

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-311
UNIT Salem #2
DATE September 10, 1980
COMPLETED BY L. K. Miller
TELEPHONE 609-365-7000 X507

MONTH August 1980

DAY AVERAGE DAILY POWER LEVEL	(MWe-NET)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY AVERAGE DAILY POWER LEVEL	(MWE-NET)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

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OPERATING DATA REPORT

50-311

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OPERATING STATUS

1. Unit Name: Salem #2
2. Reporting Period: August 1980
3. Licensed Thermal Power (MWT): 3411
4. Nameplate Rating (Gross MWe): 1162
5. Design Electrical Rating (Net MWe): 1115
6. Maximum Dependable Capacity (Gross MWe): 1149
7. Maximum Dependable Capacity (Net MWe): 1104
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
NONE

Notes:

9. Power Level To Which Restricted, If Any (Net MWe): 5% Thermal
10. Reasons For Restrictions, If Any: Pending successful completion of low power physics and natural circulation testing programs.

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	744	3,240	3,240
12. Number Of Hours Reactor Was Critical	268.2	268.2	268.2
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	0	0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	0
17. Gross Electrical Energy Generated (MWH)	0	0	0
18. Net Electrical Energy Generated (MWH)	0	0	0
19. Unit Service Factor	0	0	0
20. Unit Availability Factor	0	0	0
21. Unit Capacity Factor (Using MDC Net)	0	0	0
22. Unit Capacity Factor (Using DER Net)	0	0	0
23. Unit Forced Outage Rate	0	0	0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	6/30/80	8/02/80
INITIAL ELECTRICITY	9/01/80	N/A
COMMERCIAL OPERATION	N/A	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August 1980

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NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
									N/A

¹
 F: Forced
 S: Scheduled

² Reason:
 A-Equipment Failure(Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error(Explain)
 H-Other (Explain)

³ Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation of Previous Outage
 5-Load Reduction
 9-Other

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER) File (NUREG-0161)

⁵ Exhibit 1-Same Source

MAJOR PLANT MODIFICATIONS
August 1980

REPORT MONTH _____

DOCKET NO.: 50-311

UNIT NAME: Salem #2

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*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2ET-0630	Leak Detection	Data Required to Satisfy NRC
2EC-0800	Lighting	Add Two Lights in Parking Lot
2EC-0821	Structural	Add 4" ϕ Core Bore - TGA
2EC-0957	Radiation Monitoring System	Modify 2R18 Sampler Tubing
2ET-0970	Inverters	Perform E-154 Test
2EC-0987	Control Air	Add 1-1/2" Check Valve
2SC-0162	Waste Disposal Liquid	Resin Decant Filter

MAJOR PLANT MODIFICATIONS

REPORT MONTH August 1980

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*DCR NO.	10CFR50.59 SAFETY EVALUATION
2ET-0630	The purpose of this DCR is the accumulation of data for the refinement of the Salem Leak Detection Curves. No change to any system or operating procedure is required. This DCR does not impact any safety related system. Data will be acquired from equipment already installed at Salem.
2EC-0800	This DCR covers the addition of two lighting standards in the parking lot. This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question.
2EC-0821	The addition of a 4" Ø Core Bore Hole in the north exterior stairs of the T.G.A. will not require a change to the SAR or Technical Specifications and will not present an unreviewed safety question. There is no change in the fire safety features of the station.
2EC-0957	Change required to assure that radiation monitor performs as intended by design. The function of the monitor has not been changed. No safety analysis are affected. No fire hazard is created - just adding process tubing to radiation sampler. Penetration through fire barriers will be resealed to maintain fire protection requirements.
2ET-0970	This procedure improves maintenance -- does not affect design.
2EC-0987	The additional check valve and downstream piping will be Seismic Class I. This new flow path will not permit flow from safety related header to non safety related.
2SC-0162	This DCR does not present an unreviewed safety question and is in no way related to any safety related system. Its operability is not essential for the operation or shutdown of the plant.