



Entergy Operations, Inc.
River Bend Station
5485 U.S. Highway 61N
St. Francisville, LA 70775
Tel 225-381-4157

Steven P. Vercelli
General Manager – Plant Operations

RBG-47767

June 13, 2017

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Licensee Event Report 50-458 / 2017-002-01
River Bend Station – Unit 1
Docket No. 50-458
License No. NPF-47

RBF1-17-0063

Dear Sir or Madam:

In accordance with 10 CFR 50.73, enclosed is the subject Licensee Event Report. This is a supplement to the original report dated April 18, 2017, and is being submitted to provide annotation of the applicability of 10 CFR 21 to this event. Revisions are indicated by change bars in the right margins.

This document contains no commitments. If you have any questions, please contact Mr. Tim Schenk at 225-381-4177.

Sincerely,

A handwritten signature in black ink, appearing to read "Steven P. Vercelli".

SPV / dhw

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region IV
1600 East Lamar Blvd.
Arlington, TX 76011-4511

NRC Sr. Resident Inspector
P. O. Box 1050
St. Francisville, LA 70775

INPO
(via ICES reporting)

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Central Records Clerk
Public Utility Commission of Texas
1701 N. Congress Ave.
Austin, TX 78711-3326

Department of Environmental Quality
Office of Environmental Compliance
Radiological Emergency Planning and Response Section
Ji Young Wiley
P.O. Box 4312
Baton Rouge, LA 70821-4312



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME River Bend Station – Unit 1	2. DOCKET NUMBER 05000-458	3. PAGE 1 OF 4
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4. TITLE
Loss of Safety Function of Onsite Electrical Distribution Due to Malfunction of Control Building HVAC System

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
02	18	2017	2017	002	01	06	13	2017	FACILITY NAME	05000
									FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)			
5	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
10. POWER LEVEL	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(1)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(i)
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 73.77(a)(2)(ii)
		<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input checked="" type="checkbox"/> OTHER	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

LICENSEE CONTACT Tim Schenk, Manager – Regulatory Assurance	TELEPHONE NUMBER (Include Area Code) 225-381-4177
--	--

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
(see text)									

14. SUPPLEMENTAL REPORT EXPECTED	15. EXPECTED SUBMISSION DATE			
<input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On February 18, 2017, at 3:37 p.m. CST, while a refueling outage was in progress, the operators were shifting subsystems of the main control building ventilation system. The Division 2 "B" chiller had been in service, and it was intended to start the Division 1 "C" chiller to facilitate the outage work schedule. After the swap, operators noted that the air flow was abnormally low, and within approximately four minutes, the "C" chiller tripped. The operators were unsuccessful in attempts to restore the Division 2 subsystem to service, and the abnormal operating procedures for the loss of control building ventilation were then implemented. The electrical distribution subsystems in the control building were declared inoperable due to the loss of the ventilation system. This condition is being reported in accordance with 10 CFR 50.73(a)(2)(v)(A). As described in the causal analysis, a circuit breaker manufacturing defect that violated the single failure requirements of 10 CFR 50 Appendix A, General Design Criteria, was discovered. This is being reported in accordance with 10 CFR 50.73(a)(2)(ii)(B) as an unanalyzed condition. During the restoration of the ventilation system, main control room temperature increased from approximately 73F to 81F as recorded in the operator's logs. No high temperature alarms from the electrical equipment rooms actuated. Thus, this event was of minimal significance to the health and safety of the public. 10 CFR 21 is applicable to this event.



**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

(See NUREG-1022, R.3 for instruction and guidance for completing this form
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River Bend Station – Unit 1	05000-458	YEAR	SEQUENTIAL NUMBER	REV NO.
		2017	002	01

NARRATIVE
REPORTED CONDITION

On February 18, 2017, at 3:37 p.m. CST, while a refueling outage was in progress, the operators were shifting subsystems of the main control building ventilation system [VI]. The Division 2 "B" chiller (**CHU**) had been in service, and it was intended to start the Division 1 "C" chiller to facilitate the outage work schedule. After the swap, operators noted that the air flow was abnormally low, and within approximately four minutes, the "C" chiller tripped. The operators attempted to restore the Division 2 subsystem to service, but they were unsuccessful, and the abnormal operating procedures for the loss of control building ventilation were then implemented. The electrical distribution subsystems [ED] in the control building were declared inoperable due to the loss of the ventilation system. This condition is being reported in accordance with 10 CFR 50.73(a)(2)(v).

As described in the causal analysis below, a circuit breaker (**BKR**) manufacturing defect that violated the single failure requirements of 10 CFR 50 Appendix A, General Design Criteria, was discovered. This is being reported in accordance with 10 CFR 50.73(a)(2)(ii)(B) as an unanalyzed condition.

10 CFR 21 is applicable to this event. The Masterpact NT circuit breaker is manufactured by Nuclear Logistics Incorporated.

BACKGROUND

Two 100-percent capacity chillers provide cooling to each ventilation system sub-loop, each of which provides cooling, heating, ventilation, pressurization, and smoke removal for several areas within the building. The ventilation system comprises two independent, redundant trains of components and subsystems either of which supports the safety function of providing a controlled environment in the main control room (MCR) and the environmental requirements of the safety-related electrical equipment in the building. Each control building ventilation subsystem is supported by the respective safety-related emergency diesel generator.

The control building chilled water system (HVK) provides chilled water to the cooling coils in three separate air handlers within each division. The air handling system (HVC) removes the heat generated within the MCR and electrical equipment rooms. Air handlers serve the MCR, standby switchgear rooms, battery charger rooms, inverter rooms, and chiller rooms. Each air handler serves both trains of their assigned areas (i.e., both the Division 1 and 2 standby switchgear rooms cooling may be fully served by the air handler in either train). In the event of a total ventilation system failure, the current Technical Specification action to restore the ventilation system is bounded by the 2-hour completion times for DC sources

IMMEDIATE ACTIONS

In troubleshooting the abnormally low air flow with the Division 1 subsystem in service, it was discovered that air flow control dampers in the system had not operated correctly, which diverted air flow from the operating air handler. Operators implemented abnormal operating procedures to provide alternate means of cooling to the affected areas and monitored the areas for temperature rise.



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CONTINUATION SHEET**

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Under temporary configuration controls, operators closed the air dampers by removing their control power fuses, and restored the normal air flow path. The Division 1 "C" HVK chiller was successfully started at 5:37 p.m. CST. At the onset of the event, MCR temperatures were approximately 73F. The highest temperature recorded before the restoration of the cooling system was approximately 81F.

CAUSAL ANALYSIS

The failure of the air handler dampers to close correctly was traced to an internal failure in the 480-volt circuit breaker for the Division 2 switchgear room air handling unit. It was found that a mechanism operated cell (MOC) switch positioner linkage screw had fallen out, causing the logic circuit for the damper controls to fail to actuate. The screw in the MOC linkage failed because it was shorter than required by the circuit breaker design (i.e., one-quarter inch in length versus three-eighths inch).

"Single failure" is defined by 10 CFR 50 Appendix A, General Design Criteria for Nuclear Power Plants. A single failure means an occurrence which results in the loss of capability of a component to perform its intended safety functions. Multiple failures resulting from a single occurrence are considered to be a single failure. The control building ventilation system is required to be designed against an assumed single failure. Failure of the screw caused the failure of both the Division 1 and Division 2 ventilation subsystems.

It is possible that the MOC screw may have been loose since the last time the breaker was successfully closed on January 31. However, its actual failure can be pinpointed only to the opening of the breaker the on date of the event.

CORRECTIVE ACTIONS TO PREVENT RECURRENCE

All safety-related, electrically operated Masterpact NT breakers were categorized with regard to the design of their MOC logic circuit switches. Those breakers with the potential to cause the mis-operation of other components by the same failure mechanism of the February 18 event were inspected to verify that correct screws and lock washers were installed. The remaining population of Masterpact NT breakers will be inspected.

Design changes were implemented to alter the damper logic circuits for the main control room and electrical equipment area air handlers to eliminate the cause of this event as a potential failure mode for damper actuation.

Technical and quality requirements for receipt inspections of Masterpact circuit breakers will be revised to specifically examine the MOC hardware. Similar breakers already in the warehouse will be inspected. These actions are being tracked in the corrective action program.

EVALUATION OF PRIOR OCCURRENCES

No events reported in the last three years by RBS have occurred as a result of the breaker failure mode discovered in this evaluation.



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SAFETY SIGNIFICANCE

The normal operating upper temperature limit for the main control room and electrical equipment rooms served by the HVC system is 104F. During the restoration of the ventilation system, main control room temperature increased from approximately 73F to 81F as recorded in the operator's logs. The electrical distribution equipment rooms are served by remote temperature monitors, and none of those associated alarms actuated in the MCR during the time that the ventilation system was out of service. Thus, this event was of minimal significance to the health and safety of the public.

(NOTE: Energy Industry Identification System component function identifier and system name of each component or system referred to in the LER are annotated as (**XX**) and [XX], respectively.)

Sheet 1 of 2

Letter #: RBF1-17-0063

Response Due: 6/19/2017

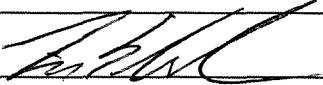
Subject: LER 2017-002-01 (Loss of safety function of onsite power sources due to HVK failure)

Date Issued for Review: 6/12/2017

Correspondence Preparer / Phone #: Danny Williamson x4279

Section I

Letter Concurrence and Agreement to Perform Actions

POSITION / NAME	Action (concurrence, certification, etc.)	Signature (sign, interoffice memo, e-mail, or telecom)
Tim Schenk	(comment / <u>concur</u>)	

Section II

Correspondence Screening

Does this letter contain commitments? If "yes," identify the commitments with due dates in the submittal and in Section III. When fleet letters contain commitments, a PCRS LO (e.g., LO-LAR, LO-WT) should be initiated with a CA assigned to each applicable site to enter the commitments into the site's commitment management system.	Yes No	<input type="checkbox"/> <input checked="" type="checkbox"/>
Does this letter contain any information or analyses of new safety issues performed at NRC request or to satisfy a regulatory requirement? If "yes," reflect requirement to update the UFSAR in Section III.	Yes No	<input type="checkbox"/> <input checked="" type="checkbox"/>
Does this letter require any document changes (e.g., procedures, DBDs, FSAR, TS Bases, etc.), if approved? If "yes," indicate in Section III an action for the responsible department to determine the affected documents. (The Correspondence Preparer may indicate the specific documents requiring revision, if known or may initiate an action for review.)	Yes No	<input type="checkbox"/> <input checked="" type="checkbox"/>
Does this letter contain information certified accurate? If "yes," identify the information and document certification in an attachment. (Attachment 9.5 must be used.)	Yes No	<input type="checkbox"/> <input checked="" type="checkbox"/>

Section III

Actions and Commitments

Required Actions	Due Date	Responsible Dept.
<i>Note: Actions needed upon approval should be captured in the appropriate action tracking system</i>		
none		
Commitments	Due Date	Responsible Dept.
<i>Note: When fleet letters contain commitments, a PCRS LO should be initiated with a CA assigned to each applicable site to enter the commitments into the site's commitment management system.</i>		
none		

Section IV Final Document Signoff for Submittal

Correspondence Preparer	Danny Williamson
Final Submittal Review (optional)	na
Responsible Department Head	na