#### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272 UNIT Salem #1 DATE February 11, 1980 COMPLETED BY L. K. Miller TELEPHONE 609-365-7000 X 507

MONTH	January 1980		
DAY A	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY AVER	(MWE-NET)
1 _	342	17	924
2 _	319	18	943
3	369	19	1,047
4	370	20	833
5	515	21	1,069
6	598	22	1,092
7	507	23	379
§	763	24	96
9	788	25	856
10 _	980	26	1,069
11	1,100	27	1,084
12	908	28	1,021
13	1,023	29	1,048
14	299		1,114
15	0	31	1,114
16	630	· 	

Pg. 2 of 18 8.1-7.R1

8002150 270

#### OPERATING DATA REPORT

DOCKET NO.: 50-272

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 365-7000 X507

#### OPERATING STATUS

1. Unit Name: Salem #1	Notes:		
2. Reporting Period: January 1980			
3. Licensed Thermal Power (MWt): 3338			
4. Nameplate Rating (Gross MANe): 1135			
5. Design Electrical Rating (Net MWe): 109	90		
6. Maximum Dependable Capacity (Gross MWe):	1124		
7. Maximum Dependable Capacity (Net MNe):	1079		
8. If Changes Occur in Capacity Ratings (Items NONE	Number 3 Through 7) Since	Last Report, Give R	eason:
9. Power Level To Which Restricted, If Any (Ne	- 11.0/ -	ONE	·
.O. Reasons For Restrictions, If Any: NOI	NE 	<u></u>	
		<u> </u>	
			<del></del>
	This Month	Year to Date	Cumulative
	744	744	22,705
1. Hours In Reporting Period	696.4	696.4	10,784.4
2. Number Of Hours Reactor Was Critical		0	22.7
3. Reactor Reserve Shutdown Hours	0 681.7		10,212.8
4. Hours Generator On-Line	001.7	681.7	0
5. Unit Reserve Shutdown Hours	1,789,716	1,789,716	29,326,013
6. Gross Thermal Energy Generated (MWH)	586,080	586,080	
7. Gross Electrical Energy Generated (MWH)		556,282	9,753,580
8. Net Electrical Energy Generated (MWH)	556,282 91.6	91.6	9,185,872 45.0
9. Unit Service Factor	91.6	91.6	45.0
O. Unit Availability Factor Unit Capacity Factor (Using MDC Net)	69.3	69.3	37.5
2. Unit Capacity Factor (Using DER Net)	68.6	68.6	37.1
3. Unit Forced Outage Rate	8.4	8.4	43.4
24. Shutdowns Scheduled Over Next 6 Months (Typo	e. Date. and Duration of E		
NONE	c, ma, un bulación de la		
	<del></del>		<del></del>
			<del> </del>
5. If Shut Down At End of Report Period, Estima	ated Date of Startup:	N/A	
6. Units In Test Status (Prior to Commercial Ope	eration):	Forecast	Achieved
INITIAL CRITICALITY		9/30/76	12/11/76
INITIAL ELECTRICITY		11/1/76	$\frac{12/11}{70}$
G-1-7.R2 COMMERCIAL OPERATION		$\frac{12/20/76}{12}$	6/20/77

### UNIT SHUTDOWNS AND POWER REDUCTIONS

January 1980 REPORT MONTH

50-272 DOCKET NO .:

Salem #1 UNIT NAME:

February 11, 1980 DATE:

L. K. Miller COMPLETED BY:

609-365-7000 X507 TELEPHONE:

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-001	1/1/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-002	1/3/80	F	0	. D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-003	1/5/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-004	1/7/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-005	1/8/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-009	1/11/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Control Strainers
80-015	1/14/80	F	37.1	A	3		RB	ZZZZZZ	Nuclear Instrumentation (Spurious noise spide on Power Range Channel N-43 while N-44 was in test).
80-017	1/17/80	F	0	A	5		НН	НТЕХСН	Clean Condensate Pumps Suction Strainers
80-019	1/20/80	F	0	A ,	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers

1

F: Forced S: Scheduled

2

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Continuation of Previous Outage

5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER) File (NUREG-0161)

5 Exhibit 1-Same Source

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# UNIT SHUIDOWNS AND POWER REDUCTIONS REPORT MONTH January 1980

**DOCKET NO.:** 50-272

UNIT NAME: Salem #1

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X50

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUITING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-022	1/23/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers
80-024	1/23/80	F	25.2	A	3		нн	TRANSF	Loss of Auxiliary Transformer
80-026	1/27/80	F	0	A	5		нн	нтехсн	Clean Condensate Pumps Suction Strainers
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			i						
							}		
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Page 5 of 18 8-1-7.R1 MAJOR PLANT MODIFICATION REPORT MGNTH January 1989

**DOCKET NO.:** 50-272

UNIT NAM

Salem #1

DATE:

February 11, 1980

COMPLETED BY:

L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ED-0191	Sampling	Install Feedwater O <sub>2</sub> Analyzer
1ED-0343	Cathodic Protection	Install Ground Beds at S.W. Structure
1EC-0438	Circulating Water	Modify Controls for Manual Operation
1EC-0465	Control Air	Remove and Replace Piping for Condenser Retubing
1EC-0561	Main Turbine Lube Oil	Install Lube Oil Filter
1EC-0568	Fire Protection	Install Additional Aux. Building Hose Reel
1EC-0623	MSIV	Remove Overload Jumpers on Hydraulic Pump
1EC-0678	Containment Ventilation	Modify 1VC5 & 6 to Improve Operation
1EC-0683	4KV Vital Busses	Change Relay Setting
1EC-0699	Containment Sump	Install Vortex Baffles and Reset Level Switches
1MD-0051	Reactor Head	Modify Themocouple Trays
1PD-0037	Aux. Temperature Indication	Replace Bristol Recorders with the Doric
1SC-0018	Communications	Install S.P. Phone at Pressurizer
1SC-0025	Fuel Handling	Modify Fuel Transfer System in Accordance With Westinghouse WRAPS Package
1SC-0046	Fire Protection	Replace HP Turbine Fire Detector
1SC-0064	Overhead Annunciator	Modify Alarm Windows D-37 and D-45
1SC-0112	Control Room	Install Improved Clock

<sup>\*</sup> DESIGN CHANGE REQUEST 8-1-7.R1

MAJOR PLANT MODIFICATION January 1980 REPORT MONTH

DOCKET NO .: 50-272

UNIT NE:

Unit #1

DATE:

February 11, 1980

COMPLETED BY:

L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1ED-0191	This change is not safety related and does not affect safety related equipment.
1ED-0343	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR 50.59.
1EC-0438	This system is not safety related. The change increases the flexibility of operation without impairing reliability. The type of materials employed in this change are identical to those used in the current design.
1EC-0465	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Technical Specifications are not affected.
1EC-0561	This design change will not require a change to the Tech. Specs. or FSAR.  The additional filter equipment nor the surrounding equipment is safety related. This modification will improve the main turbine lube oil cleanliness and increase reliability.
1EC-0568	The installation of the hose station required by this DCR has been in the previously reviewed and it has been found that it does not present an unresolved safety question. No additional fire hazard is introduced by the addition of this hose station. It improves the plant safety features against fire.
1EC-0623	The modification does not change the functional operation of the system nor degrade its performance.
1EC-0678	The reworked valves will not adversely affect existing safety equipment or invalidate existing safety analysis. This design change will assure that the valve closes when required to.
1EC-0683	This change does not alter the functional design of the system. It reflects preferred method of applying protective relays. The existing design was necessary only because seismic data on preferred design was not previously available. No modifications affecting fire systems are made.
1EC-0699	Implementation of this DCR will insure a safer shutdown of the NSSS equipment following an accident involving a breach in the primary coolant system by providing anti-vortex baffles in the reactor building sump pit on the suction side of the residual heat removal pumps.
1MD-0051	Modifying the two cable trays does not involve any safety questions.

MAJOR PLANT MODIFICATIO REPORT MONTH January 1980

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DOCKET NO.: 50-272 UNIT NAE: Salem #1

DATE:

February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE:

609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1PD-0037	The system and instruments involved in this change are not safety related but the instruments are mounted in a Class I cabinet (1RP1). Class I cabinet requirements have been considered for 1RP1 and have not been altered by this change.
1SC-0018	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR 50.59.
1SC-0025	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Tech. Specs. are not affected.
1SC-0046	This is not a safety related system and has no effect upon the safe shutdown capability of the plant.
1SC-0064	This change is not functionally safety related and supports a design analysis revising the alarms so that the alarm windows remain "black" unless both the reactor power is above 50% and the quadrant power tilt ration exceeds 1.02. It is classified as safety related because the work performed will be in a safety related cabinet.
1SC-0112	This design change is not safety related and does not affect any safety related systems or the safe shutdown of the unit.
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SALEM GENERATING STATION 0002 MONTHLY REPORT OF SAFETY RELATED REPAIRS WORK DATE PROBLEM DESCRIPTION DATE DESCRIPTION OF REPAIR ORDER WRITTEN COMPLETED REPLACED BONNET SEAL 903061 04 21 79 VALUE 14AF23 HAS PACKING LEAK 12 08 79 REPACKED 903829 02 26 79 16 SERVICE WATER PUMP 03 05 79 PUMP DISMANTLED AND INSPECT & REPAIR SENT TO OUTSIDE VENDOR FOR WORK 905924 07 03 79 VALVE ISW223 BINDING 08 21 79 REPOSITIONED BELL CRANK REPACKED 913739 10 27 79 1A DIESEL GENERATOR EXHAUST INSTALLED NEW GASKETS MANIFOLD LEAKS 914344 11 15 79 VALUE 11MS45 HAS PACKING LEAK REPACKED 914345 11 15 79 VALUE 12AF23 HAS PACKING LEAK 12 07 79 REPACKED REPLACED BONNET SEAL 914346 11 15 79 VALVE 12BE22 HAS PACKING LEAK 12 07 79 REPACKED 914349 11 15 79 VALUE 14MS130 HAS PACKING LEAK REPACKED 914391 12 31 79 REPLACED BEARINGS . 11 CHILLED WATER PUMP RUNNING HOT 01 UL 80 11 CHILLED WATER PUMP DILER LEAKING 01 04 80 914417 01 03 80 REPLACED MECHANICAL SEAL REPLACED DIL SEALS 914456 01 04 80 11 CHILLED WATER PUMP TRIPPING \_01\_05\_80\_ REPLACED OVERLOADS INSTALLED NEW VALVE BODY 914574 01 14 80 VALVE 115W24 FLANGE LEAK 01 15 80 08 17 79 REPACKED 915871 08 15 79 VALVE IRH26 HAS PACKING LEAK 915878 08 15 79 VALUE LIRHAO HAS PACKING LEAK Ŭ8 17 79 REPACKED 915880 08 15 79 **08 17 79** REPACKED VALVE 11RH18 HAS PACKING LEAK SHIPPED FOR REPAIR 916090 12 13 79 VALUE 14AF21 LEAKS THRU 12 19 79 RETURNED TO SERVICE 12 19 79 REPLACED INTERNALS 916092 12 14 79 VALVE 11AF21 LEAKS THRU REPACKED REPLACED DIAPHRAGM 11 22 79 917436 11 06 79 VALUE 135W264 LEARS THRU REPLACED FILTERS 917502 11 11 79 15 CONTAINMENT EAN COIL

14 CONTAINMENT FAN SOIL 11 26 79 REPLACED FILTERS

REPAIRED\_LIMIT\_SWITCHES \_\_\_\_

ON VALVES 1MS132 AND 1MS52

13 AUXILIARY FEEDWATER 01 17 80 ....

ROUGHING FILTERS DIRTY

ROUGHING FILTERS DIRTY

PUMP START LIGHT DOES

NOT LIGHT

Page 10 of 18

917563 11 15 79

918851 01 16 80

#### SALEM GENERATING STATION MONTHLY REPORT OF SAFETY RELATED REPAIRS

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PAIR
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Manager (1) (1) (1) (1) (1) (1) (1) (1) (1) (1)

		MONTHLY REPORT OF SAFETY		0004
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
919701	11 18 79	VALVE 125J154 HAS EACKING LEAK	11 21 79	REPACKED
919702	11 18 79	VALUE ISJ149 HAS PACKING LEAK	11 21 79	REPACKED
919703	11 18 79	VALVE 13SJ138 HAS PACKING LEAK	11 21 79	REPACKED
919708	11 20 79	AIRLOCK_ELEVATION_100 INSPECT INNER DOOR GASKET		REVERSED_INNER_AND_OUTER_DOOR
919725_	11 24 79	VALVE 14RC26 HOS PACKING LEAK	11_24_79	REPACKED
919753	11 30 79	VALVE 13RC24 HAS PACKING LEAK	12 05 79	REPACKED
919754	11 30 79	VALVE 13RC28 HAS PACKING LEAK	12 05 79	REPACKED . CLEANED BORON OFF VALVE
919769	12 05 79	VALVE 13RC25 HAS PACKING LEAK	12 05 79	REPACKED
919770	12 05 79	VALVE 12AF21 LEAKS THRU	12_12_ <i>79</i>	REPLACED STEM CAGE AND SEAT OF VALVE INSTALLED NEW GASKETS AND REPACKED
919772	12 05 79	VALVE 1CV79 HAS PACKING LEAK	12 05 79	REPACKED
919774	12 05 79	16 SERVICE WATER PUMP INVESTIGATE HIGH VIBRATION	12_21_ <i>79</i>	REBUILT PUMP. LOWER ASSEMBLY REBUILT BY A&A COMPANY
919783	12 06 79	VALVE 11MS9 HAS PACKING LEAK	11 08 79	REPACKED
. 919784	12 06 79	VALVE 11MS168 HAS PACKING LEAK	12 07 79	REPACKED
919785	12 06 79	VALVE 13M5168 HAS PACKING LEAK	12 07 79	REPACKED
919786	12 06 79	VALVE 13HS45 HAS PACKING LEAK	12 08 79	REPACKED
919788	12 06 79	VALVE 14MS199 HAS PACKING LEAK	12 07 79	REPACKED
929769	12 17 79	VALVE 1CV238 HAS BONNET LEAK	01 02 80	INSTALLED NEW DIAPHRAGM FIXED BUSHING
929862	12 22 79	VALVE 115W23 NO OPEN INDICA- TION IN CONTROL ROOM	12 04 79	REFLACED RELAY 33X-3
929877	12 23 79	16 SERVICE WATER STRAINER BROKEN SHEAR PIN	12 24 79	REPLACED SHEAR KEY AND BARREL SEAL PLATE
929922	12 27 79	VALVE 11MS167 WON'T OPEN	12 27 79	ADJUSTED PACKING
929933	12 27 79	CONTAINMENT SPRAY ADDITIVE TANK MANWAY LEAKING	12_31_79.	INSTALLED_GASKET_ON_ADDITIVE FILLPIPE
		<del></del>	. W.	

		MONTHLY REPORT OF SAFETY RI	ELATED REPAIRS	0005
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
				,
929935	12 29 79	12 FUEL HANDLING BUILDING	12 29 79	REPLACED BROKEN BELTS
		EXHAUST FAN HAS BROKEN BELT		
929941	12 28 79	11 RESIDUAL HEAT REMOVAL	01_02_80	REPAIRED MICRO SWITCH ON ACTUATING DEVICE
		SUMP ALARMS ON OVERFLOW		
<u>931957</u>	12 30 79	13 STEAM FLOW VENT VALUE NEAR TRANSMITTER FT532	12_30_79	REPLACED BONNET OF SAMPLE VALVE
		HAS BONNET LEAK		
931971	01 03 80	1A SAFEGUARD EMERGENCY CABINET INSPECTION BY VENDOR	01 03 80	REPLACED BAD RELAY
931997	01 08 80	PENETRATION I-25 & I-26 RECHARGE AND CHECK FOR LEAKS	01 09 80	RECHARGED FOUND NO LEAKS
·		********		
		*** PERFORMANCE DEPARTMENT WORK ***		
	·.	*********		·
905424	08 16 79	N-31 SOURCE RANGE SENSITIVE TO ELECTRICAL NOISE	12 13 79	REBUILT CABLE PLUGS REPLACED JACK ON PHD CIRCUIT
911838	10 11 79	N31 DETECTOR EXCESSIVE NOISE	10 19 79	REPLACED N31 AND N35 DETECTORS REPUILT TRIAXIAL PLUGS AT DRAWE
914397	01 01 80	SIGNAL ISOLATOR IN N-43 OUT OF CALIBRATION	01 01 80	REPLACED SIGNAL ISOLATOR RM203A SERIAL R0455 WITH R0453
914476	01 06 80	R31B FAILED FUEL PROTECTION READS LOW	01 06 80	REPLACED BOARD 1PB1 CALIBRATED DRAWER
914491	01 07 80	13 STEAM GENERATOR FEEWWATER FLOW CHANNEL II MIDRANGE FAILED	01 07 80	REPLACED AMPLIFIER BOARD RECALIBRATED
914521	01 09 80	VALVE 145W232 LIMIT SWITCH ARM BENT		STRAIGHTENED LIMIT SWITCH VERIFIED OPERATION
•				
915296	11 12 79	N41 CABLES SHORTED	11 14 79	LOCATED SHORT AND CLEARED IT
915322	12 05 79	N31 DETECTOR VERIFY PROPER		REPLACED CABLE PLUG ON
710022	12 VJ /7	DETECTOR VOLTAGE AND METER ACCURACY		HIGH VOLTAGE POWER SUPPLY

WORK ORDER	DATE WRITTEN	PROBLÉM DESCRIPTION	*** *** * *** * **** ***	DESCRIPTION OF REPAIR
915615	. 11 15 79	N-42 SOURCE RANGE OUT OF SPEC	11 30 79	REPLACED MODULE XA5904RB WITH
				HODULE FROM UNIT 2
919394	11 24 79	VALVE 125W223 DOES NOT CONTROL FLOW TO 12 CONTAINMENT FAN COIL UNIT		BLEW DOWN TRANSMITTER FA31652-1 AND FA354021 REPLACED AMPLIEIER IN FA354021
919591	12 20 79		12 22 79	
929776	12 18 79	100 Mg	12 19 79	DISASSEMBLED AND CLEANED
929806	12 14 79	12 REACTOR COOLANT PUMP LOW_SEAL_LEAK_OFF_FLOW_ INDICATOR/TRANSMITTER STICKS	12 18 79	CLEANED AND FLUSHED FLOAT REINSTALLED FLANGE WITH NEW GASK
929820	12 07 79	INSTRUMENT 1R18 FAILED LOW.	12 07 79	SOLDERED RAD CONNECTIONS
929894	12 25 79	VALVE 14MS18 DOES NOT CLOSE	12 26 79	REPLACED SOLENOID VALVE
TOTAL L	INES = 000170	· · · · · · · · · · · · · · · · · · ·		

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# SALEM UNIT #1 OPERATING SUMMARY JANUARY 1980

1/1 At 40% power, steady state conditions for entire day. 1/2 At 0330, commenced inserting rods by dilution to 140 steps to swing  $\Delta I$  negative in preparation for a flux map. Power remained at 40% for the whole day. 1/3 At 0500 on 1/3/80, waiting for  $\Delta I$  to stabilize for power coefficient test. Between 0640 and 0740, turbine load being increased then decreased. At 0750, commenced increasing power to 47% at 3% per hour. The reactor remained at 47% for the remainder of the day. 1/4 Power remained at 47% for the entire day. 1/5 Power being increased from 48% to 60% on 1/5/80 in preparation for thru next physics test. Once physics test was complete, power was reduced 1/6 to 50% by 2400 on 1/6/80 to clean feed pump strainers and to evaluate quadrant power tilt. 1/7 Power remained at 50% until authorization was given to go to 80%. thru At 2200 on 1/7/80, power escalation began at approximately 3% per hour. 1/8 Power was held at 80% for physics testing. At 2115 on 1/8/80, reactor power reduction began from 80% to 65% power to clean condensate pump strainers. 1/9 At 1200 on 1/9/80, power escalation began at 1.5% per hour to 95% which thru is the next physics testing plateau. Power remained at 95% through 1/10 1/10 while physics testing was in progress. 1/11Still performing physics testing. At 0630 on 1/11/80, commenced power thru reduction at 3% per hour down to 70% to clean condensate pump strainers. 1/12 At 2100, with strainers cleaned, power escalation began at 3% per hour reactor thermal power. Power remained at 95% for the remainder of this period.

- 1/13 At 95% power until 1400 when load reduction in order to perform governor valve closure of #12 stop valve was commenced. Load reduction terminated at 93% power and remained there for the remainder of the day.
- 1/14 Power escalated from 93% to 95%. At 0956 on 1/14/80, reactor tripped from 95% on a false high neutron flux rate trip signal. Reactor remained subcritical until 1013 on 1/15/80 when reactor was taken critical. Remained at zero power until reactor tripped at 1630 on steam generator low low level. Reactor brought critical until it tripped again at 1900 on steam generator low low level. Reactor brought critical and power escalation began. At 2400 on 1/15/80, power was 18.2%.
- 1/16 Power escalation continued. At 1234, experienced turbine load swing due to E.H. control system malfunction. Turbine placed in manual.

  At 2400, reactor power was at 89.5%.
- 1/17 Power reached 95% at 0400. At 1620, commenced dropping load to 70% to allow cleaning of main feedwater pump strainers. Power reduced to 55% to keep plant stabilized while cleaning strainers. At 2200, commenced increasing load to 95% at 5% per hour.
- 1/18 Load increase in progress. Load stabilized at 95% by 1600 and stayed there for the remainder of the day.
- 1/19 Load was stabilized at 95% power until 2010 when load reduction was commenced at 5% per hour.
- 1/20 Load reduction in progress at 5% per hour. Load stabilized at 70% by 0200 to allow cleaning of condensate pump strainers. Load increase commenced at 1245 at 3% per hour. Load at 100% by 2330.
- 1/21 Load stabilized at 100%.
- 1/22 Load stabilized at 100%. Load reduction commenced at 2200 to allow cleaning of condensate pump strainers.

- 1/23 Load decrease in progress. Stabilized load at 70% at 0100. Condensate strainers cleaned and increasing load by 0345 at 3% per hour. Unit trip from 96% at 1029. 1H 460 volt group transformer dropped out of service which resulted in the tripping of a steam generator feedwater pump and a reactor trip on low steam generator water level. Unit down for the remainder of the day.
- 1/24 Reactor critical and load increase commenced at 1130. Load being increased over the remainder of the day.
- 1/25 Load increase in progress to 100%. Load stabilized at 100% by 1900.
- 1/26 Load stabilized at 100% power for the day.
- 1/27 Load stabilized at 100% until 2030 when a load decrease was commenced to 70% by 2300 to allow cleaning of condensate pump strainers.
- 1/28 Load increase commenced at 0115 from 70%. Load stabilized at 100% by 1030 for the remainder of the day.
- Load stabilized at 100% until 2100 when load was decreased to 95%.

  Load reduced to maintain adequate NPSH to steam generator feedwater pumps which was dropping because of increasing differential pressure across the condensate polishing system. Load stabilized at 95% while backflushing operations continue with the condensate polishing system to reduce the differential pressure across the polisher vessels. Load increased and stabilized at 100% by 1020 for the remainder of the day.
- 1/30 Load stabilized at 100% for the day.
- 1/31 Load stabilized at 100% for the day.

# REFUELING INFORMATION

DOCKET NO.: 50-272

UNIT: Salem #1

COMI	DATE: February 11, 198
	relephone: 609-365-7000
	x507
ONTH:	
. Refueling information has changed from last mo	onth:
YES_X_NO	D
Scheduled date of next refueling: September	er 20, 1980
Scheduled date for restart following refueling	: November 16, 1980
. A. Will Technical Specification changes or o	ther license
amendments be required? YESNO	o
NOT DETERMINED	TO-DATE January 1980
B. Has the reload fuel design been reviewed	by the Station Operating
Review Committee? YESNO	ox
If no, when is it scheduled? At	igust 1980
. Scheduled date(s) for submitting proposed lice	ensing action:
August 198	0 (If required)
. Important licensing considerations associated NONE	with refueling:
. Number of Fuel Assemblies:	
A. In-Core	193
B. In Spent Fuel Storage	40
Present licensed spent fuel storage capacity:	264
Present licensed spent fuel storage capacity: Future spent fuel storage capacity:	



Public Service Electric and Gas Company P.O. Box 168 Hancocks Bridge, New Jersey 08038

Salem Nuclear Generating Station

February 11, 1980

Director, Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT SALEM NO. 1
DOCKET NO. 50-272

In compliance with section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of January 1980 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Summary of Safety Related Maintenance
Operating Summary
Refueling Information

Sincerely yours,

N. J. While

H. J. Midura

Manager - Salem Generating Station

LKM:vd

cc: Mr. Boyce H. Grier
Director of U.S. NRC

Office of Inspection and Enforcement

Region I

631 Park Avenue

King of Prussia, PA 19406

Director, Office of Management Information and Program Control U.S. Nuclear Regulatory Commission Washington, DC 20555

Enclosures Page 1 of 18 8-1-071 R4

#### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272

UNIT Salem #1

DATE February 11, 1980

COMPLETED BY L. K. Miller

TELEPHONE 609-365-7000

X 507

			A 307	
MON	January 1980 TH			
DAY	AVERAGE DAILY POWER LEVEL (MWe-NET) -	DAY AVERA	GE DAILY POWER LEVEL (MWE-NET)	
1	342	17	924	
2	319	18	943	
3	369	19	1,047	
4	370	20	833	
5 .	515	21	1,069	
6	598	22	1,092	
7 _	507	23	379	
8_	763	24	96	
9	788	25	856	
10	980	26	1,069	
11	1,100	27	1,084	
12	908	28	1,021	
13	1,023	29	1,048	
14	299	30	1,114	
15	0 .	31	1,114	
16	630			

#### OPERATING DATA REPORT

DOCKET NO.:\_\_\_\_

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

тетерноме: 365-7000 Х507

#### OPERATING STATUS

	Onit Name: Salem #1	Notes:	Notes:				
	Reporting Period: January 1980						
3.	Licensed Thermal Power (MWt): 3338						
4. 1	Nameplate Rating (Gross MWe): 1135						
5. 1	Design Electrical Rating (Net MWe): 1090						
6. 1	Maximum Dependable Capacity (Gross MWe): $\_$	.124					
7. !	Maximum Dependable Capacity (Net MWe): $\_$	.079					
8.	If Changes Occur in Capacity Ratings (Items No NONE	umber 3 Through 7) Since	Last Report, Give R	eason:			
	Power Level To Which Restricted, If Any (Net Non-	are) .	ONE				
		This Month	Year to Date	Cumulative			
		744	744	22,705			
	Hours In Reporting Period	696.4	696.4	10,784.4			
	Number Of Hours Reactor Was Critical			22.7			
	Reactor Reserve Shutdown Hours	0	0				
	Hours Generator On-Line	681.7	681.7	10,212.8			
	Unit Reserve Shutdown Hours	0		0			
	Gross Thermal Energy Generated (MWH)	1,789,716	<del></del>	29,326,013			
	Gross Electrical Energy Generated (MWH)	586,080	586,080	9,753,580			
	Net Electrical Energy Generated (MWH)	556,282	556,282	9,185,872			
19.	Unit Service Factor	91.6	91.6	45.0			
20.	Unit Availability Factor	91.6	91.6	45.0			
21. 1	Unit Capacity Factor (Using MDC Net)	69.3	69.3	37.5			
22. 1	Unit Capacity Factor (Using DER Net)	68.6	68.6	37.1			
	Unit Forced Outage Rate	8.4	8.4	43.4			
24. :	Shutdowns Scheduled Over Next 6 Months (Type, NONE	Date, and Duration of E	ach):				
25.	If Shut Down At End of Report Period, Estimate	ed Date of Startup:	N/A				
26. 1	Units In Test Status (Prior to Commercial Opera	ation):	Forecast	Achieved			
			9/30/76	12/11/76			
	INITIAL CRITICALITY			12./ 11./ / 11.			
	INITIAL CRITICALITY INITIAL ELECTRICITY		11/1/76	12/25/76			

#### UNIT SHUIDOWNS AND POWER REDUCTIONS

REPORT MONTH January 1980

50-272 DOCKET NO .:

Salem #1 UNIT NAME:

February 11, 1980 DATE:

COMPLETED BY: L. K. Miller

609-365-7000 X507 TELEPHONE:

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	reason <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE .
80-001	1/1/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-002	1/3/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-003	1/5/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-004	1/7/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-005	1/8/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-009	1/11/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers
80-015	1/14/80	F	37.1	A	3		RB	ZZZZZZ	Nuclear Instrumentation (Spurious noise spide on Power Range Channel N-43 while N-44 was in test).
80-017	1/17/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers
80-019	1/20/80	F	0	A	5		нн	нтехсн	Clean Condensate Pumps Suction Strainers

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F: Forced S: Scheduled Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Continuation of Previous Outage

5-Load Reduction

9-Other

Exhibit G - Instructions

for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1-Same Source

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH\_ January 1980

50-272 DOCKET NO.:

Salem #1 UNIT NAME:

February 11, 1980 DATE:

COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000

NO.	DATE	туре1	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-022	1/23/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers
80-024	1/23/80	F	25.2	A	3		НН	TRANSF	Loss of Auxiliary Transformer
80-026	1/27/80	F	0	A	5		НН	нтехсн	Clean Condensate Pumps Suction Strainers

Page 5 of 18 8-1-7.R1

MAJOR PLANT MODIFICATION January 1980 REPORT MONTH

DOCKET NO.: 50-272

UNIT NAM

Salem #1

DATE:

February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE:

609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ED-0191	Sampling	Install Feedwater O <sub>2</sub> Analyzer
1ED-0343	Cathodic Protection	Install Ground Beds at S.W. Structur
1EC-0438	Circulating Water	Modify Controls for Manual Operation
1EC-0465	Control Air	Remove and Replace Piping for Condenser Retubing
1EC-0561	Main Turbine Lube Oil	Install Lube Oil Filter
1EC-0568	Fire Protection	Install Additional Aux. Building Hose Reel
1EC-0623	MSIV	Remove Overload Jumpers on Hydraulic Pump
1EC-0678	Containment Ventilation	Modify 1VC5 & 6 to Improve Operation
1EC-0683	4KV Vital Busses	Change Relay Setting
1EC-0699	Containment Sump	Install Vortex Baffles and Reset Level Switches
1MD-0051	Reactor Head	Modify Themocouple Trays
1PD-0037	Aux. Temperature Indication	Replace Bristol Recorders with the Doric
1SC-0018	Communications	Install S.P. Phone at Pressurizer
1SC-0025	Fuel Handling	Modify Fuel Transfer System in Accordance With Westinghouse WRAPS Package
1SC-0046	Fire Protection	Replace HP Turbine Fire Detector
1SC-0064	Overhead Annunciator	Modify Alarm Windows D-37 and D-45
1SC-0112	Control Room	Install Improved Clock
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<sup>\*</sup> DESIGN CHANGE REQUEST 8-1-7.R1

MAJOR PLANT MODIFICATIO REPORT MONTH January 1980 DOCKET NO .: 50-272

UNIT N. Unit #1

DATE:

February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1ED-0191	This change is not safety related and does not affect safety related equipment.
1ED-0343	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR 50.59.
1EC-0438	This system is not safety related. The change increases the flexibility of operation without impairing reliability. The type of materials employed in this change are identical to those used in the current design.
1EC-0465	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Technical Specifications are not affected.
1EC-0561	This design change will not require a change to the Tech. Specs. or FSAR.  The additional filter equipment nor the surrounding equipment is safety related. This modification will improve the main turbine lube oil cleanliness and increase reliability.
1EC-0568	The installation of the hose station required by this DCR has been previously reviewed and it has been found that it does not present an unresolved safety question. No additional fire hazard is introduced by the addition of this hose station. It improves the plant safety features against fire.
1EC-0623	The modification does not change the functional operation of the system nor degrade its performance.
1EC-0678	The reworked valves will not adversely affect existing safety equipment or invalidate existing safety analysis. This design change will assure that the valve closes when required to.
1EC-0683	This change does not alter the functional design of the system. It reflects preferred method of applying protective relays. The existing design was necessary only because seismic data on preferred design was not previously available. No modifications affecting fire systems are made.
1EC-0699	Implementation of this DCR will insure a safer shutdown of the NSSS equipment following an accident involving a breach in the primary coolant system by providing anti-vortex baffles in the reactor building sump pit on the suction side of the residual heat removal pumps.
1MD-0051	Modifying the two cable trays does not involve any safety questions.

MAJOR PLANT MODIFICATIO REPORT MONTH January 1980

DOCKET\_NO.: \_\_\_\_ 50-272

UNIT NAME: Salem #1

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1PD-0037	The system and instruments involved in this change are not safety related but the instruments are mounted in a Class I cabinet (1RP1). Class I cabinet requirements have been considered for 1RP1 and have not been altered by this change.
1SC-0018	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR 50.59.
1SC-0025	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Tech. Specs. are not affected.
1SC-0046	This is not a safety related system and has no effect upon the safe shut-down capability of the plant.
1SC-0064	This change is not functionally safety related and supports a design analysis revising the alarms so that the alarm windows remain "black" unless both the reactor power is above 50% and the quadrant power tilt ration exceeds 1.02. It is classified as safety related because the work performed will be in a safety related cabinet.
1SC-0112	This design change is not safety related and does not affect any safety related systems or the safe shutdown of the unit.

#### SALEM GENERATING STATION MONTHLY REPORT OF SAFETY RELATED REPAIRS

•		MONTHLY REPORT OF SAFETY R	ETHIEN KELHIKO	0001
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
· · · · · · · · · · · · · · · · · · ·		*********		
	· · · · · · · · · · · · · · · · · · ·	*** HAINTENANCE DEPARTMENT WORK ***		
		********		
900236	09 14 79	FLANGE LEAK NEAR VALVE 115W223	10 02 79	WELD REPAIRED PIPE AND APPLIED BELZONA COATING
900632	12 16 78	VALVE 115J143 HAS PACKING LEAK	ŭ4 11 79	REPACKED
901258	03 21 79 .	VALVE 11H5167 REPACK	05_10_79	REPACKED
901366	04 14 79	VALVE 12GB4 REWORK	04 15 79	MACHINED DISC AND SEATINSTALLED_NEW GASKETS REPACKED
<u> 401914 </u>	04 30 79	VALUE 13MSII REWORK	05_24_ <u>79</u>	DISASSEMBLED LAPPED ASSEMBLED
901620	04 30 79	VALVE 13M512 REWORK	05 23 79	DISASSEMBLED LAPPED ASSEMBLED
901621	04 30 79	VALVE 13MS13 REWORK	05 29 79	DISASSEMBLED ASSEMBLED ASSEMBLED
901622	04 30 79	VALVE 13MS14 REWORK	<u>05 29 79</u>	DISASSE <u>MBLED</u> LAPPED ASSEMBLED
901623	04 30 79	VALVE 13MS15 REWORK	05 30 79	DISASSEMBLED LAPPED ASSEMBLED
901629	03 21 79	VALVE 12HS167 REPACK	05 10 79	REPACKED
901630	03 21 79	VALVE 13H5167 REPACK	05 10 79	REPACKED
<u>901631</u>	03 21 79	VALVE 14H5167 REPACK	05_10_79	REPACKED
901725	04 27 79	VALVE 115J34 LEAKS THRU	ō5 14 79	LOOSTENED HANGER BOLT
902801	04 11 79	VALVE 14GB4 FAILED LEAK RATE TEST	04 16 79	MACHINED DISC & SEAT INSTALLED NEW GASKETS REPACKED

		SALEM GENERATING STA MONTHLY REPORT OF SAFETY RE	0002	
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
<u> </u>				
903061	04 21 79	VALVE 14AF23 HAS PACKING LEAK	12 08 79	REPLACED BONNET SEAL REPACKED
903829	02 26 79	16 SERVICE WATER PUMP INSPECT & REPAIR	03 05 79	PUMP DISMANTLED AND SENT TO OUTSIDE VENDOR FOR WORK
905924	07 03 79	VALVE 15W223 BINDING	Ŭ8 21 79	REPOSITIONED BELL CRANK REPACKED
913739	10 27 79	LA DIESEL GENERATOR EXHAUST MANIFOLD LEAKS	•	INSTALLED NEW GASKETS
914344	11 15 79	VALVE 11M545 HAS PACKING LEAK		REPACKED
914345	11 15 79	VALVE 12AE23 HAS FACKING LEAK	12_07_79	REPACKED REPLACED BONNET SEAL
914346_	11 15 79	VALVE_128F22_HAS_PACKING_LEAK	12 07 79	REPACKED
914349	11 15.79	VALVE 14MS130 HAS PACKING LEAK		REPACKED
914391	12 31 79	. 11 CHILLED WATER PUHP RUNNING HOT	o1 o1 80	REPLACED BEARINGS
914417	01 03 80	11 CHILLED WATER PUMP DILER LEAKING	01_04_80	REPLACED <u>HECHANICAL</u> SEAL REPLACED OIL SEALS
914456_	01 04 80	11_CHILLED_WATER_PUMP_IRIPPING	01 05_80	REPLACED OVERLOADS
914574	01 14 80	VALVE 115W24 FLANGE LEAK	ŭ1 15 80	INSTALLED NEW VALVE BODY
915871	08 15 79	VALVE IRH26 HAS PACKING LEAK	ŬB 17 79	REPACKED
915878	08 15 79	VALVE 11RH40 HAS PACKING LEAK	ŬB 17 79	REPACKED
915880	08 15 79	VALVE 11RH18 HAS PACKING LEAK	08 17 79	REPACKED
916090	12 13 79	VALVE 14AF21 LEAKS THRU	12 19 79	SHIPPED FOR REPAIR RETURNED TO SERVICE
916092	12 14 79	VALVE 11AF21 LEAKS THRU	12 19 79	REPLACED INTERNALS REPACKED
917436	11 06 79	VALVE 135W264 LEAKS THRU	11 22 79	REPLACED DIAPHRAGH
917502	11 11 79	15 CONTAINMENT FAN COIL ROUGHING FILTERS DIRTY		REPLACED_FILTERS
917563	11 15 79	14 CONTAINMENT_FAN SOIL ROUGHING FILTERS DIRTY	11 26 79	REPLACED FILTERS
918851	01 16 80	13 AUXILIARY FEEDWATER PUMP START LIGHT DOES NOT LIGHT	01,17,80	REPAIRED_LIHIT SWITCHES ON VALVES IMS132 AND IMS52

#### SALEM GENERATING STATION MONTHLY REPORT OF SAFETY RELATED REPAIRS

•		MUNITURE NEPONT OF SHEET	RECHIED REPHINS	6003
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
919398	11 25 79	VALVE 12MS168 HAS PACKING LEAK	12 05 79	REPACKED
919399	11_25_79	VALVE 14MS168 HAS PACKING LEAK	12_06_79	PEPACNED
919450	11 26 79	VALVE 1P52 HAS BROKEN REACH	11 28 79	REPAIRED REACH ROD
919464	11 27 79	VALVE 125J40 HAS PACKING LEAK	11 28 79	REPACKED
919539	12 01 79	11 BORIC ACID TRANSFER FUMP SEAL LEAKING	12 04 79	REPLACED MECHANICAL SEAL
919590	12 21 79	11 STEAM GENERATOR SNUBBERS SIGHTGLASS LEAKING	12 21 79	DRAINED OIL TIGHTENED SIGHT GLASS FILLED OIL AND TIGHTENED PACKIN
919592	12 21 79	VITAL HEAT TRACING HEAT TAPE 06028 NOT WORKING		REPAIRED
919593	12 21 79	VITAL HEAT TRACINGHEAT TAPE 0622A DOES NOT WORK	12 27 79	REPAIRED HEAT TAPE
919611	11 15 79	REACTOR HEAD INSTRUMENT	11 17 79	·
	•	REPLACE GASNETS		•
919615	11_16_79	VALVE 11RC21 HAS PACKING LEAK	11_21_79	REPACKED
919616	11 16 79	VALVE 11RC23 HAS PACKING LEAK	11 20 79	REPACKED
919620	11 16 79	VALVE 11RC17 HAS PACKING LEAK	11 21 79	REPACKED
919621	11_16_79	VALUE 11RC20 HAS PACKING LEAK	11_21_79	REPACKED
919622	11 16 79	VALVE 11RG25 HAS PACKING LEAK	11 21 79	REPACKED
919627	11 16 79	VALVE 13RC17 HAS PACKING LEAK	11 22 79	REPACKED
919632_	11 16 79	VALVE IPSI3 HAS PACKING LEAK		REPACKED
919671	11 16 79	VALUE 14MS27 HAS PACKING LEAK	11 18 79	REPACKED
919673	11 16 79	VALVE 12M510 HAS PACKING LEAK	11 18 79	REPACNED
919675	11 16 79	VALVE 12MS18 HAS PACKING LEAK	11.1779	REPACKED
919676	11 16 79	VALUE 14M518 HAS PACKING LEAK	11 17 79	REPACKED
919681	11 16 79	VALUE 14AF23 HAS PACKING LEAK	11 21 79	REPACKED
919700	11 18 79	VALVE_1CV2_HAS_PACKING_LEAK		REPACKED

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WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
919701	11 18 79		11 21 79	REPACKED
919702	11 18 79	VALVE 15J149 HAS PACKING LEAK	11 21 79	REPACKED
919703	11 18 79	VALVE 13SJ138 HAS PACKING LEAK	11 21 79	REPACKED
919708	11 20 79	AIRLOCK_ELEVATION_100 . INSPECT INNER DOOR GASKET	11 20.79	REVERSED INNER AND OUTER DOOR
919725	11 24 79	VALVE 14RC26 HAS PACKING LEAK	11_24_79	REPACKED
919753	11 30 79	VALUE 13RC24 HAS PACKING LEAK	12 05 7 <b>9</b> .	REPACKED
919754	11 30 79	VALVE 13RC28 HAS PACKING LEAK	12 05 79	REPACKED CLEANED BORON OFF VALVE
919769	12 05 79	VALVE 13RC25 HAS PACKING LEAK	12 05 79	REPACKED
919770	12 05 79	VALVE_12AF21_LEAKS_THRU	12_12_79	REPLACED STEM CAGE AND SEAT OF VALVE INSTALLED NEW GASKETS AND REPACKE
919772	12 05 79	VALUE 1CU79 HAS PACKING LEAK	12 05 79	REPACKED
919774	12 05 79	16 SERVICE WATER PUMP INVESTIGATE HIGH VIBRATION	12_21_79	REBUILT FUMP. LOWER ASSEMBLY REBUILT BY A&A COMPANY
919783	12 06 79	VALUE 11MS9 HAS PACKING LEAK	11 08 79	REPACKED
919784	12 06 79	VALVE 11MS168 HAS PACKING LEAK	12 07 79	REPACKED
919785	12 06 79	VALVE 13M5168 HAS PACKING LEAK	12 07 79	REPACKED
919786	12 06 79	VALUE 13HS45 HAS PACKING LEAK	12 08 79	REPACKED
919788	12 06 79	VALUE 14MS199 HAS PACKING LEAK	12 07 79	REPACKED
929769	12 17 79	VALVE ICU238 HAS BONNET LEAK	01 02 80	INSTALLED NEW DIAPHRAGM FIXED BUSHING
929862	12 22 79	VALVE 115W23 NO OPEN INDICA- TION IN CONTROL ROOM	12 04 79	REPLACED RELAY 33X-3
929877	12 23 79	16 SERVICE WATER STRAINER BROKEN SHEAR PIN	12 24 79	REPLACED SHEAR KEY AND BARREL SEAL PLATE
929922	12 27 79	VALVE 11M5167 WON'T OPEN	12 27 79	ADJUSTED PACKING
929933	12 27 79	CONTAINHENT SPRAY ADDITIVE TANK MANWAY LEAKING	12_31_79	INSTALLED_GASNET_ON_ADDITIVE FILLPIPE
····			Williams Committee Committ	

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		MONTHLY REPORT OF SAFETY RI	ELATED REPAIRS	0005
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
929935	12 29 79	12 FUEL HANDLING BUILDING	12 29 79	REPLACED BROKEN BELTS
		EXHAUST FAN HAS BROKEN BELT		
929941	12 28 79	11 RESIDUAL HEAT REHOVAL	010280	
		SUMP ALARMS ON OVERFLOW		ACTUATING DEVICE
931957	12 30 79	NEAR TRANSMITTER FT532 HAS BONNET LEAK	12_30_79	REPLACED BONNET OF SAMPLE VALVE
931971	01 03 80	1A SAFEGUARD EMERGENCY CABINET INSPECTION BY VENDOR	01 03 80	REPLACED BAD RELAY
931997	01 08 80	PENETRATION I-25 & I-26 RECHARGE AND CHECK FOR LEAKS	01 09 80	RECHARGED FOUND NO LEAKS
		*********		
		*** PERFORMANCE DEPARTMENT WORK ***		
		*************		
905424	08 16 79	N-31 SOURCE RANGE SENSITIVE TO	12 13 79	REBUILT CABLE PLUGS REPLACED JACK ON PHD CIRCUIT
911838	10 11 79	N31 DETECTOR EXCESSIVE NOISE	10 19 79	REPLACED N31 AND N35 DETECTORS REBUILT TRIAXIAL PLUGS AT DRAWERS
914397	01 01 80	SIGNAL ISOLATOR IN N-43 OUT OF CALIBRATION	01 01 80	REPLACED SIGNAL ISOLATOR QM203A SERIAL Q0455 WITH Q0453
914476	01 06 80	R31B FAILED FUEL PROTECTION READS LOW	01 06 80	REPLACED BOARD 1PB1 CALIBRATED DRAWER
914491	01 07 80	13 STEAM GENERATOR FEEWWATER FLOW CHANNEL II MIDRANGE FAILED	01 07 80	REPLACED AMPLIFIER BOARD RECALIBRATED
914521	01 09 80	VALUE 145W232 LIMIT SWITCH ARM BENT		STRAIGHTENED LIMIT SWITCH VERIFIED OPERATION
915296	11 12 79	N41 CABLES SHORTED	11 14 79	LOCATED SHORT AND CLEARED IT
915322	12 05 79	N31 DETECTOR VERIFY PROPER DETECTOR VOLTAGE AND METER ACCURACY	12 05 79	REPLACED_CABLE_PLUG_ON HIGH VOLTAGE POWER SUPPLY

		VVV6	
WORK DATE ORDER WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED .	DESCRIPTION OF REPAIR
			•
915615 11 15 79	N-42 SOURCE RANGE OUT OF SPEC		REPLACED HODULE XA5904RB WITH HODULE FROM UNIT 2
919394 11 24 79	VALVE 125W223 DOES NOT CONTROL FLOW TO 12 CONTAINMENT FAN COIL UNIT		BLEW DOWN TRANSHITTER FA31652-1 AND FA354021 REPLACED AMPLIEIER IN FA354021
919591 12 20 79	VALVE 13AF21DOES NOT REGULATE		REPLACED SQUARE ROOT EXTRACTOR FA3971 IAW DR# PD-0958
929776 12 18 79	AUDIO COUNT RATE SOUND DISTORIED		DISASSEMBLED AND CLEANED
929806 12 14 79	12 REACTOR COOLANT PUMP LOW_SEAL_LEAK_OEF_FLOW INDICATOR/TRANSMITTER STICKS	12 18 79	CLEANED AND FLUSHED FLOAT REINSTALLED FLANGE WITH NEW GASKE
929820 12 07 79	INSTRUMENT_IRLS FAILED_LOW.	12_07_79	SOLDERED BAD CONNECTIONS
929894 12 25 79	VALVE 14MS18 DOES NOT CLOSE	12 26 79	REPLACED SOLENOID VALVE

# SALEM UNIT #1 OPERATING SUMMARY JANUARY 1980

1/1 At 40% power, steady state conditions for entire day. 1/2 At 0330, commenced inserting rods by dilution to 140 steps to swing  $\Delta I$  negative in preparation for a flux map. Power remained at 40% for the whole day. 1/3 At 0500 on 1/3/80, waiting for  $\Delta I$  to stabilize for power coefficient test. Between 0640 and 0740, turbine load being increased then decreased. At 0750, commenced increasing power to 47% at 3% per hour. The reactor remained at 47% for the remainder of the day. 1/4 Power remained at 47% for the entire day. 1/5 Power being increased from 48% to 60% on 1/5/80 in preparation for thru next physics test. Once physics test was complete, power was reduced 1/6 to 50% by 2400 on 1/6/80 to clean feed pump strainers and to evaluate quadrant power tilt. 1/7 Power remained at 50% until authorization was given to go to 80%. At 2200 on 1/7/80, power escalation began at approximately 3% per hour. thru 1/8 Power was held at 80% for physics testing. At 2115 on 1/8/80, reactor power reduction began from 80% to 65% power to clean condensate pump strainers. 1/9 At 1200 on 1/9/80, power escalation began at 1.5% per hour to 95% which thru is the next physics testing plateau. Power remained at 95% through 1/10 1/10 while physics testing was in progress. 1/11 Still performing physics testing. At 0630 on 1/11/80, commenced power thru reduction at 3% per hour down to 70% to clean condensate pump strainers. 1/12 At 2100, with strainers cleaned, power escalation began at 3% per hour reactor thermal power. Power remained at 95% for the remainder of this

period.

- 1/13 At 95% power until 1400 when load reduction in order to perform governor valve closure of #12 stop valve was commenced. Load reduction terminated at 93% power and remained there for the remainder of the day.
- 1/14 Power escalated from 93% to 95%. At 0956 on 1/14/80, reactor tripped from 95% on a false high neutron flux rate trip signal. Reactor remained subcritical until 1013 on 1/15/80 when reactor was taken critical. Remained at zero power until reactor tripped at 1630 on steam generator low low level. Reactor brought critical until it tripped again at 1900 on steam generator low low level. Reactor brought critical and power escalation began. At 2400 on 1/15/80, power was 18.2%.
- 1/16 Power escalation continued. At 1234, experienced turbine load swing due to E.H. control system malfunction. Turbine placed in manual. At 2400, reactor power was at 89.5%.
- 1/17 Power reached 95% at 0400. At 1620, commenced dropping load to 70% to allow cleaning of main feedwater pump strainers. Power reduced to 55% to keep plant stabilized while cleaning strainers. At 2200, commenced increasing load to 95% at 5% per hour.
- 1/18 Load increase in progress. Load stabilized at 95% by 1600 and stayed there for the remainder of the day.
- 1/19 Load was stabilized at 95% power until 2010 when load reduction was commenced at 5% per hour.
- 1/20 Load reduction in progress at 5% per hour. Load stabilized at 70% by 0200 to allow cleaning of condensate pump strainers. Load increase commenced at 1245 at 3% per hour. Load at 100% by 2330.
- 1/21 Load stabilized at 100%.
- 1/22 Load stabilized at 100%. Load reduction commenced at 2200 to allow cleaning of condensate pump strainers.

- Load decrease in progress. Stabilized load at 70% at 0100. Condensate strainers cleaned and increasing load by 0345 at 3% per hour. Unit trip from 96% at 1029. 1H 460 volt group transformer dropped out of service which resulted in the tripping of a steam generator feedwater pump and a reactor trip on low steam generator water level. Unit down for the remainder of the day.
- 1/24 Reactor critical and load increase commenced at 1130. Load being increased over the remainder of the day.
- 1/25 Load increase in progress to 100%. Load stabilized at 100% by 1900.
- 1/26 Load stabilized at 100% power for the day.
- 1/27 Load stabilized at 100% until 2030 when a load decrease was commenced to 70% by 2300 to allow cleaning of condensate pump strainers.
- 1/28 Load increase commenced at 0115 from 70%. Load stabilized at 100% by 1030 for the remainder of the day.
- Load stabilized at 100% until 2100 when load was decreased to 95%.

  Load reduced to maintain adequate NPSH to steam generator feedwater pumps which was dropping because of increasing differential pressure across the condensate polishing system. Load stabilized at 95% while backflushing operations continue with the condensate polishing system to reduce the differential pressure across the polisher vessels. Load increased and stabilized at 100% by 1020 for the remainder of the day.
- 1/30 Load stabilized at 100% for the day.
- 1/31 Load stabilized at 100% for the day.

### REFUELING INFORMATION

DOCKET NO.:\_\_\_\_\_\_\_

	UNIT: Salem #1
	DATE: February 11, 1
CO.	MPLETED BY: L. K. Miller
	TELEPHONE: 609-365-7000
	X507
TH:	
Refueling information has changed from last:	month:
YES_X	NO
Conton	how 20 1000
	ber 20, 1980
Scheduled date for restart following refueli	ng: November 16, 1980
A. Will Technical Specification changes or	other license
amendments be required? YES	<del></del>
NOT DETERMINED	TO-DATE January 1980
B. Has the reload fuel design been reviewed	by the Station Operation
Review Committee? YES	NO_X
If no, when is it scheduled?	August 1980
Scheduled date(s) for submitting proposed li	censing action:
	•
August 1	980 (If required)
Important licensing considerations associate	980 (If required) d with refueling:
<del></del>	
Important licensing considerations associate	
Important licensing considerations associate NONE	
Important licensing considerations associate NONE  Number of Fuel Assemblies: A. In-Core	d with refueling:
Important licensing considerations associate NONE  Number of Fuel Assemblies: A. In-Core B. In Spent Fuel Storage	d with refueling:  193 40
Important licensing considerations associate NONE  Number of Fuel Assemblies: A. In-Core B. In Spent Fuel Storage Present licensed spent fuel storage capacity	d with refueling:  193 40
Important licensing considerations associate NONE  Number of Fuel Assemblies: A. In-Core B. In Spent Fuel Storage	193 40 : 264 1,170

Page 18 of 18 8-1-7.R4



Public Service Electric and Gas Company P.O. Box 168 Hancocks Bridge, New Jersey 08038

Salem Nuclear Generating Station

February 11, 1980

Director, Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT SALEM NO. 1
DOCKET NO. 50-272

In compliance with section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of January 1980 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Summary of Safety Related Maintenance
Operating Summary
Refueling Information

Sincerely yours,

H.J. While

H. J. Midura

Manager - Salem Generating Station

LKM:vd

cc: Mr. Boyce H. Grier
Director of U.S. NRC

Office of Inspection and Enforcement

Region I

631 Park Avenue

King of Prussia, PA 19406

Director, Office of Management Information and Program Control U.S. Nuclear Regulatory Commission Washington, DC 20555

Enclosures Page 1 of 18

## AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272

UNIT Salem #1

DATE February 11, 1980

COMPLETED BY L. K. Miller

TELEPHONE 609-365-7000

X 507

MONTH_	January 1980		X 507	
DAY AV	/ERAGE DAILY POWER LEVEL	DAY AVER	AGE DAILY POWER LEVEL (MWE-NET)	
1	342	17	924	
2	319	18	943	
3	369	19	1,047	
4	370	20	833	
5	515	21	1,069	
6	598	22	1,092	
7	507	23	379	
8	763		96	
9	788	25	856	
10	980	26	1,069	
11	1,100	27	1,084	
12	908	28	1,021	
13	1,023	29	1,048	
14	299		1,114	
15	0 .	31	1,114	
16	630	<del></del>		

Pg. 2 of 18 8.1-7.R1

#### OPERATING DATA REPORT

DOCKET NO.: 50-272

DATE: February 11, 1980
COMPLETED BY: L. K. Miller

TELEPHONE: 365-7000 X507

### OPERATING STATUS

1. Unit Name: Salem #1	Notes:		
2. Reporting Period: January 1980			:
3. Licensed Thermal Power (MWt): 3338			
4. Nameplate Rating (Gross MWe): 1135			İ
5. Design Electrical Rating (Net MWe): 1090			
6. Maximum Dependable Capacity (Gross MWe): 1	.124		
7. Maximum Dependable Capacity (Net MWe):	.079		
8. If Changes Occur in Capacity Ratings (Items No.	umber 3 Through 7) Since L	ast Report, Give Re	eason:
9. Power Level To Which Restricted, If Any (Net No. Reasons For Restrictions, If Any: NONE		NE	
	This Month	Year to Date	Cumulative
II. Hanna Ta Banaudiaia Danied	744	744	<b>22,70</b> 5
LL. Hours In Reporting Period  L2. Number Of Hours Reactor Was Critical	696.4	696.4	10,784.4
13. Reactor Reserve Shutdown Hours	0	0	22.7
14. Hours Generator On-Line	681.7	681.7	10,212.8
L5. Unit Reserve Shutdown Hours	0	0	0
L6. Gross Thermal Energy Generated (MWH)	1,789,716	,789,716	29,326,013
17. Gross Electrical Energy Generated (MWH)	586,080	586,080	9,753,580
LB. Net Electrical Energy Generated (MWH)	556,282	556,282	9,185,872
19. Unit Service Factor	91.6	91.6	45.0
20. Unit Availability Factor	91.6	91.6	45.0
21. Unit Capacity Factor (Using MDC Net)	69.3	69.3	37.5
22. Unit Capacity Factor (Using DER Net)	68.6	68.6	37.
23. Unit Forced Outage Rate	8.4	8.4	43.4
24. Shutdowns Scheduled Over Next 6 Months (Type, NONE	Date, and Duration of Eac	h):	
25. If Shut Down At End of Report Period, Estimate	ed Date of Startup:	N/A	
26. Units In Test Status (Prior to Commercial Opera	ation):	Forecast	Achieved
INITIAL CRITICALITY		9/30/76	12/11/76
INITIAL ELECTRICITY		11/1/76	$\frac{12/11/76}{12/25/76}$
8-1-7.R2 Pg. 3 of 18 COMMERCIAL OPERATION		12/20/76	6/20/77

#### UNIT SHUTDOWNS AND POWER REDUCTIONS

January 1980 REPORT MONTH

50-272 DOCKET NO .:

Salem #1 UNIT NAME:

February 11, 1980 DATE:

COMPLETED BY: L. K. Miller

609-365-7000 X507 TELEPHONE:

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
80-001	1/1/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-002	1/3/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-003	1/5/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-004	1/7/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-005	1/8/80	F	0	D	5		SH	ZZZZZZ	NRC Tech. Specs./Reg. Guides
80-009	1/11/80	F	0	A	5		нн	нтехсн	Clean Condensate Pumps Suction Strainers
80-015	1/14/80	F	37.1	A	3		RB	ZZZZZZ	Nuclear Instrumentation (Spurious noise spide on Power Range Channel N-43 while N-44 was in test).
80-017	1/17/80	F	0	Α	5		нн	нтехсн	Clean Condensate Pumps Suction Strainers
80-019	1/20/80	F	0	<b>A</b>	5		нн	нтехсн	Clean Condensate Pumps Suction Strainers

1

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3 Method:

1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Continuation of Previous Outage

5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data

Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1-Same Source UNIT SITURIOWNS AND POWER REDUCTIONS

REPORT MONTH January 1980

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

80-024 1/23	/23/80 F /23/80 F	0	A		 j		
1			_	5	 нн	нтехсн	Clean Condensate Pumps Suction Strainers
80-026 1/27		25.2	A	3	 НН	TRANSF	Loss of Auxiliary Transformer
	/27/80 F	0		5	нн	НТЕХСН	Clean Condensate Pumps Suction Strainers

Page 5 of 18 8-1-7.R1 MAJOR PLANT MODIFICATION
REPORT MONTH
January 1980

DOCKET NO.:

UNIT NAM

50-272 Salem #1

DATE:

February 11, 1980

COMPLETED BY:

L. K. Miller

TELEPHONE:

609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ED-0191	Sampling	Install Feedwater O, Analyzer
1ED-0343	Cathodic Protection	Install Ground Beds at S.W. Structure
1EC-0438	Circulating Water	Modify Controls for Manual Operation
1EC-0465	Control Air	Remove and Replace Piping for Condenser Retubing
1EC-0561	Main Turbine Lube Oil	Install Lube Oil Filter
1EC-0568	Fire Protection	Install Additional Aux. Building Hose Reel
1EC-0623	MSIV	Remove Overload Jumpers on Hydraulic Pump
1EC-0678	Containment Ventilation	Modify 1VC5 & 6 to Improve Operation
1EC-0683	4KV Vital Busses	Change Relay Setting
1EC-0699	Containment Sump	Install Vortex Baffles and Reset Level Switches
1MD-0051	Reactor Head	Modify Themocouple Trays
1PD-0037	Aux. Temperature Indication	Replace Bristol Recorders with the Doric
1SC-0018	Communications	Install S.P. Phone at Pressurizer
1SC-0025	Fuel Handling	Modify Fuel Transfer System in Accordance With Westinghouse WRAPS Package
1SC-0046	Fire Protection	Replace HP Turbine Fire Detector
1SC-0064	Overhead Annunciator	Modify Alarm Windows D-37 and D-45
1SC-0112	Control Room	Install Improved Clock
		·
	·	
'		·

MAJOR PLANT MODIFICATION January 1980 REPORT MONTH

DOCKET NO .: 50-272

UNIT NE: Unit #1

DATE:

February 11, 1980

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1ED-0191	This change is not safety related and does not affect safety related equipment.
1ED-0343	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unreviewed safety question per 10CFR 50.59.
1EC-0438	This system is not safety related. The change increases the flexibility of operation without impairing reliability. The type of materials employed in this change are identical to those used in the current design.
1EC-0465	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Technical Specifications are not affected.
1EC-0561	This design change will not require a change to the Tech. Specs. or FSAR. The additional filter equipment nor the surrounding equipment is safety related. This modification will improve the main turbine lube oil cleanliness and increase reliability.
1EC-0568	The installation of the hose station required by this DCR has been previously reviewed and it has been found that it does not present an unresolved safety question. No additional fire hazard is introduced by the addition of this hose station. It improves the plant safety features against fire.
1EC-0623	The modification does not change the functional operation of the system nor degrade its performance.
1EC-0678	The reworked valves will not adversely affect existing safety equipment or invalidate existing safety analysis. This design change will assure that the valve closes when required to.
1EC-0683	This change does not alter the functional design of the system. It reflects preferred method of applying protective relays. The existing design was necessary only because seismic data on preferred design was not previously available. No modifications affecting fire systems are made.
1EC-0699	Implementation of this DCR will insure a safer shutdown of the NSSS equipment following an accident involving a breach in the primary coolant system by providing anti-vortex baffles in the reactor building sump pit on the suction side of the residual heat removal pumps.
1MD-0051	Modifying the two cable trays does not involve any safety questions.

MAJOR PLANT MODIFICATIO REPORT MONTH January 1980 DOCKET\_NO.: \_\_\_\_50-272

UNIT NAME: Salem #1

TELEPHONE:

DATE: February 11, 1980

COMPLETED BY: L. K. Miller

609-365-7000 X507

*DCR NO.	10CFR50.59 SAFETY EVALUATION
1PD-0037	The system and instruments involved in this change are not safety related but the instruments are mounted in a Class I cabinet (1RP1). Class I cabinet requirements have been considered for 1RP1 and have not been altered by this change.
1SC-0018	This DCR will not affect any safety function of the system involved. The proposed changes do not involve any of the criteria associated with an unresolved safety question per 10CFR 50.59.
1SC-0025	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. The bases of the Tech. Specs. are not affected.
1SC-0046	This is not a safety related system and has no effect upon the safe shutdown capability of the plant.
1SC-0064	This change is not functionally safety related and supports a design analysis revising the alarms so that the alarm windows remain "black" unless both the reactor power is above 50% and the quadrant power tilt ration exceeds 1.02. It is classified as safety related because the work performed will be in a safety related cabinet.
1SC-0112	This design change is not safety related and does not affect any safety related systems or the safe shutdown of the unit.
	•

#### SALEM GENERATING STATION MONTHLY REPORT OF SAFETY RELATED REPAIRS

. • ' .		HONTHLY REPORT OF SAFETY R	ELATED REPAIRS	0001
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED.	DESCRIPTION OF REPAIR
		********		· · · · · · · · · · · · · · · · · · ·
		*** MAINTENANCE DEPARTMENT WORK ***		· ·- <u></u> ·.
		********		
900236	09 14 79	FLANGE LEAK NEAR VALVE 115W223	10 02 79	WELD REPAIRED PIPE AND APPLIED BELZONA COATING
900632	12 16 78	VALVE 115J143 HAS PACKING LEAK	04 11 79	REPACKED
901258	03.21.79	VALUE_11m5167_REPACK	<u> </u>	REPACKED
901366	04 14 79	VALVE 12584 REWORK	04 15 79	MACHINED DISC AND SEATINSTALLED_NEW GASKETSREPACKED
901819	04 30 79	VALVE 13HSII REWORK		
901620	04 30 79	VALVE 13MS12 REWORK	05 23 79 ·	DISASSEMBLED LAPPED ASSEMB <u>LED</u>
901621	04 30 79	VALVE 13MS13 REWORK	05 29 79	DISASSEHBLED ASSEHBLED
901622	04 30 79	VALVE 13HS14 REWORK	<u> </u>	DISASSE <u>HBLED</u> LAPPED ASSEMBLED
901623	04 30 79	VALVE 13HS15 REWORK	05 30 79	DISASSEMBLED LAPPED ASSEMBLED
901629	03 21 79	VALVE 12MS167 REPACK	05 10 79	REPACKED
901630	03 21 79	VALVE 13MS167 REPACK	05 10 79	REPACKED
901631	03 21 79	VALVE 14H5167 REPACK	<u> </u>	REPACKED
901725	04 27 79	VALVE 115J34 LEAKS THRU	05 14 79	LOOSTENED HANGER BOLT
902801	Ö4 11 79	VALVE 14GB4 FAILED LEAK RATE TEST	ŭ4 16 79	MACHINED DISC & SEAT INSTALLED NEW GASKETS REPACKED

		SALEM GENERATING STA MONTHLY REPORT OF SAFETY RI	0002	
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
				. •
903061	04 21 79	VALVE 14AF23 HAS PACKING LEAK	12 08 79	REPLACED BONNET SEAL REPACKED
903829	02 26 79	16 SERVICE WATER PUMP INSPECT & REPAIR	03 05 79	PUMP DISMANTLED AND SENT TO OUTSIDE VENDOR FOR WORK
905924	07 03 79	VALVE ISW223 BINDING	ŭ8 21 79	REPOSITIONED BELL CRANK REPACKED
913739	10 27 79	IA DIESEL GENERATOR EXHAUST MANIFOLD LEAKS		INSTALLED NEW GASKETS
914344	11 15 79	VALVE 11HS45 HAS PACKING LEAK		REPACKED
914345	11 15 79	· VALVE 12AF23 HAS FACNING LEAK	12_07_79	REPLACED BONNET SEAL
914346	11 15 79	VALVE 128F22 HAS PACKING LEAK	12	REPACKED
914349	11 15 79	VALVE 14MS130 HAS PACKING LEAK		REPACKED
914391	12 31 79	11 CHILLED WATER PUMP RUNNING HOT	ŭ1 ŭ1 80	REPLACED BEARINGS
914417	01 03 80	11 CHILLED WATER PUMP DILER LEANING	01_04_80	REPLACED <u>HECHANICAL SEAL</u> REPLACED OIL SEALS
914456_	01_04_80	11_CHILLED_WATER_PUMP_TRIPPING	01_05_80	REPLACED OVERLOADS
914574	01 14 80	VALUE 115W24 FLANGE LEAK	Öl 15 80	INSTALLED NEW VALVE BODY
915871	08 15 79	VALUE IRH26 HAS PACKING LEAK	08 17 79	REPACKED
915878	08 15 79	VALVE 11RH4Q HAS PACKING LEAK	08_17_ <u>79</u>	REPACKED
915880	08 15 79	VALVE 11RH18 HAS PACKING LEAK	Ŭ8 17 79	REPACKED
916090	12 13 79	VALVE 14AF21 LEAKS THRU	12 19 79	SHIPPED FOR REPAIR RETURNED TO SERVICE
916092	12 14 79	VALVE 11AF21 LEAKS THRU	12 19 79	REPLACED INTERNALS REPACKED
917436	11 06 79	VALVE 135W264 LEAKS THRU	11 22 79	REPLACED DIAPHRAGM
917502	11 11 79	15 CONTAINMENT FAN COIL ROUGHING FILTERS DIRTY		REPLACED FILTERS
917563	11 15 79	14 CONTAINMENT_FAN_SOIL ROUGHING FILTERS DIRTY	11 26 79	REPLACED FILTERS
918851_	01 16 80	13 AUXILIARY FEEDWATER FUMP START LIGHT DOES NOT LIGHT	01 17 80	REPAIRED_LIHIT SWIJCHESON VALVES 1H5132 AND 1H552

Page 10 of 18

#### SALEH GENERATING STATION HONTHLY REPORT OF SAFETY RELATED REPAIRS

•		MONTHLY REPORT OF SAFETY	RELATED REPAIRS	0003
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
919398	11 25 79	VALVE 12MS168 HAS PACKING LEAK	12 05 79	REPACKED
919399	11 25 79	VALVE_14MS168_HAS_PACKING_LEAK	12_06_79	PEPACKED T
919450	11 26 79	VALVE 1P52 HAS BROKEN REACH		
919464	11 27 79	VALVE 125J40 HAS PACKING LEAK	11 28 79	REPACKED
919539	12 01 79	11 BORIC ACID TRANSFER PUMP SEAL LEAKING	12 04 79	REPLACED MECHANICAL SEAL
919590	12 21 79	11 STEAM GENERATOR SNUBBERS	12 21 79	DRAINED OIL TIGHTENED SIGHT GLASS FILLED OIL AND TIGHTENED PACKING
919592	12 21 79	VITAL HEAT TRACING HEAT TAPE OGOZB NOT WORKING	12 26 79	REPAIRED
919593	12 21 79	VITAL HEAT TRACING HEAT TAPE 0622A DOES NOT WORK	12 27 79	REPAIRED HEAT TAPE
919611	11 15 79	REACTOR HEAD INSTRUMENT		·
		REPLACE GASKETS		
919615	11 16 79	VALVE 11RC21 HAS PACKING LEAK	11_21_79	REPACKED
919616	11 16 79	VALUE 11RC23 HAS PACKING LEAK	11 20 79	REPACKED
919620	11 16 79	VALVE 11RC17 HAS PACKING LEAK	11 21 79	REPACKED
919621	11 16 79	VALVE 11RC20 HAS PACKING LEAK	11_21_79	REPACKED
919622	11 16 79	VALVE 11RC25 HAS PACKING LEAK	11 21 79	REPACKED
919627	11 16 79	VALVE 13RC17 HAS PACKING LEAK	11 22 79	REPACKED
919632	11 16 79	VALVE 1PS13 HAS PACKING LEAK		REPACKED
919671	11 16 79	VALUE 14MS27 HAS PACKING LEAK	11 18 79	REPACKED
919673	11 16 79	VALVE 12M510 HAS PACKING LEAK	11 18 79	REPACKED
919675	11 16 79	VALVE 12MS18 HAS PACKING LEAK	11.1779	REPACKED
919676	11 16 79	VALVE 14MS18 HAS PACKING LEAK	11 17 79	REPACKED
919681	11 16 79	VALVE 14AF23 HAS PACKING LEAK	11 21 79	REPACKED
919700	11 18 79	VALUE_ICU2_HAS_PACKING_LEAK	11_22_79	REPACKED

		MONTHLY REPORT OF SAFETY		0004
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
919701	11 18 79	VALVE_125J154_HAS_PACKING.LEAK	11_21.79	REPACKED
919702	11 18 79	VALVE 15J149 HAS PACKING LEAK	11 21 79	REPACKED
919703	11 18 79	VALVE 13SJ138 HAS PACKING LEAK	11 21 79	REPACKED
<u>919708</u>	11 20 79	AIRLOCK ELEVATION 100 INSPECT INNER DOOR GASKET	11 20.79	REVERSED_INNER_AND_OUTER_DOOR SEALS
919725	11 24 79	VALVE 14RC26 HAS PACKING LEAK	1124 . 79	REPACKED
919753	11 30 79	VALVE 13RC24 HAS PACKING LEAK	12 05 79 .	REPACKED
919754	11 30 79	VALVE 13RC28 HAS PACKING LEAK	12 05 79	REPACKED CLEANED BORON OFF VALVE
919769	12 05 79	VALVE 13RC25 HAS PACKING LEAK	12 05 79	REPACKED
919770_	_12_05_79	VALVE 12AF21 LEAKS THRU	12_12_79	REPLACED STEM CAGE AND SEAT OF VALVE INSTALLED NEW GASKETS AND REPACK
919772	12 05 79	VALVE 1CV79 HAS PACKING LEAK	12 05 79	REPACKED
919774	12 05 79	16_SERVICE_WATER_PUMP	12_21_79	REBUILT PUMP. LOWER ASSEMBLY REBUILT BY A&A COMPANY
919783	12 06 79	VALUE 11MS9 HAS PACKING LEAK	11_08_79	REPACKED
919784	12 06 79	VALVE 11MS168 HAS PACKING LEAK	12 07 79	REPACKED
919785	12 06 79	VALVE 13MS168 HAS PACKING LEAK	12 07 79	REPACKED
919786	12 06 79	VALVE 13M545 HAS PACKING LEAK	12_08_79	REPACKED
919788	12 06 79	VALUE 14MS199 HAS PACKING LEAK	12 07 79	REPACKED
929769	12 17 79	VALVE ICV238 HAS BONNET LEAK	01 02 80	INSTALLED NEW DIAPHRAGM FIXED BUSHING
929862	12 22 79	VALVE 115W23 NO OPEN INDICA- TION IN CONTROL ROOM	12 04 79	REPLACED RELAY 33X-3
929877	12 23 79	16 SERVICE WATER STRAINER BROKEN SHEAR PIN	12 24 79	REPLACED SHEAR KEY AND BARREL SEAL PLATE
929922	12 27 79	VALVE 11MS167 WON'T OPEN	12 27 79	ADJUSTED PACKING
929933	12 27 79	CONTAINMENT SPRAY ADDITIVE	12_31_79	INSTALLED_GASKET_ON_ADDITIVE

,		MONTHLY REPORT OF SAFETY RE	LATED REPAIRS	0005
WORK ORDER	DATE WRITTEN	PROBLEM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
				,
929935_	12 29 79	12 FUEL HANDLING BUILDING EXHAUST FAN HAS BROKEN BELT	12_29_79	REPLACED BROKEN BELTS
929941	12 28 79	11 RESIDUAL HEAT REMOVAL SUMP ALARMS ON OVERFLOW	01_02_80	REPAIRED MICRO SWITCH ON ACTUATING DEVICE
931957	12 30 79	13_STEAH_FLOW_VENT_VALVE NEAR TRANSMITTER FT532	12_30_79	REPLACED BONNET OF SAMPLE VALVE
2		HAS BONNET LEAK		
931971	01 03 80	1A SAFEGUARD EMERGENCY CABINET INSPECTION BY VENDOR	o1 03 B0	REPLACED BAD RELAY
931997	01 08 80	PENETRATION I-25 & I-26 RECHARGE AND CHECK FOR LEAKS	01 09 80	RECHARGED FOUND NO LEAKS
		************		
3		*** PERFORMANCE DEPARTMENT WORK ***		
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905424	08 16 79	N-31 SOURCE RANGE SENSITIVE TO ELECTRICAL NOISE	12 13 79	REBUILT CABLE PLUGS REPLACED JACK ON PHD CIRCUIT
911838	10 11 79	N31 DETECTOR EXCESSIVE NOISE	10 19 79	REPLACED N31 AND N35 DETECTORS REBUILT TRIAXIAL PLUGS AT DRAWERS
914397	01 01 80	SIGNAL ISOLATOR IN N-43 OUT OF CALIBRATION	01 01 80	REPLACED SIGNAL ISOLATOR  QM203A SERIAL Q0455 WITH Q0453
914476	01 06 80	R31B FAILED FUEL PROTECTION READS LOW	01 06 80	REPLACED BOARD 1PB1CALIBRATED_DRAWER
914491	01 07 80	13 STEAM GENERATOR FEEWWATER FLOW CHANNEL II MIDRANGE FAILED	01 07 80	REPLACED AMPLIFIER BOARD RECALIBRATED
914521	01 09 80	VALVE 145W232LIMIT_SWITCH_ARM_BENT		STRAIGHTENED LIMIT SWITCH VERIFIED OPERATION
, 1				
915296	11 12 79	N41 CABLES SHORTED	11 14 79	LOCATED SHORT AND CLEARED IT
915322	12 05 79	N31_DETECTORVERIFY_PROPER DETECTOR VOLTAGE AND METER ACCURACY	12_05_79	REPLACED CABLE PLUG ON HIGH VOLTAGE POWER SUPPLY
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	,		SALEH GENERATING STATION MONTHLY REPORT OF SAFETY RELATED REPAIRS		vóoa .
١	WORK ORDER	DATE WRITTEN	PROBLÉM DESCRIPTION	DATE COMPLETED	DESCRIPTION OF REPAIR
1				COM CLIED	-
3	915615	11 15 79	N-42 SOURCE RANGE OUT OF SPEC	11 30 79	REPLACED HODULE XA5904RB WITH HODULE FROM UNIT 2
9	919394	11 24 79	VALVE 125W223 DOES NOT CONTROL FLOW TO 12 CONTAINMENT FAN COIL UNIT	11 25 79	BLEW DOWN TRANSMITTER FA31652-1 AND FA354021 REPLACED AMPLIEIE <u>R IN FA</u> 354021
17	919591	12 20 79	VALVE 13AF21DOES NOT REGULATE	12 22 79	REPLACED SQUARE ROOT EXTRACTOR FA3971 IAW DR# PD-0958
14	929776	12 18 79	AUDIO COUNT RATE SOUNDDISTORIED	12 19 79 .	
17 18	929806	12 14 79	12 REACTOR COOLANT PUMP LOW_SEAL_LEAK_OFF_FLOW INDICATOR/TRANSMITTER_STICKS	12 18 79	CLEANED AND FLUSHED FLOATREINSTALLED FLANGE WITH NEW GASKETS
20 21	929920	12 07 79		12 0 <b>7 79</b>	SOLDERED_BAD_CONNECTIONS
22 23 24		12 25 79	VALVE 14MS18 DOES NOT CLOSE	12 26 79	REPLACED SOLENOID VALVE
25 28		,			
?7 -3	TOTAL LI	NES = 000170	·		
30					
32 33	·				
и 15					,,
38 30					
ec -	<del>-   -   -   -   -   -   -   -   -   - </del>				
11.					
44 45			•		
15 17		,			
19 13					
51 -					

# SALEM UNIT #1 OPERATING SUMMARY JANUARY 1980

1/1 At 40% power, steady state conditions for entire day. 1/2 At 0330, commenced inserting rods by dilution to 140 steps to swing  $\Delta I$  negative in preparation for a flux map. Power remained at 40% for the whole day. 1/3 At 0500 on 1/3/80, waiting for  $\Delta I$  to stabilize for power coefficient Between 0640 and 0740, turbine load being increased then decreased. At 0750, commenced increasing power to 47% at 3% per hour. The reactor remained at 47% for the remainder of the day. 1/4 Power remained at 47% for the entire day. 1/5 Power being increased from 48% to 60% on 1/5/80 in preparation for thru next physics test. Once physics test was complete, power was reduced 1/6 to 50% by 2400 on 1/6/80 to clean feed pump strainers and to evaluate quadrant power tilt. 1/7 Power remained at 50% until authorization was given to go to 80%. At 2200 on 1/7/80, power escalation began at approximately 3% per hour. thru 1/8 Power was held at 80% for physics testing. At 2115 on 1/8/80, reactor power reduction began from 80% to 65% power to clean condensate pump strainers. 1/9 At 1200 on 1/9/80, power escalation began at 1.5% per hour to 95% which thru is the next physics testing plateau. Power remained at 95% through 1/10 1/10 while physics testing was in progress. 1/11 Still performing physics testing. At 0630 on 1/11/80, commenced power thru reduction at 3% per hour down to 70% to clean condensate pump strainers. 1/12 At 2100, with strainers cleaned, power escalation began at 3% per hour reactor thermal power. Power remained at 95% for the remainder of this

period.

- 1/13 At 95% power until 1400 when load reduction in order to perform governor valve closure of #12 stop valve was commenced. Load reduction terminated at 93% power and remained there for the remainder of the day.
- 1/14 Power escalated from 93% to 95%. At 0956 on 1/14/80, reactor tripped from 95% on a false high neutron flux rate trip signal. Reactor remained subcritical until 1013 on 1/15/80 when reactor was taken critical. Remained at zero power until reactor tripped at 1630 on steam generator low low level. Reactor brought critical until it tripped again at 1900 on steam generator low low level. Reactor brought critical and power escalation began. At 2400 on 1/15/80, power was 18.2%.
- 1/16 Power escalation continued. At 1234, experienced turbine load swing due to E.H. control system malfunction. Turbine placed in manual.

  At 2400, reactor power was at 89.5%.
- 1/17 Power reached 95% at 0400. At 1620, commenced dropping load to 70% to allow cleaning of main feedwater pump strainers. Power reduced to 55% to keep plant stabilized while cleaning strainers. At 2200, commenced increasing load to 95% at 5% per hour.
- 1/18 Load increase in progress. Load stabilized at 95% by 1600 and stayed there for the remainder of the day.
- 1/19 Load was stabilized at 95% power until 2010 when load reduction was commenced at 5% per hour.
- 1/20 Load reduction in progress at 5% per hour. Load stabilized at 70% by 0200 to allow cleaning of condensate pump strainers. Load increase commenced at 1245 at 3% per hour. Load at 100% by 2330.
- 1/21 Load stabilized at 100%.
- 1/22 Load stabilized at 100%. Load reduction commenced at 2200 to allow cleaning of condensate pump strainers.

- Load decrease in progress. Stabilized load at 70% at 0100. Condensate strainers cleaned and increasing load by 0345 at 3% per hour. Unit trip from 96% at 1029. 1H 460 volt group transformer dropped out of service which resulted in the tripping of a steam generator feedwater pump and a reactor trip on low steam generator water level. Unit down for the remainder of the day.
- 1/24 Reactor critical and load increase commenced at 1130. Load being increased over the remainder of the day.
- 1/25 Load increase in progress to 100%. Load stabilized at 100% by 1900.
- 1/26 Load stabilized at 100% power for the day.
- 1/27 Load stabilized at 100% until 2030 when a load decrease was commenced to 70% by 2300 to allow cleaning of condensate pump strainers.
- 1/28 Load increase commenced at 0115 from 70%. Load stabilized at 100% by 1030 for the remainder of the day.
- Load stabilized at 100% until 2100 when load was decreased to 95%.

  Load reduced to maintain adequate NPSH to steam generator feedwater pumps which was dropping because of increasing differential pressure across the condensate polishing system. Load stabilized at 95% while backflushing operations continue with the condensate polishing system to reduce the differential pressure across the polisher vessels. Load increased and stabilized at 100% by 1020 for the remainder of the day.
- 1/30 Load stabilized at 100% for the day.
- 1/31 Load stabilized at 100% for the day.

# REFUELING INFORMATION

¥	, DOC	CKET NO.: 50-272			
		UNIT: Salem #1			
		DATE: February 11, 19			
	COME	PLETED BY: L. K. Miller			
		TELEPHONE: 609-365-7000			
		X507			
MON	TH:				
1.	Refueling information has changed from last mo	onth:			
	: YES X NO	)			
2.	Scheduled date of next refueling: September	er 20, 1980			
3.	Scheduled date for restart following refueling: November 16, 1980				
4.	A. Will Technical Specification changes or of	ther license			
	amendments be required? YESNO	o			
	NOT DETERMINED	O-DATE January 1980			
	B. Has the reload fuel design been reviewed h	by the Station Operating			
	Review Committee? YES NO	>x			
	If no, when is it scheduled? Au				
5.	Scheduled date(s) for submitting proposed lice	ensing action:			
	August 198	0 (If required)			
6.	Important licensing considerations associated NONE	with refueling:			
		<del></del>			
7.	Number of Fuel Assemblies:				
	A. In-Core	193			
	B. In Spent Fuel Storage	40			
8.	Present licensed spent fuel storage capacity:	264			
	Future spent fuel storage capacity:	1,170			
9.	Date of last refueling that can be discharged pool assuming the present licensed capacity:	to the spent fuel September 1982			
	poor assuming the present incensed capacity:				