

ATTACHMENT C TO L-77-291

PROPOSED TECHNICAL
SPECIFICATION CHANGES

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

- b. A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.

4.0.3 Performance of a Surveillance Requirement within the specified time interval shall constitute compliance with OPERABILITY requirements for a Limiting Condition for Operation and associated ACTION statements unless otherwise required by the specification.

4.0.4 Entry into an OPERATIONAL MODE or other specified applicability condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the stated surveillance interval or as otherwise specified.

4.0.5 Surveillance Requirements for in-service inspection and testing of ASME Code Class 1, 2 and 3 components shall be applicable as follows:

- a. In-service inspection of ASME Code Class 1, 2 and 3 components and in-service testing of ASME Code Class 1, 2 and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i). In cases where, during the process of inspection and testing, certain requirements are found to be impractical, due to unforeseen circumstances, relief may be requested at that time and satisfaction of these requirements may be deferred pending NRC action upon that request.
- b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the in-service inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda shall be applicable as follows in these Technical Specifications:

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for in-service inspection and testing criteria	Required frequencies for performing in-service inspection and testing activities
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Test Period:

- | | |
|-----------------------|--|
| 1. Refueling Shutdown | A scheduled shutdown (with reactor head off) with reactor maintained in *MODE 6 for at least 72 hours. |
| 2. Cold Shutdown | A scheduled shutdown with reactor maintained in *MODE 5 for at least 168 hours. |
| 3. Operation | 3 Months or less to Code |

*Table 1.1, Operational Modes

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for in-service inspection and testing criteria

Required frequencies for performing in-service inspection and testing activities

Monthly
3 Months
Yearly or Annually

At least once per 31 days
At least once per 92 days
At least once per 366 days

Performance of the above in-service inspection and testing activities shall be in addition to other specified Surveillance Requirements. The provisions of Specification 4.0.2 are applicable to surveillance intervals associated with in-service inspection and testing activities required by Section XI of the above ASME Boiler and Pressure Vessel Code and applicable Addenda.

REACTOR COOLANT SYSTEM

SAFETY VALVES - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.2 A minimum of one pressurizer code safety valve shall be OPERABLE with a lift setting of 2500 PSIA \pm 1%.

APPLICABILITY: MODES 4 and 5.

ACTION:

With no pressurizer code safety valve OPERABLE, immediately suspend all operations involving positive reactivity changes and place an OPERABLE shutdown cooling loop into operation.

SURVEILLANCE REQUIREMENTS

4.4.2 No additional Surveillance Requirements other than those required by Specification 4.0.5.

REACTOR COOLANT SYSTEM

3.4.10 STRUCTURAL INTEGRITY

ASME CODE CLASS 1, 2 AND 3 COMPONENTS

LIMITING CONDITION FOR OPERATION:

3.4.10.1 The structural integrity of ASME Code Class 1, 2 and 3 components (except Steam Generator Tubes) shall be maintained in accordance with Specification 4.4.10.1.

APPLICABILITY: MODES 1, 2, 3 & 4

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component to within its limit or isolate the affected component from service.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.10.1 No additional Surveillance Requirements other than those required by Specification 4.0.5.

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EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

e. At least once per 18 months, during shutdown, by:

1. Verifying that each automatic valve in the flow path actuates to its correct position on a Safety Injection Actuation Signal.
2. Verifying that each of the following pumps start automatically upon receipt of a Safety Injection Actuation Signal:
 - a. High-Pressure Safety Injection pump.
 - b. Low-Pressure Safety Injection pump.

f. No additional Surveillance Requirements other than those required by Specification 4.0.5.

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CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- I. Verifying that upon a recirculation actuation signal, the containment sump isolation valves open and that a recirculation mode flow path via an OPERABLE shutdown cooling heat exchanger is established, and.
 2. Verifying that each valve (manual, power operated or automatic) in the flow path is positioned to take suction from the RWT on a Containment Pressure--High-High signal.
- b. At least once per 18 months, during shutdown, by:
1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment Pressure--High-High signal.
 2. Verifying that each spray pump starts automatically on a Containment Pressure--High-High signal.
- c. At least once per 5 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed. 3/18/77
- d. No additional Surveillance Requirements other than those required by Specification 4.0.5.

CONTAINMENT SYSTEMS

3/4.6.3 CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves specified in Table 3.6-2 shall be OPERABLE with isolation times as shown in Table 3.6-2.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more of the isolation valve(s) specified in Table 3.6-2 inoperable, either:

- a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or
- b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or
- c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange, or
- d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves specified in Table 3.6-2 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair, or replacement work is performed on the valve or its associated actuator, control, or power circuit by performance of a cycling test and verification of isolation time.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each isolation valve specified in Table 3.6-2 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by verifying that on a containment isolation test signal, each isolation valve actuates to its isolation position.

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4.6.3.1.3 No additional surveillance requirements other than those required by Specification 4.0.5.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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b. No additional surveillance requirements other than those required by
Specification 4.0.5.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 18 months during shutdown, by verifying that each automatic valve servicing safety related equipment actuates to its correct position on a Safety Injection Actuation Signal.

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c. No additional surveillance requirements other than those required by Specification 4.0.5.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 18 months during shutdown, by verifying that each automatic valve servicing safety related equipment actuates to its correct position on a Safety Injection Actuation Signal.

c. No additional surveillance requirements other than those required by Specification 4.0.5.

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APPLICABILITY

BASES

4.0.5 This specification ensures that in-service testing of ASME Code Class 1, 2, and 3 pumps and valves and in-service inspection of ASME Code Class 1, 2, and 3 Components will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50.55a. Relief from any of the above requirements has been provided in writing by the Commission and is not a part of these technical specifications.

This specification includes a clarification of the frequencies for performing the in-service testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout these technical specifications and to remove any ambiguities relative to the frequencies for performing the required in-service testing activities.

REACTOR COOLANT SYSTEM

BASES

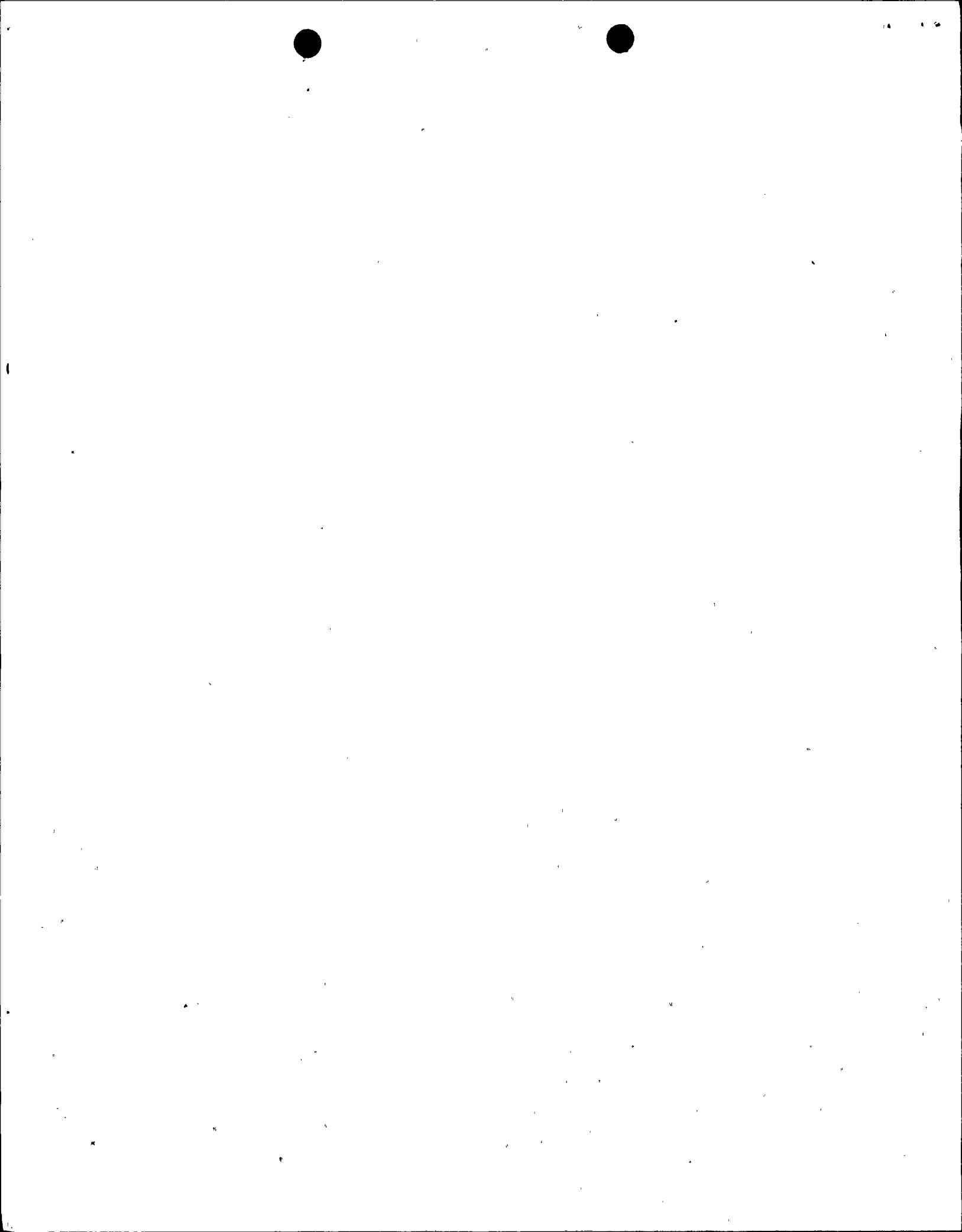
for piping, pumps and valves. Below this temperature, the system pressure must be limited to a maximum of 20% of the system's hydrostatic test pressure of 3125 psia.

The number of reactor vessel irradiation surveillance specimens and the frequencies for removing and testing these specimens are provided in Table 4.4-5 to assure compliance with the requirements of Appendix H to 10 CFR Part 50.

The limitations imposed on the pressurizer heatup and cooldown rates and spray water temperature differential are provided to assure that the pressurizer is operated within the design criteria assumed for the fatigue analysis performed in accordance with the ASME Code requirements.

3/4.4.10 STRUCTURAL INTEGRITY

The inspection programs for the ASME Code Class 1, 2 and 3 components ensure that the structural integrity of these components will be maintained at an acceptable level throughout the life of the plant. To the extent applicable, the inspection program for these components is in compliance with Section XI of the ASME Boiler and Pressure Vessel Code.



REACTOR COOLANT SYSTEM

BASES

3/4.4.11 CORE BARREL MOVEMENT

This specification is provided to ensure early detection of excessive core barrel movement if it should occur. Core barrel movement will be detected by using four excore neutron detectors to obtain Amplitude Probability Distribution (APD) and Special Analysis (SA). Baseline core barrel movement Alert Levels and Action Levels at nominal THERMAL POWER levels of 20%, 50%, 80% and 100% of RATED THERMAL POWER will be determined during the reactor startup test program.

A modification to the required monitoring program may be justified by an analysis of the data obtained and by an examination of the affected parts during the plant shutdown at the end of the first fuel cycle.