J of 10CFR 50. Specifically the Technical Specification paragraph states that three Type "A" tests be conducted at 40 ± 10 month intervals during each ten year service period, and that the third test of each set be conducted when the plant is shutdown for the ten year plant inservice inspections. Neither the Technical Specification nor Appendix J define the term "service period", but there is such a definition in Section XI of the ASME Boiler and Pressure Vessel Code which governs inservice inspection at Salem. Paragraph 1WA-2400 of Section XI states that the (10 year) inspection "intervals" represent calendar years after the reactor facility has been placed into Commercial Service.

Due to an earlier interpretation of the Technical Specifications, the first inservice Type "A" test for Salem Unit 1 was completed on August 13, 1979, approximately 25 1/2 months after the date of commercial operation (approximately 32 months after initial criticality). Scheduling of the second Type "A" test at the maximum limit of the 40 ± 10 month duration from the August, 1979 date would require the test be conducted in October, 1983. Because of the unforeseen delays encountered during the restart of Salem Unit 1 from its fourth refueling outage, scheduling of the test in October, 1983 would require a four to five week shutdown of the plant for the single purpose of performing the test, after only approximately 5 months of operation.

SAFETY EVALUATION

a. The probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report will not be increased.

- Paragraph 4.6.1.2 of the Unit No. 1 Technical Specification states that the tests shall be in conformance with the criteria specified in Appendix J of 10CFR 50. The logic in the intervals specified in Appendix J by the NRC is to have a set of three tests performed at approximately equal intervals. Waiting until the next scheduled refueling outage to perform the second periodic Type "A" test does not appear to be inconsistent with the intent of the periodic retest schedule specified in Appendix J.

- Acceptable integrated leakage tests have been performed for both the preoperational Type "A" test and for the first Type "A" retest. The preoperational Type "A" test resulted in a total leakage at the 95% confidence level of 0.718 La; where La is defined as 0.1 percent by weight of the containment air per 24 hours at design pressure (47.0 psig). The first Type "A" retest resulted in a total leakage at the 95% confidence level of 0.62 La.

b. The possibility of an accident or malfunction of a different type than any evaluated previously in the safety analysis report will not be created.

- This LCR is not related to any plant modification.

c. The margin of safety as defined in the basis for any Technical Specification is not reduced.

- Unit No. 1 has not experienced any unusual temperature or pressure excursions within the Reactor Containment Building since the last Type "A" test and we therefore have no reason to suspect that the containment liner integrity has in any way been reduced.

- A complete local leak rate test program was completed on all penetrations and valves requiring Type "B" and "C" testing during the most recent refueling outage. At the end of that outage, the combined leakage from all Type "B" and "C" penetrations and valves was well within the allowable limit of 0.6 La. Type "B" and "C" tests will be repeated within the 24 month interval specified in the Technical Specification.

- The limiting dose values of 10CFR 100 for purpose of licensing are 25 rem whole body and 300 rem thyroid. The present design bases LOCA calculations yield a whole body dose of 3 rem and a thyroid dose of 96 rem at the minimum exclusion boundary. The assumed design bases leak rate of .001 containment free volume/day was utilized in this calculation. Considerable margin exists between the calculated radiological dose resulting from the design basis containment leakage and the radiological dose rates specified in 10CFR 100.

Our evaluation of the conditions described herein enable us to determine that this change introduces no Unreviewed Safety Questions and involves no Significant Hazards Consideration.