

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272
 UNIT Salem #1
 DATE June 10, 1980
 COMPLETED BY L. K. Miller
 TELEPHONE 609-365-7000 X507

MONTH May 1980

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

1	1113
2	1112
3	1103
4	1100
5	1074
6	1115
7	1101
8	1106
9	1098
10	1103
11	1043
12	918
13	1074
14	1100
15	1110
16	1069

DAY AVERAGE DAILY POWER LEVEL
(MWE-NET)

17	1021
18	1108
19	1088
20	1113
21	1094
22	1093
23	900
24	0
25	989
26	554
27	1086
28	1111
29	1116
30	1100
31	1113

8006160012 R

OPERATING DATA REPORT

DOCKET NO.: 50-272

DATE: June 10, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

OPERATING STATUS

1. Unit Name: Salem #1
2. Reporting Period: May 1980
3. Licensed Thermal Power (Mwt): 3338
4. Nameplate Rating (Gross MWe): 1135
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
NONE

Notes:

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons For Restrictions, If Any: NONE

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	744	3,647	25,608
12. Number Of Hours Reactor Was Critical	720	3,532.6	13,620.6
13. Reactor Reserve Shutdown Hours	0	0	22.7
14. Hours Generator On-Line	712.4	3,425.7	12,956.8
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2,320,788	10,550,702.4	38,086,999.4
17. Gross Electrical Energy Generated (MWH)	795,480	3,583,020	12,750,520
18. Net Electrical Energy Generated (MWH)	762,859	3,430,589	12,060,179
19. Unit Service Factor	95.8	93.9	50.6
20. Unit Availability Factor	95.8	93.9	50.6
21. Unit Capacity Factor (Using MDC Net)	95.0	87.2	43.6
22. Unit Capacity Factor (Using DER Net)	94.1	86.3	43.2
23. Unit Forced Outage Rate	4.2	6.1	38.2
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Refueling - 9/20/80 - 12/7/80</u>			

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	<u>09/30/76</u>	<u>12/11/76</u>
INITIAL ELECTRICITY	<u>11/01/76</u>	<u>12/25/76</u>
COMMERCIAL OPERATION	<u>12/20/76</u>	<u>06/20/77</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: May 1980

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: June 10, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-008	5/11/80	F	0	A	5	- - -	HH	FILTER	Clean #11 Condensate Pump Suction Strainers
80-110	5/12/80	F	0	A	5	- - -	HH	FILTER	Clean #11 Heater Drain Pump Suction Strainers
80-112	5/16/80	F	0	A	5	- - -	SD	VALVEX	Containment Penetration - Valve 1SS33 Malfunction
80-114	5/16/80	F	0	A	5	- - -	HH	FILTER	Change #11 Condensate Pump Suction Strainers
80-116	5/23/80	F	31.6	A	3	- - -	CH	VALVOP	#11 Steam Generator Feedwater Control Malfunction
80-118	5/25/80	F	0	A	5	- - -	HH	FILTER	#11 Heater Drain Pump Flange Leak

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure(Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error(Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

⁴
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER) File (NUREG-0161)

⁵
Exhibit I-Same Source

MAJOR PLANT MODIFICATIONS

REPORT MONTH May 1980DOCKET NO: 50-272UNIT NAME Salem 1DATE: June 10, 1980COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
LED-0033	Shielding Doors	Incore Interlock
LED-0060	CVCS	Vacuum Breakers
LED-0089	Heater Vents & Drains	Add Pipe Restraint
LED-0097A	Steam Generator Blowdown	Modify Drains
LED-0115	Analyzer Room	Replace Door
LED-0291	Heating Water	Modify LHW214 & 219
LED-0315	Fire Protection	Install Fire Rated Doors
LED-0341	Aux. Feed Room Cooler	Relocated TD-757X
LEC-0373 Package 19	Steam Generator Feed	Condensate Polishing Tie In to Existing Plant Systems
LEC-0379	Main Steam	Modify Mounting
LEC-0458A	Control Air	Install Air Receivers
LEC-0466	Fire Protection	Install Couplings
LEC-0469	TGA Structural	Install Mono Rails
2EC-0520	SW Pump Motor	Replace Cooler
LEC-0543	Pressurizer	Add Limit Switches
LED-0591	MSR	Modify 12 W MSR
LEC-0593	Pipe Support	Modify Pipe Supports
LEC-0714	Safety Injection	Remove Alarm Switch and Level Indication From LIT921 and Replace With Separate Indicating Switch
2EC-0738	Safety Related Piping	Pipe Support Changes
LEC-0739	Radiation Monitoring	Change Alarm and Warning Setpoint
LOD-0018	Chilled Water	Install Flowmeter
LOD-0024	W.D. Gas	Replace Valves and Tubing
LPD-0116	Feed and Condensate	Modify to Correct First Stage Pressure Indication
LSC-0021	Aux. Building	Reverse Wires 1&2 at TB7-20, TB8-1
LSC-0288	Non-Rad Liquid Waste	Change Discharge Setpoint

* DESIGN CHANGE REQUEST

8-1-7.R1

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MAJOR PLANT MODIFICATION

REPORT MONTH May 1980

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: June 10, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1ED-0033	Interlocking the incore cable drive reels will not increase the probability of occurrence or the consequences of an accident. It will not create a malfunction that had not been previously evaluated.
1ED-0060	The subject design change request is non safety related and does not impair any safety related system function. The definition for an unreviewed safety question is not met.
1ED-0089	Additional support is required to alleviated excessive vibration to the M.S.R. shell Nos. 15 & 16 FWH drain pipint. This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Tech. Specs. are not affected. The system is non-nuclear, non safety related.
1ED-0097A	This DCR requests rerouting blowdown sample drains from the laboratory drains to the chemical waste basin. This change does not increase the probability of or consequences of an accident; create a possibility for an accident or reduce the margin of safety as previously defined. System or piping is not safety related outside of containment.
1ED-0115	The bases of the the Technical Specifications are not affected. This is a non safety related change.
1ED-0291	This change is not safety related and does not have any adverse effects on safety related equipment.
1ED-0315	The changing of these doors does not increase the probability of occurrence or increase the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report and does not create the possibility for an accident or malfunction of a different type than evaluated previously in the safety analysis report. The changing of these doors does not affect the bases of the Tech. Specs.
1EC-0341	This change is safety related and does affect safety related systems. The purpose of this change is to relocate a thermostat so that temperature can be controlled properly within the pump room.
1EC-0373 Package #19	This design change is not safety related and does not affect any safety related system or the safe shutdown of the unit.
1EC-0379	This change does not functionally change the design of the system. This change improves the physical mounting and is in conformance with approved seismic mounting methods.

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*DCR NO.	10CFR50.59 SAFETY EVALUATION
1EC-0458A	The subject design change request is non safety related and does not impair any safety related system function. The changes will improve system performance and reliability.
1EC-0466	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Technical Specifications are not affected.
1EC-0469	Supports for trolley beams are temporary and do not affect the safety of the plant. This does not constitute an unreviewed safety question.
2EC-0520	This change does not impair safety functions, decrease safety margins or jeopardize safety analysis. These changes are being made to minimize motor cooler maintenance.
1EC-0543	The design change will serve to expand operator information only. The change does not alter the functional operation of the valves.
1ED-0591	All the changes proposed in this DCR are in non safety related systems and they do not interfere with any safety related systems, therefore, it will not affect the safety evaluation of any other system.
1EC-0593	This DCR incorporates design changes to implement the provisions of NRC Bulletin 79-02 which deals with additional design conservatism and verification of installation of pipe supports. As such, the changes covered by this DCR will not present and unreviewed safety question.
1EC-0714	This change removes the alarm switch and indication from LIT-921. It does not alter the function of the level loop nor does it affect any other safety related functions or create any new safety hazards.
2EC-0738	This DCR concerns only hanger changes as required by NRC Bulletin 79-07. All designs will not functionally alter any safety systems, nor will they create or aggravate any safety hazards.
1EC-0739	The current setpoint of the 1R31 channel is too low in that it will cause an alarm if the coolant activity is slightly higher than the <u>expected</u> equilibrium activity. The revised setpoint will be based upon the fuel leakage which is <u>permitted</u> by Technical Specifications.
10D-0018	This change is not safety related but is required for surveillance SP(O) 4.0.5P. This provides continuous flow measurement for the chilled water pumps discharge.

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*DCR NO.	10CFR50.59 SAFETY EVALUATION
10D-0024	This change is not safety related and does not affect safety related equipment.
1PD-0116	Basic system design remains unchanged. Design change will facilitate system surveillance.
1SC-0021	This change is safety related but corrects the control action for each individual valve. Circuit change if for valve heaters.
1SC-0288	This setpoint change is not safety related and does not affect any safety related systems or the safe shutdown of the unit.

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2
3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0001
5

6 WORK

7 ORDER
8 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED
9

10
11 OP0497 M VALVE, 1CV275
12

13 DESCRIPTION OF PROBLEM, SEAL WELD BONNET TO BODY.
14

15 CORRECTIVE ACTION, AS PER INSTRUCTIONS.
16

17 906018 M VALVE, 14SW005
18

19 DESCRIPTION OF PROBLEM, ERRODED SPOT ON VALVE. LINING MISSING
20 AT SAME SPOT.
21

22 CORRECTIVE ACTION, REPAIRED RUBBER LINING OF VALVE USING
23 BALZONA.
24

25 912339 M STRAINER, 11 SERVICE WATER
26

27 DESCRIPTION OF PROBLEM, PACKING LEAK.
28

29 CORRECTIVE ACTION, REPACKED.
30

31 913351 M STRAINER, 11 SERVICE WATER
32

33 DESCRIPTION OF PROBLEM, STRAINER SHAFT GLAND LEAKING BADLY.
34

35 CORRECTIVE ACTION, REPLACED WORN PACKING.
36

37 913352 M STRAINER, 11 SERVICE WATER
38

39 DESCRIPTION OF PROBLEM, STRAINER RUNNING CONTINUOUSLY WITH
40 DELTA P OF 3 PSIG. STRAINER TROUBLE ALARM
41 ANNUNCIATING.
42

43 CORRECTIVE ACTION, REPLACED DEFECTIVE PRESSURE DEVISE.
44

45 913401 M VALVE, 12SW046
46

47 DESCRIPTION OF PROBLEM, VALVE INDICATOR INDICATES VALVE CLOSED
48 WHEN VALVE IS ACTUALLY LOCKED OPEN.
49

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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0002

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6 WORK
7 ORDER
8 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED

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10
11 CORRECTIVE ACTION, TIGHTENED VAVLE INDICATOR IN OPEN
12 POSITION.

13
14 913454 M STRAINER, 15 SERVICE WATER

15
16 DESCRIPTION OF PROBLEM, 15 SERVICE WATER PUMP STRAINER DELTA P
17 GREATER THAN 15 PSIG WITH CONSTANT
18 BACKWASH.

19
20 CORRECTIVE ACTION, CLEANED STRAINERS AND INSTALLED NEW
21 PACKING.

22
23 913472 M STRAINER, 11 SERVICE WATER

24
25 DESCRIPTION OF PROBLEM, 11 SERVICE WATER PUMP STRAINER PACKING
26 GLAND LEAKING AROUND PACKING HOUSING.

27
28 CORRECTIVE ACTION, REPLACED PACKING AND GLAND.

29
30 913493 M FAN COIL UNIT, 15

31
32 DESCRIPTION OF PROBLEM, 15 FAN COIL UNIT WILL NOT START ON HIGH
33 SPEED. UNIT WILL RUN ON LOW SPEED.

34
35 CORRECTIVE ACTION, REPLACED DEFECTIVE RELAY.

36
37 913522 M FAN, 11 CONTROL ROD VENT

38
39 DESCRIPTION OF PROBLEM, FAILED TO GET RUN INDICATION. FAN IS
40 RUNNING.

41
42 CORRECTIVE ACTION, ADJUSTED LIMIT SWITCH ON DAMPER 1CAA006.

43
44 913575 M FAN, 12 FUEL HANDLING BLDG EXHAUST

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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0003
5

6 WORK

7 ORDER
8 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED
9

10 DESCRIPTION OF PROBLEM, 1 BELT OFF PULLEY AND OTHER 3 ARE SLIPPING
11

12 CORRECTIVE ACTION, REPLACED BELTS ON FAN.
13

14 913578 M COOLER, 11 COMPONENT COOLING PUMP ROOM
15

16 DESCRIPTION OF PROBLEM, ROOM COOLER LEAKING FROM UNDER-INSULATION.
17

18 CORRECTIVE ACTION, INSTALLED .125 INCH RUBBER COMPRESSION
19 PATCH OVER LEAK WITH HOSE CLAMP.
20

21 914648 M STRAINER, 13 SERVICE WATER
22

23 DESCRIPTION OF PROBLEM, SHEAR PIN BROKEN.
24

25 CORRECTIVE ACTION, REPLACED KEY AND 2 LOWER SHOES.
26

27 914659 M STRAINER, 13 SERVICE WATER
28

29 DESCRIPTION OF PROBLEM, PACKING LEAKING.
30

31 CORRECTIVE ACTION, REPACKED.
32

33 917832 M PUMP, 13 COMPONENT COOLING
34

35 DESCRIPTION OF PROBLEM, NO. 13 COMPONENT COOLING PUMP
36 HAS A INBOARD SEAL LEAK.
37

38 CORRECTIVE ACTION, REPAIRED SEAL GASKET.
39

40 919692 M VALVE, 13MS007
41

42 DESCRIPTION OF PROBLEM, AREA OF VALVE BODY UNDER SEAT EATEN AWAY.
43

44 CAUSE, EROSION/CORROSION.
45

46 CORRECTIVE ACTION, REPLACED VALVE.
47

48 921037 M VALVES, 1SJ5 AND 1SJ74
49

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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0004
5

6 WORK
7 ORDER
8 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED
9

10
11 DESCRIPTION OF PROBLEM, BOTH VALVES ARE LEAKING.
12

13 CORRECTIVE ACTION, TIGHTENED UP ON PACKING.
14

15 933909 M STRAINER, 16 SERVICE WATER
16

17 DESCRIPTION OF PROBLEM, STRAINER DIFFERENTIAL PRESSURE APPROX.
18 8 PSI WITH STRAINER CONTINUOUSLY
19 BACKWASHING.
20

21 CORRECTIVE ACTION, CHANGED PLUGGED STRAINERS.
22 RENEWED PACKING AND THRUST BEARING.
23

24 933911 M AIR COMPRESSOR, 13A DIESEL STARTING
25

26 DESCRIPTION OF PROBLEM, STARTING AIR COMPRESSOR IS LOSING
27 OIL AND OIL WAS FOUND IN STARTING AIR
28 RECEIVER.
29

30 CORRECTIVE ACTION, INSTALLED A NEW COMPRESSOR.
31

32 933913 M PUMP, 12 COMPONENT COOLING
33

34 DESCRIPTION OF PROBLEM, INBOARD PUMP BEARING HAS SLIGHT AXIAL
35 MOVEMENT.
36

37 CORRECTIVE ACTION, REPLACED OIL SEAL AND RESET THRUST
38 BEARING HOUSING.
39

40 935653 M CABLE AUDIT DISCREPANCIES
41

42 DESCRIPTION OF PROBLEM, REPAIR MISCELLANEOUS CABLE DISCREPANCIES.
43

44 CORRECTIVE ACTION, SATISFIED CABLE AUDIT INSPECTION SHEET.
45
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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0005
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6 WORK
7 ORDER
8 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED
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10
11 913441 P GENERATOR, 1C DIESEL
12

13 DESCRIPTION OF PROBLEM, DESICANT IN STARTING AIR DRIER IS WET
14 WITH OIL.
15

16 CORRECTIVE ACTION, REPLACED DESSICANT.
17 REPLACED LEFT TOWER PURGE CHECK VALVE.
18 REPLACED DESSICANT IN MOISTURE INDICATOR.
19

20 913510 P VITAL HEAT TRACE, CIRCUIT 603
21

22 DESCRIPTION OF PROBLEM, DORIC POINT 202 INDICATES 149 DEGREES
23 LOCAL PRIMARY CIRCUIT SET FOR 165 DEGREES
24

25 CAUSE, PRIMARY CAPILARY TUBE IS BROKEN.
26 ORDERED PARTS FAR PRIMARY CIRCUIT.
27

28 RAISED SETPOINT OF SECONDARY CIRCUIT BY
29 10 DEGREES.
30

31 913526 P DAMPER, 1CAA003
32

33 DESCRIPTION OF PROBLEM, DAMPER WILL NOT OPEN AND
34 SOLENOID VALVE THAT CONTROLS IT
35 IS HOT.
36

37 THREE OF FOUR LIMIT SWITCHES STICKING.
38

39 CORRECTIVE ACTION DISASSEMBLED, CLEANED, AND LUBRICATED.
40 RESET SWITCH INDICATION POINTS.
41 TESTED SATISFACTORILY.
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1/8
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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG 0006
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7 WORK
8 ORDER
9 NUMBER DEPT EQUIPMENT IDENTIFICATION EXPLANATION OF WORK PERFORMED
10

11 914688 P PUMP, 13 AUXILIARY FEEDWATER
12

13 DESCRIPTION OF PROBLEM, FAILED TO GET RUN LIGHT ON TEST RUN.
14 CAUSE, LIMIT SWITCH ON STEAM INLET VALVE OUT OF
15 ADJUSTMENT.
16
17 CORRECTIVE ACTION, ADJUSTED LIMIT SWITCH.
18

19 914695 P RADIATION MONITOR, R13E
20

21 DESCRIPTION OF PROBLEM, 15 CONTAINMENT FAN COIL UNIT RADIATION
22 MONITOR IN ALARM.
23
24 CORRECTIVE ACTION, REPLACED THE CHASSIS AND RATE DRAWER.
25

26 914825 P VALVE, 13MS007
27

28 DESCRIPTION OF PROBLEM, REGULATOR INDICATES 90 PSIG WHILE
29 WHILE OTHER VALVE (11MS7) INDICATES 32
30 PSIG.
31
32 CAUSE, REGULATOR DIAPHRAGM FAILED.
33
34 CORRECTIVE ACTION, REPLACED REGULATOR DIAPHRAGM.
35

36 920666 P VITAL HEAT TRACE, CIRCUIT 701 & 706
37

38 DESCRIPTION OF PROBLEM, BOTH HEAT TRACE CIRCUITS HAVE BROKEN
39 THERMOCOUPLES.
40
41 CORRECTIVE ACTION, REPLACED THERMOCOUPLES.
42

43 920679 P INSTRUMENT, FA3165Z
44

45 DESCRIPTION OF PROBLEM, NO FLOW INDICATION FOR NO. 12 CONTAINMENT
46 FAN COIL UNIT.
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3 SORTED BY SALEM GENERATING STATION
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG

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WORK
ORDER
NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

CAUSE, TRANSMITTER PLUGGED.

CORRECTIVE ACTION, UNPLUGGED TRANSMITTER FA3165Z.

920680 P INSTRUMENT, FA3165Z

DESCRIPTION OF PROBLEM,

NO SERVICE WATER FLOW INDICATION FOR
NO. 12 CONTAINMENT FAN COIL UNIT.

CORRECTIVE ACTION,

REPLACED FLOW TRANSMITTER AND CALIBRATED.

920726 P INSTRUMENT, 1LC527C

DESCRIPTION OF PROBLEM,

WHILE PERFORMING PROCEDURE 1PD.2.2.043 ON

12 STEAM GENERATOR LEVEL CANNEL IV, THE

TRIP POINT WAS FOUND TO BE LOWER THAN
TECHNICAL SPECIFICATION LIMITS.

CAUSE,

FAULTY POWER SUPPLY FOR COMPARATOR
CAUSING TRIPPOINT TO DRIFT TO 23.75 PERCENT
WHICH IS .25 PERCENT BELOW TECH SPEC LIMIT

CORRECTIVE ACTION,

REPLACED CAPACITORS IN POWER SUPPLY.

920733 P INSTRUMENT, 1HC500A AND 1FI500B

DESCRIPTION OF PROBLEM,

REPLACE MANUAL AUTO STATION.
SWITCHING TO MANUAL OUTPUT SAT.

CORRECTIVE ACTION,

REPLACED MANUAL AUTO STATION ON 1HC500A.
INDICATOR FI500B WAS STICKING. IT WAS
REPLACED.

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3 SORTED BY
4 DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0008

5 WORK

6 ORDER

7 NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

10
11 929894 P VALVE, 14MS018

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13 DESCRIPTION OF PROBLEM,

VALVE 14MS018 DOES NOT CLOSE ON ISOLATION
14 SIGNAL.

15
16 CAUSE,

17 FAILED SOLENOID VALVE SV0583.

18
19 CORRECTIVE ACTION,

20 REPLACED SOLENOID VALVE.

21 TOTAL LINES = 000154

22 TOTAL A-RECS = 000033

23
24 LAST UPDATE

25 800605

26 084046

27 ENTER COMMANDS

28 END OF RUN

29
30
31 @BRKPT PRINTS

SALEM UNIT #1
OPERATING SUMMARY
MAY 1980

5/1
thru Stabilized at 100% load.
5/10

5/11 At 1800 on May 11, started reducing load down to 65% to clean
thru heater drain pump strainers. At 0400 on May 12, a load increase
5/12 was begun. Load increased to 100% by 1800 on May 12.

5/13
thru Load stabilized at 100%
5/15

5/16 At 2200 on May 16, a load reduction began at approximately
thru 15% per hour because of a ruptured RCS sample line. The sample
5/17 line was isolated and a load increase from 69% began. Load reached
100% at 1300 on May 17, and remained there for the rest of the period.

5/18
thru Load stabilized at 100%.
5/22

5/23 Reactor tripped at 1954. Surveillance testing of feed regulatory
valves (operating valves in manual control). Controller was faulty.
Tripped on low steam generator level.

5/24 Reactor went critical at 1730 and when it reached 3.6% power the
reactor tripped on low steam generator level.

5/25 Started escalating power and reached 100% at 0200 on May 26. Load
thru remained at 100% for the rest of this period.
5/31

REFUELING INFORMATION

DOCKET NO.: 50-272
UNIT: Salem #1
DATE: June 10, 1980
COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000 X507

MONTH: May 1980

1. Refueling information has changed from last month:

YES _____ NO X

2. Scheduled date of next refueling: September 20, 1980

3. Scheduled date for restart following refueling: December 7, 1980

4. A. Will Technical Specification changes or other license amendments be required? YES _____ NO _____

NOT DETERMINED TO-DATE May 1980

B. Has the reload fuel design been reviewed by the Station Operating Review Committee? YES _____ NO X

If no, when is it scheduled? August 1980

5. Scheduled date(s) for submitting proposed licensing action:

September 1980 (If Required)

6. Important licensing considerations associated with refueling:

NONE

7. Number of Fuel Assemblies:

A. In-Core 193

B. In Spent Fuel Storage 40

8. Present licensed spent fuel storage capacity: 264

Future spent fuel storage capacity: 1,170

9. Date of last refueling that can be discharged to the spent fuel

pool assuming the present licensed capacity: September 1982