### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-272
UNIT	Salem #1
DATE	April 10, 1980
COMPLETED BY	L. K. Miller
TELEPHONE	609-365-7000

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Ext. 507

MONTH	March 1980	
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DAY AVERAGE DAILY POWER LEVEL

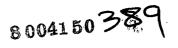
(MWe-NET) ·

1	797
2	908
3	1023
4	1117
5	1125
6	1123
7	1110
8	1084
9	894
10	1120
11	1098
12	980
13	1083
14	1105
15	1110
16	1098

DAY	AVERAGE DAILY POWER LEVEL
	(MWE-NET)
17	1081
18	1112
19	1093
20	1094
21	1113
22	1108
23	1068
24	1021
25	1101
26	1075
27	1106
28	1077
29	1072
30	1113
31	1003

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DOCKET NO .:	50-272
DATE :	April 10, 1980
COMPLETED BY:	L. K. Miller
TELEPHONE:	609-365-7000 Ext. 507

#### OPERATING STATUS

1.	Unit Name: Salem #1		Notes:
2.	Reporting Period: March	1980	
3.	Licensed Thermal Power (MWt):	3338	
4.	Nameplate Rating (Gross MWe):	1135	· · · ·
5.	Design Electrical Rating (Net M	We): 1090	
6.	Maximum Dependable Capacity (Gr	oss MWe): 1124	
7.	Maximum Dependable Capacity (Ne	t MMe): 1079	

NONE

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason: NONE

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any: NONE

This Month Year to Date Cumulative 2,184 744 24,145 11. Hours In Reporting Period 12,181.6 2,093.6 744 12. Number Of Hours Reactor Was Critical 0 22.7 0 13. Reactor Reserve Shutdown Hours 11,525.4 744 1,994.3 14. Hours Generator On-Line. ō 0 15. Unit Reserve Shutdown Hours 0 2,405,018 5,959,980 33,496,277 16. Gross Thermal Energy Generated (MWH) 823,870 2,012,600 11,180,100 17. Gross Electrical Energy Generated (MWH) 1,923,293 10,552,883 792.307 18. Net Electrical Energy Generated (MWH) 91.3 100.0 47.7 19. Unit Service Factor 91.3 100.0 47.7 20. Unit Availability Factor 98.7 81.6 21. Unit Capacity Factor (Using MDC Net) 40.5 97.7 80.8 40.1 22. Unit Capacity Factor (Using DER Net) 0 8.7 40.9 23. Unit Forced Outage Rate

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling - 9/20/80

25.	If Shut Down At	End of Report Period, Estimated Date of Startup:	N/A		
26.	Units In Test St	atus (Prior to Commercial Operation):		· · · · ·	
		-	Forecast	Achieved	
		INITIAL CRITICALITY	0 <u>9/30/7</u> 6	1 <u>2/11/7</u> 6	
8-1	7 10	INITIAL ELECTRICITY	1 <u>1/01/7</u> 6	1 <u>2/25/7</u> 6	
	3 of 20	COMMERCIAL OPERATION	1 <u>2/20/7</u> 6	0 <u>6/20/7</u> 7	

### UNIT SHUTDOWNS AND POWER REDUCTIONS

### REPORT MONTH March 1980

### DOCKET NO.: 50-272 UNIT NAME: Salem #1 DATE: April 10, 1980 COMPLETED BY: L. K. Miller TELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-062	3/1/80	F	0	А	5		нн	FILTER	Clean Cond. Pump Suction Strainers
80–064	3/2/80	F	0	· A	5		HG	DEMINX	High Differential Pressure Condensate Polishers
80–066	3/3/80	F	0	A	5		HH	PUMPXX	13A Vacuum Pump Tagged Out For Repairs
80-068	3/6/80	F	0	· A	5		нн	HTEXCH	Clean 13B Condenser Water Box
80-070	3/7/80	F	0	A	5		HH	HTEXCH	Clean 13B Condenser Water Box
80-072	3/9/80	F	0	A	5		HH .	FILTER	Clean Cond. Pump Suction Strainers
80-073	3/13/80	F	0	А	5		HH	HTEXCH	Clean 13A Condenser Water Box
80-074	3/12/80	F	0	A	5		HH	FILTER	Clean Cond. Pump Suction Strainers
80-076	3/16/80	F	0	A	5		НН	HTEXCH	Clean 13B Condenser Water Box
80-078	3/16/80	F	0	A	5		нн	HTEXCH	Clean 13A Condenser Water Box
80-079	3/18/80	F	0	А	5		HH	HTEXCH	Clean 13B Condenser Water Box
80-081	3/18/80	F	0	. A	5		нн	HTEXCH	Clean 13B Condenser Water Box

1 F: S:

: Forced : Scheduled	2 Reason: A-Equipment Failure(Explain) B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Administrative G-Operational Error(Explain) H-Other (Explain)	<pre>3. Method: 1-Manual 2-Manual Scram. 3-Automatic Scram. 4-Continuation of Previous Outage 5-Load Reduction 0.011</pre>	4	Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER) File (NUREG-0161)	5 Exhibit 1-Same Source
		9-Other			

### UNIT SHUIDOWNS AND POWER REDUCTIONS

REPORT MONTH March 1980

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DOCKET NO.: 50-272 UNIT NAME: Salem #1 DATE: April 10, 1980 COMPLETED BY: L. K. Miller TELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUITING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE5	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-083	3/23/80	F	0	Ä	5	- <b>-</b> -	НН	НТЕХСН	Clean 11B Condenser Water Box
80-085	3/23/80	F	0	А	5		нн	FILTER	Clean Condenser Pipe Strainers
80-086	3/26/80	F	0	А	5		нн	HTEXCH	Clean 13A Condenser Water Box
80-088	3/31/80	F	0	A	5		нн	FILTER	Clean Cond. Pump Suction Strainers
	5 of 2								

Page 5 of 20 8-1-7.R1 MAJOR PLANT MODIFICATION REPORT MONTH March 1980 DOCKET NO: 50-272 UNIT NAME: Lem 1 DATE: April 10, 1980 COMPLETED BY: L. K. Miller TELEPHONE: 609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ED-0062	SGFP	Replace 11 & 12BF32 · Valves
1ED-0137	Hydrogen Supply	Relocate the Hydrogen Storage
lED- 0146 Package 2	"B" Building Services	Install and Relocate Services
1ED-0241A	Fire Protection	Additional Fire Protection to Containment and Auxiliary Building
led-0300	Station Lighting	Installation of Yard Lighting
1ED-0304	Ventilation	Automatic Damper in Electrical Pen.
1ED-0311	CVCS	Core Bore Monitor Holes in Shield Plug
1ED-0314	Fire Protection	Install Hose Reel in North Pen.
1ED-0334	WDL	Waterproof WDL Trench-Drum Storage Area
1ED-0350A	WDL	Waste Evaporator Piping Mod.
lED-0354- Package l	Fire Protection	Expand Fire Protection
1EC-0373- Package 2	Condensate	Condensate Polishing Condenser Tie-I
1EC-0373- Package 15	Condensate	Condensate Polishing Controls
lEC-0373- Package 16		Condensate Polishing Elect. Tie-In
1EC-0377	Fire Protection	Provide Alarms on Fire Boundry
1EC-0442	Containment Hatch	Modify Containment Personnel Hatch
1EC-0451	Safety Injection	CMC Switch on 1SJ67 and 68
lEC-0474	RMS	Reset IR11A and IR12A
1EC-0480	Fire Protection	Install Additional Fire Detection

\* DESIGN CHANGE REQUEST 8-1-7.Rl Page 6 of 20 MAJOR PLANT MODIFICATION

REPORT MONTH March 1980

DOCKET NO: 50-272 UNIT NAME: Lem 1 DATE: April 10, 1980

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COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-0492	Aux Building Vent.	Install Hi Temperature Alarm
lEC-0534	Solid State Protection	Delete Low Pressure Level Coincidence Circuit
1EC-0536	Aux Feedwater	Modify Controls
1EC-0565	Non Rad Waste	Construct Baffle and Install
lEC-0590	CVCS	Install Snubber on CVCS Charging Pump
1EC-0662	Stress Calculation	Seismic Stress Calculations
1EC-0697	Aux Feedwater	Aux Feedwater Hanger Change
1MD-0081	Met Tower	Installed Alarm in Control Room
1MD-0131	Telephone	Permanent Telephone in Containment
1MD-0132	Fire Protection	Core Bore Holes at Fire Doors
10D-0035	WDL	Add Two Pumps and Associated Piping
1PD-0075	DM Water	Install Heat Trace on DM Water
1PD-0124	Nitrogen	Core Bore Monitor Holes in Shield Plug
1PD-0141	Communications	Install PA at Chlorine House
1PD-0162- Package 1	Non Rad Waste	Additional Heat Tracing
1SC-0052	РА	Install PA in South Penetration
1SC-0073	TGA - Structural	Installed Condensate Pump Platform
1\$C-0081	DM Water	DM Make-Up Water to VAC Pumps
1SC-0097	Pressurizer	Installed Safety Ladder Cages
1SC-0122	Circ. Water	Additional Cathodic Protection
2SC-0215	Spent Fuel Cooling	Install Chain Operator on 2-SF-35

\* DESIGN CHANGE REQUEST 8-1-7.R1

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MAJOR PLANT MODIFICATIONS REPORT MONTH\_\_\_\_\_March 1980

DOCKET NO :	50-272
UNIT NAME	Salem #1
DATE:	April 10, 1980
COMPLETED BY:	L. K. Miller
TELEPHONE:	609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION	
1ED-0062	Not a safety related change.	
1ED-0137	This design change involves the relocation of H <sub>2</sub> storage bottles from the interior to the exterior of a safety related structure (the Auxiliary Building). The potential for or consequences of this change affecting plant safety are non-existant.	
1ED-0146 Package 2	There is notifundamental change to seal design or function nor to No. 1 & 2 seals. The change enhances service life. The requirements for an unreviewed safety question are not met.	
1ED-0241A	This design change does not affect any presently performed safety analysis nor does it create any new hazards. The bases of the Technical Specifi- cations are not affected.	
1ED-0300	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CRF50.59.	
1ED-0304	The operation of safety related equipment associated is not affected.	
1ED-0311	Modification of the shield plugs over the CVC demineralizers will not affect the safe operation or shutdown of the plant. The access holes drilled in the shield plugs will be covered with an equivalent amount of steel shielding to maintain design radiation levels at the outer surface of the concrete shield plug. This change has no effect on the Tech. Specs.	
1ED-0314	The bases of the Tech. Specs. are not affected by the addition of this hose station. This hose station is to be added to the list of hose stations in the Technical Specifications. Implementation of this DCR may proceed prior to submittal of License Change Request No. 78-10.	
1ED-0334	Applying waterproofing to areas in the liquid waste disposal trench and the drum storage vault where seepage is suspected does not alter any approved design criteria, procedures or setpoints.	
1ED-0350A	Minor revision to provide construction opening and temporary enclosure. This does not alter any approved design criteria, procedures or setpoints.	
1ED-0354 Package 1	This DCR will not affect any safety function of the system involved. The proposes changes downot involve any of the criteria associated with an unresolved safety question per 10CFR50.59.	
1EC-0373 Package <sub>.</sub> 2	Change involves addition of non safety related valves. This change won't affect any safety related function nor require safety analysis or margin of safety stipulated in the Tech. Specs. No unreviewed safety question is involved for this mechanical portion of the DCR.	
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MAJOR PLANT MODIFICATIONS

REPORT MONTH

DOCKET NO	50-272
UNIT NAME	Salem #1
DATE:	April 10, 1980
COMPLETED BY:	L. K. Miller
• TELEPHONE:	609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1EC-0373 Package 15	This design change is not safety related and does not affect any safety related systems or the safe shutdown of the unit.
1EC-0373 Package 16	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR50.59.
1EC-0377	This change expands the present fire protection system to meet some requirements of Branch Tech. Position 9.2, Appendix A. It is not safety related and does not require a revision to any Tech. Spec. No additional safety analysis need be performed.
1EC-0442	Worn internal parts of containment personnel air lock doors are to be replaced by new parts under the supervision of original vendor. No change in design or parts or operation of air locks will take place. They will be rebuilt to their original condition and therefore, no unresolved safety question is involved as the finished condition will be as indicated in existing Tech. Specs., design drawings, Engineering Specs. and FSAR.
1EC-0451	This change will satisfy the simple failure criteria required for safety related equipment and will increase the margin of safety operation. No additional safety analysis required; no previous safety evaluation will be violated.
1EC-0474	The 1R11A and 1R12A provide a safety related function in isolating the containment on high activity due to primary coolant leakage, however, the ambient radiation levels in the containment are known to increase with time. If the leakage in the containment is known to be within specifi- cation, then the setpoint may be altered to twice the background value. The setpoint change will not be altered such that an excessive amount of activity could be discharged before the channels isolate the containment. (See Engineering instructions section for further explanation on why this change does not constitute and USQ).
1EC-0480	This design change provides fire detection for areas not previously protected. It will interface with and complement the existing fire detection system. It will not affect the function or operation of any equipment involved with the safe shutdown of the plant.
1EC-0492	The installation of temperature monitoring devices will improve overall plant reliability by alerting the Control Room Operator to potentially elevated ambient temperature conditions in areas where equipment vital to plant operation and safety is located. By using these devices, action may be taken vefore any postulated effect on safety related equipment occurs. This installation is an NRC requirement.

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MAJOR PLANT MODIFICATIONS

REPORT MONTH March 1980

DOCKET NO .:	50272
UNIT NAME	Salem #1
DATE:	April 10, 1980
COMPLETED BY:	L. K. Miller
TELEPHONE:	609-365-7000 X507
-	

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1EC-0534	Logic change complies with NRC positions regarding use of pressurizer level coincidence circuitry. Change improves reliability of S.I. function for small breaks in pressurizer steam space.
1EC-0536	Logic change complies with NRC question regarding operator flexibility to initiate auxiliary feedwater.
1EC-0565	Installation of weir wall in Non-radioactive Waste Basin will not affect any previously performed safety analysis, will not affect the safe operation of the plant and does not constitute an unreviewed safety question.
1EC-0590	The addition of 1A-CVCVE-427A does not present an unreviewed safety question. The vibration eliminator will reduce vibration of 8" suction line without need for modifying the pipe configuration. A change to both the SAR and Technical Specifications are not involved.
1EC-0662	This DCR is issued to authorize changes on safety related piping systems such as pipe hardware modifications, document updating to reflect all "as built" conditions as required by NRC Bulletin I&C 79-14 to verify seismic inputs. Since the majority of changes are minor support modifi- cations, document updating, etc., there is no unreviewed safety question involved.
1EC-0697	This DCR incorporates design changes to implement provisions of NRC Bulletin IE 79-07 which requires utilities to reanalyze piping systems using algebraic summation to combine earthquake loads. This DCR does not present any unreviewed safety question as a result of changes to pipe support and installations.
1MD-0081	Not safety related and does not affect any safety related systems.
1MD-0131	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR50.59.
1MD-0132	Providing core bore holes with removal fire seal plugs does not change the SAR or Tech. Specs. or present and unreviewed safety question. There is no change to the safety features which protect the plant from fire hazards.
10D-0035	This design change pachage is not safety related and does not in any way affect any safety related systems or the safe shutdown of the plant.
1PD-0075	This DCR will not affect the safety function of any system. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR50.59.

# MAJOR PLANT MODIFICATIONS

REPORT MONTH March 1980

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DOCKET NO	50-272
UNIT NAME	Salem #1
DATE:	April 10, 1980
COMPLETED BY:	DL. K. Miller
TELEPHONE:	609-365-7000 X507
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*DCR NO.	10CFR50.59 SAFETY EVALUATION
1PD-0124	This change is not safety related and does not affect safety related equipment.
1PD-0141	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR50.59.
1PD-0162 Package 1	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR50.59.
1SC-0052	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR50.59.
1SC-0073	Installation of platforms to condensate discharge isolation valves will not affect any previously performed safety analysis, will not affect the safe operation of the plant and does not constitute an unreviewed safety question.
1SC-0081	The modification to this system does not affect any presently performed safety analysis nor does it create any new hazards. This system is not a safety related system. The bases of the Technical Specifications remains unchanged.
1SC-0097	Installation of cage around ladder to pressurizer penthouse will not change any previously performed safety analysis, will not affect the safe operation of the plant and does not constitute an unreviewed safety question.
1SC-0122	This DCR involves additional cathodic protection on condenser water boxes and is not safety related. Criteria for an unreviewed safety question are not met.
2SC-0215	The addition of a chain operator to a valve does not affect the design philosophy of the system. This modification does not affect any present analysis nor does it create any new hazards. The Tech. Specs. are not affected.
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WORK ORDER			
NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
902972	. <b>M</b>	FLANGE, SERVICE WATER	
	•	DESCRIPTION OF PROBLEM,	30 INCH FLANGE AT EXPANSION JOINT FC-3-41 IN BAY <b>3</b> ERRODED
		CORRECTIVE ACTION,	CLEANED ERR <b>OSION, WELDED, AND COVERED THE</b> FACES OF THE FLANGES WITH BITUMASTIC-300-M
905377	м	PUMP, SERVICE WATER BAY 4 SUMP	
		DESCRIPTION OF PROBLEM,	BAY 4 SUMP PUMP WILL NOT RUN.
•		CAUSE,	MOISTURE FOUND IN CONDUIT.
		CORRECTIVE ACTION,	DRIED OUT WIRING AND REPLACED SUMP FUMP.
911506	м	VALVE, 14MS171	
		DESCRIPTION OF PROBLEM,	BLOWING THROUGH.
		CAUSE,	SEATING SURFACES CUT BY STEAM.
		CORRECTIVE ACTION,	REPLACED VALVE STEM AND SEAT.
911536	м	VALVE, 115W024	
		DESCRIPTION OF PROBLEM,	BOLTS ARE LOOSE ALL THE WAY AROUND
		CORRECTIVE ACTION,	TIGHTENED BOLTS.
914291	м	FIPE HANGER, CTAG-807	
		DESCRIPTION OF PROBLEM,	REJECTED RICHMOND ANCHOR FOUND DURING WALKDOWN VERIFICATION.
·		CORRECTIVE ACTION,	WELDED TWO PIECES OF CHANNEL TO HANGER INSTALLED TWO KWIK-BOLTS.
914391	M	PUMP, 11 CHILLED WATER	
		DESCRIPTION OF PROBLEM,	CHILLED WATER PUMP RUNNING HOT. BINDING DURING TEST RUN.
		CAUSE,	PUMP WAS RUN WITHOUT OIL.
		CORRECTIVE ACTION,	REPLACED BEARINGS.
916610	M .	VITAL HEAT TRACE, CIRCUIT CONTROLLER T204S	
		DESCRIPTION OF PROBLEM,	GREEN POINTER( <b>AL</b> ARM) INOPERABLE. SET AT 35 DEGF <b>AN</b> D CAN NOT BE RAISED.
		CAUSE,	FOINTER (ALARM) BROKEN
ge 12 of	20	CORRECTIVE ACTION,	REFLACED.

WORK ORDER			
NUMBER	DEPT	EQUIPMENT IDENTIFICATI <b>ON</b>	EXPLANATION OF WORK PERFORMED
916662	м	STRAINER,13 SERVICE WATER	
		DESCRIPTION OF PROBLEM,	PACKING LEAKING
		CORRECTIVE ACTION,	REPLACED PACKING
916663	M	PUMP, 12 COMPONENT COOLING	
		DESCRIPTION OF FROBLEM,	EXCESSIVE OIL LEAK AT OUTBOARD BEARING
		CORRECTIVE ACTION,	REFLACED GASKET ON THRUST BEARING.
916679	м	VITAL HEAT TRACE, CIRCUIT 702	
. •		DESCRIPTION OF PROBLEM,	UNABLE TO MAINTAIN CORRECT TEMPERATURE.
		CORRECTIVE ACTION,	REPLACED THERMOSTAT. REPLACED PRIMARYHHEAT TRACE.
916689	м	TRAVELLING SCREEN, 12 SERVICE WATER	
		DESCRIPTION OF PROBLEM,	SHEAR FIN BROKEN
		CORRECTIVE ACTION,	REPLACED PIN.
916698	м	INSTRUMENT, PT6298	
		DESCRIPTION OF PROBLEM,	12 COMPONENT COOLING FUMP SUCTION GAUGE HAS EXCESSIVE MOVEMENT WHEN FUMP IS IN SERVICE.
		CORRECTIVE ACTION,	SECURED GAUGE BY USING A BACKING PLATE BEHIND GAUGE AND BOLTING IT TO AN EXISTING ANGLE IRON.
916701	м	DAMPER, 1CAA003	
		DESCRIPTION OF PROBLEM,	DAMPER 1CAA003 WILL NOT DPEN.
		CAUSE,	LUBRICATION PROBLEM.
		CORRECTIVE ACTION,	LUBRICATED AND FREED DAMPER.
916762	м	PUMP, 11 CHILLED WATER	
		DESCRIPTION OF PROBLEM,	THERMAL OVERLOAD IN BREAKER KEEPS TRIP- PING AFTER PUTTING THE PUMP IN SERVICE APPROXIMATELY THREE HOURS
		CORRECTIVE ACTION,	REPLACED OVERLOADS. TIGHTENED ALL CON- NECTIONS. TESTED SAT.
918933	м	DAMPER, 1CAAOO1	
		DESCRIPTION OF PROBLEM,	DAMPER 1CAA001 WILL NOT OPEN.
		CAUSE,	LUBRICATION PROBLEM.
Page 13 of	20	CORRECTIVE ACTION,	USED PENETRATING OIL TO FREE DAMPER.

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USED PENETRATING OIL TO FREE DAMPER.

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ORDER	
NUMBER	

DEPT

WORK

927722 STRAINER, 11 SERVICE WATER PUMP Μ DESCRIPTION OF PROBLEM, PACKING LEAK. CORRECTIVE ACTION, REPACKED. 927889 м DAMPER, 1CAA003 DESCRIPTION OF PROBLEM, DAMPER 1CAA003 WILL NOT OPEN. CAUSE, SHAFT JAMMED. CORRECTIVE ACTION, FREED SHAFT. 927893 м COMPRESSOR, 2 EMERGENCY AIR DESCRIPTION OF PROBLEM, ON LOW STATION AIR HEADER PRESSURE, EMERG AIR COMPRESSOR RAN FOR THREE MINUTES AND TRIPPED ON LOW LUBE PRESSURE CAUSE, INVESTIGATION SHOWED CRANKCASE FILLED WITH WATER. SOURCE OF WATER UNKNOWN. CORRECTIVE ACTION, CLEANED SUMP, BEARING, AND OIL PUMP. FLUSHED WITH OIL TWICE AND INSTALLED A NEW FILTER. . 927894 PUMP, CONTAINMENT SUMP м DESCRIPTION OF PROBLEM, EXCESSIVE RUN TIMES. CAUSE, DEBRIS IN SUMP. CORRECTIVE ACTION, CLEANED SUMP AND STRAINERS. 927906 ENGINE, 1A DIESEL M DESCRIPTION OF PROBLEM, EMERGENCY DIESEL WILL NOT SHUTDOWN NORM-ALLY. EMERGENCY STOP MUST BE USED. CAUSE . LOOSE WIRE ON SDR RELAY CORRECTIVE ACTION, REPAIRED WIRE. 927946 м VITAL HEAT TRACE, CIRCUIT 603 DESCRIPTION OF PROBLEM, SECONDARY CIRCUIT UNABLE TO MAINTAIN TEMPERATURE ABOVE 145 DEGF WITH PRIMARY CIRCUIT OUT OF SERVICE DUE TO LACK OF PARTS CAUSE, HEAT TRACE BURNED OPEN. Page 14 of 20 CORRECTIVE ACTION, REPAIRED AND REINSULATED HEAT TRACE.

WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATI <b>on</b>	EXPLANATION OF WORK PERFURMED
928017	м	ELECTRICAL SEALS, #1 CONTROL ROOM .	
		DESCRIPTION OF PROBLEM,	CHECK ALL ELECTRICAL SEALS FROM #1 CONTROL ROOM TO #1 RELAY ROOM.
		CORRECTIVE ACTION,	FOUND ONE DAMAGED SEAL IN 1RP3. REPAIRED SEAL USING 3 802. SEMKIT
928178	м	VALVE, 11MS168	
		DESCRIPTION OF PROBLEM,	BONNET LEAK.
		CORRECTIVE ACTION,	FURMANITED TIGHTENED PACKING NUTS.
928194	м	PUMP, 12 COMPONENT COOLING	
		DESCRIPTION OF PROBLEM,	HIGH VIBRATION NOTICED BY SITE PERSONNEL.
		CAUSE,	WORN BEARINGS
		CORRECTIVE ACTION,	REPLACED MECHANICAL SEAL, OIL SEAL, AND BEARINGS.
928200	M	FIRE SEALS, CONTROL ROOM	
		DESCRIPTION OF PROBLEM,	INSPECT AND REPAIR CONTROL ROOM FLOOR FIRE SEALS
		CORRECTIVE ACTION,	REPAIRED HOLES'IN 1RP4 AND 1RP5
929418	М	VALVE OPERATOR, 148F019	
		DESCRIPTION OF PROBLEM,	CONSOLE INDICATES 50 PERCENT WHILE LOCAL INDICATOR READS 70 PERCENT, LEVEL CONTROL ERRATIC PERIODICALLY GETTING 10 PERCENT LEVEL SWINGS,
		CAUSE ,	FAILURE DUE TO NORMAL DEGRADATION OF VALVE OPERATOR.
		CORRECTIVE ACTION,	CALIBRATED POSITIONER, REPLACED CAM AND CAM HUB AND CHECKED LIMIT SWITCHES, STROKED VALVE,
933704	M	COMPRESSOR, 11 WASTE GAS	
		DESCRIPTION OF PROBLEM,	EXCESSIVE NOISE AND WATER LEAKING FROM SEALS.
		CORRECTIVE ACTION,	INSTALLED REBUILT COMPRESSOR.
933717	м	PIFE, SERVICE WATER	1
i.		DESCRIPTION OF PROBLEM,	PIPE LEAK NEAR 125W124.
,		CAUSE,	ERRUSION/CORROSION.
Page 15 of	E 20	CORRECTIVE ACTION,	INSTALLED NEW PIPE AND FLANGE.

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WORN ORDER NUMBER	DEFT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
933740	м	OVERHEAD ALARM, 12 RHR SUMP OVERFLOW	
	•	DESCRIPTION OF PROBLEM,	12RHR SUMP OVERFLOW ALARM ANNUCIATED WITH NO OVERFLOW CONDITION.
		CAUSE	GROUNDED WIRE ON MICROSWITCH.
		CORRECTIVE ACTION,	REINSULATED WIRE.
933754	м	DAMPER, 1CAA003	
•		DESCRIPTION OF PROBLEM,	CONTROL ROOM INTAKE DAMPER 1CAA003 FROZEN
		CAUSE	LUBRICATION PROBLEM.
		CORRECTIVE ACTION,	LUBRICATED DAMPER. TEST SATISFACTORY.
933760	м	FLANGE, SERVICE WATER	
		DESCRIPTION OF PROBLEM,	FLANGE LEAK <b>in se</b> rvice water structure Bay 3
		CORRECTIVE ACTION,	REPLACED GASKET
933761	м	VALVE, 145W024	
		DESCRIPTION OF PROBLEM,	VALVE 145W024 HAS A RUPTURED DIAPHRAGM.
		CORRECTIVE ACTION,	REPLACED DIAPHRAGM.
933807	м	VALVE, 115J134	N
		DESCRIPTION OF PROBLEM,	PACKING LEAK
		CORRECTIVE ACTION,	ADJUSTED PACKING. Cleaned Bonnet Stud and Packing Nuts.
933825	м	STRAINERS, 16 SERVICE WATER FUMP	
		DESCRIPTION OF PROBLEM,	SHEAR PINS BROKEN.
		CAUSE ,	SEAL PLATES WORN
		CORRECTIVE ACTION,	REPLACED 3 SEAL PLATES AND INSTALLED NEW Shear pin
935518	М	PUMP, 13 COMPONENT COOLING	
		DESCRIPTION OF PROBLEM,	HIGH VIBRATION NOTICED BY SITE PERSONNEL
		CAUSE,	WORN THRUST BEARING.
Page 16 of	20	CORRECTIVE ACTION,	REPLACED THRUST BEARING AND INBOARD BEARING HOUSING.

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• WORK ORDER				
NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFURMED	
935533	. M	STRAINER, 12 SERVICE WATER		
		DESCRIPTION OF PROBLEM,	HAND HOLE LEAKING	
		CORRECTIVE ACTION,	REPLACED GASKET	
911416	P	FILTER, AIR PARTICULATE DETECTOR		
		DESCRIPTION OF PROBLEM,	FILTER FAILURE ALARM RECEIVED FOR AIR PARTICULATE DETECTOR.	
		CAUSE,	SHEAR FINS ON DRIVE MECHANISM BROKEN. MICRO SWITCH HAD ALSO FAILED. FAILURE OF SWITCH THOUGHT TO BE RESULT OF MISHANDLING DURING SHEAR PIN REPAIR.	
		CORRECTIVE ACTION,	REPLACED SHEAR PINS AND MICROSWITCH.	
914476	P	INSTRUMENT, R31B		
		DESCRIPTION OF PROBLEM,	FUEL PROTECTION-RATIO LOW ALARMS WITHOUT CAUSE	
	•.	CORRECTIVE ACTION,	REPLACED PC BOARD.	
914491	P	INSTRUMENT, FA5694		
		DESCRIPTION OF PROBLEM,	FALSE INDICATIONS	
		CAUSE ,	FAILED MIDRANGE.	
		CORRECTIVE ACTION,	REPLACED AMPLIFIER BOARD.	
914559	P	SWITCH, BIT TANK RECIRC LOW FLOW ALARM	۰ ۱	
		DESCRIPTION OF PROBLEM,	IS ON WITH GREATER THAN IO GPM RECIRC	
		CAUSE,	DEFECTIVE REED SWITCH	
		CORRECTIVE ACTION,	SWITCH DESTROYED AND REPLACED.	
921020	P	INSTRUMENT, TA4554Z	•	
		DESCRIPTION OF PROBLEM,	REACTOR COOLANT LOOP 11 CHANNEL DELTA T / TAVG OUT OF CALIBRATION	
		CAUSE,	SIGNAL ISOLATOR OUT OF SPECIFICATION	
		CORRECTIVE ACTION,	REPLACED SIGNAL ISOLATOR. OLD ISOLATOR TO BE REWORKED.	
921100	P	INSTRUMENT, 1LC459		
		DESCRIPTION OF PROBLEM,	SIGNAL COMPARATOR OUT OF SPECIFICATIONS.	
Page 17 of	20	CORRECTIVE ACTION,	REPLACED COMPARATOR, OLD COMPARATOR TO BE REWORKED	

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REWORKED

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## SALEM UNIT #1 OPERATING SUMMARY MARCH 1980

- 3/1 Escalating power at 5% per hour. At 2000, power reached 100%. Reduced power at 2355 to 70% to allow cleaning of condensate strainers.
- 3/2 At 0500, reached 70% power to clean condensate strainers. Started increasing load from 70% at 0625 until 1430 when load reached 100%. At 1430, started decreasing load to 80% to clean heater drain pump strainers. Power remained at 80% for the remainder of the day.
- 3/3 At 0001, load was increased until 0930 when load reached 97%. Load remained there for the remainder of the day.
- 3/4 Load remained at 97% for the entire day.
- 3/5Load was increased from 97% to 100% at 0830 on 3/5/80. The loadthruremained at 100% for the rest of this period.

3/8

3/9 At 0100, started load decrease to 70% to allow cleaning of condensate pump strainers. At 0110, started load increase until 2300 when load reached 100%.

3/10

thru Remained at 100% load for this period.

3/11

3/12 Started decreasing load at 0550 to 70% to allow cleaning of condensate pump strainers. Started increasing load at 1300 until 2300 when load was at 100%.

thru Remained at 100% load for this period.

3/22

3/13

3/23 At 100% load until 2100 when started decreasing load to 70% to thru allow cleaning of condensate pump strainers. Reached 70% at 0100 3/24 on 3/24/80. Started increasing load at 0200 until 100% load was reached at 2000. Remained at 100% load for the remainder of this period.

3/25 Remained at 100% load for this period.

3/26 At 0300, started reducing load to 86% in preparation to clean 13B condenser. Load was increased to 100% by 0800.

3/27 Load at 100% for this period except for 5% load reduction at 1509 thru on 3/28/80 due to drop in steam generator feed pump suction pressure. 3/29 Load returned to 100% at 1630 on 3/28 and remained there for the rest of this period.

3/30 At 100% load until 2150 when load reduction began at 7% per hour thru down to 70% to allow cleaning of condensate pump strainers. Load 3/31 increase began at 0500 on 3/31/80 until 100% load was reached at 1130. Load remained at 100% for the rest of this period.

UELING INFORMATION R

	DOC	CKET NO .:_	50-272
			Salem #1
		-	April 10, 1980
	COM	-	L. K. Miller
		TELEPHONE :	609-365-7000
			Ext. 507
MON	TH: March 1980		
1.	Refueling information has changed from last mo	onth:	
	YESNO	<u>x</u>	
2.	Scheduled date of next refueling:	er 20, 1980	
3.	Scheduled date for restart following refueling	J:Decen	ber 7, 1980
4.	A. Will Technical Specification changes or of	ther licer	ise
	amendments be required? YES NO	D	•
	NOT DETERMINED	IO-DATE	March 1980
	B. Has the reload fuel design been reviewed b	by the Sta	ation Operating
	Review Committee? YES NO	<u> </u>	
	If no, when is it scheduled? August 1	980	
5.	Scheduled date(s) for submitting proposed lice	ensing act	ion:
	September 1980	(If Requir	ed)
6.	Important licensing considerations associated NONE	with refu	leling:
		······································	
7.	Number of Fuel Assemblies:		
	A. In-Core	193	
	B. In Spent Fuel Storage	40	
8.	Present licensed spent fuel storage capacity:	264	
	Future spent fuel storage capacity:	1,170	
9.	Date of last refueling that can be discharged	to the sp	ent fuel
	pool assuming the present licensed capacity:	September	

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