NRC FORM 366 (7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | NJSGS1 (2) | 00-00000-00 (3) | 41111 (4) | (5)
LICENSEE CODE (1–14) LICENSE NUMBER (15–25) LICENSE TYPE (26–30) CAT (57–58)

CON'T 01 REPORT SOURCE | L (6) | 05000272 (7) | 051680 (8) | 061280 (9)
DOCKET NUMBER (60–68) EVENT DATE (69–74) REPORT DATE (75–80)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While performing a routine surveillance, the level transmitter for #12 Steam
03 Generator Level Channel IV for the low level trip setpoint was out of specification.
04 Action Statement 3.3.1.1 was entered at 1139 hours. (77-15, 77-18, 77-49, 77-82,
05 78-1 and 78-18).
06
07
08

09 SYSTEM CODE IA (11) | CAUSE CODE E (12) | CAUSE SUBCODE F (13) | COMPONENT CODE INSTRU (14) | COMP. SUBCODE T (15) | VALVE SUBCODE Z (16)

(17) LER RO REPORT NUMBER: EVENT YEAR 80 | — | SEQUENTIAL REPORT NO. 026 | / | OCCURRENCE CODE 03 | REPORT TYPE L | — | REVISION NO. 0

ACTION TAKEN A (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0000 | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB. Y (24) | PRIME COMP. SUPPLIER L (25) | COMPONENT MANUFACTURER H020 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Capacitors within the circuitry had failed and were replaced with an improved design.
11 The circuit was satisfactorily calibrated and the Action Statement terminated at 1313
12 hours.
13
14

15 FACILITY STATUS E (28) | % POWER 100 (29) | OTHER STATUS (30) N/A | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Surveillance Test

16 ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) N/A | LOCATION OF RELEASE (36) N/A

17 PERSONNEL EXPOSURES NUMBER 000 (37) | TYPE Z (38) | DESCRIPTION (39) N/A

18 PERSONNEL INJURIES NUMBER 000 (40) | DESCRIPTION (41) N/A

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) N/A

20 PUBLICITY ISSUED N (44) | DESCRIPTION (45) N/A | NRC USE ONLY

NAME OF PREPARER M. J. Murphy PHONE: 609-935-0998

8006170578

GPO 917-926

Report Number: 80-26/03L

Report Date: 6/12/80

Occurrence Date: 5/16/80

Facility: Salem Generating Station
Public Service Electric & Gas Company
Hancock's Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Steam Generator Low Level Trip Setpoint Out of Specification

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1
Reactor Power 100%

DESCRIPTION OF OCCURRENCE:

While performing a routine surveillance, the level transmitter for #12 Steam Generator Level Channel IV for the low level trip setpoint (25%) was found to be out of specification. The trip setpoint was found to be at 23.75% which is outside the tolerance by 0.25%. Action Statement 3.3.1.1 was entered at 1139 hours, which states power operation may continue until performance of the next required functional test provided the inoperable channel is placed in the tripped condition within one hour.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Failure of capacitors in the channel module.

ANALYSIS OF OCCURRENCE:

During active troubleshooting, it was determined capacitors had deteriorated and caused the out of specification condition.

CORRECTIVE ACTION:

Upon investigation, it was determined the capacitors required replacement. The capacitors were replaced with an improved design and tested satisfactorily. The Action Statement was terminated at 1313 hours on May 16, 1980.

FAILURE DATA:

Capacitor Model: 39D
Manufacturer: Sprague

Prepared By M. J. Murphy

N. J. Fiduccia
Manager - Salem Generating Station

SORC Meeting No. 39-80