

Public Service Electric and Gas Company 80 Park Place Newark, N.J. 07101 Phone 201/430-7000

Ref. 79-04

January 22, 1979

Director of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention:

Mr. Albert Schwencer, Chief Operating Reactors Branch #1 Division of Operating Reactors

Gentlemen:

REQUEST FOR AMENDMENT FACILITY OPERATING LICENSE DPR-70 UNIT NO. 1 SALEM GENERATING STATION DOCKET NO. 50-272

In accordance with the Atomic Energy Act of 1954, as amended and the regulations thereunder, we hereby transmit copies of our request for amendment and our analysis of the changes to Facility Operating License DPR-70 for Salem Generating Station Unit No. 1.

This change is being sent in anticipation for the need for relief from Section 3.0.4 of the Safety Technical Specifications while under Action Statement 3.1.3.2.A.l of the Safety Technical Specifications.

This change is deemed to involve a single safety issue and is deemed not to involve a significant hazards consideration and, therefore, is determined to be a Class III amendment as defined by 10CFR170.22. A check for the amount of \$4,000 will be forwarded when it is determined that the actual need for relief is required.

This submittal includes three signed originals and 40 copies.

Very truly yours,

OULATORY DOCKET FILE COPY

F. P. Librižzi General Manager -Electric Production



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The Energy People

Ref. LCR 79-04

U.S. NUCLEAR REGULATORY COMMISSION DOCKET NO. 50-272

PUBLIC SERVICE ELECTRIC AND GAS COMPANY FACILITY OPERATING LICENSE NO. DPR-70 NO. 1 UNIT SALEM GENERATING STATION

Public Service Electric and Gas Company hereby submits proposed changes to Facility Operating License No. DPR-70 for Salem Generating Station, Unit No. 1. This change request relates to Safety Technical Specifications (Appendix A) of the Operating License, and pertains to relief from Section 3.0.4.

Respectfully submitted,

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

SCHNEIDER DERTCK W.

VICE PRESIDENT

STATE OF NEW JERSEY)) SS: COUNTY OF ESSEX)

FREDERICK W. SCHNEIDER, being duly sworn according to law deposes and says:

I am a Vice President of Public Service Electric and Gas Company, and as such, I signed the request for change to FACILITY OPERATING LICENSE NO. DPR-70.

The matters set forth in said change request are true to the best of my knowledge, information, and belief.

Subscribed and sworn to before me

this 22 day of <u>aniay</u>, 1979.

Notary Public of New Jersey

My commission expires on November 8,1983

BARBARA VALLEE A NOTARY PUBLIC OF NEW JERSEY My Commission Expires Nov. 8, 1983



TEMPORARY CHANGE TECHNICAL SPECIFICATIONS SALEM NO. 1 SECTION 3.0.4

Background

On Thursday, January 4, 1979, during the 4 x 12 shift, control rod 2D5 (D bank control rod) rod position indicator channel was determined to be inoperable as defined by Technical Specification 3.1.3.2 and Action Statement 3.1.3.2.A.1 was implemented.

The electronic circuitry was verified operable between the indicator meter and inside containment. Any further checks cannot be performed while the station is operating, because of radiation exposure levels. Depending on where the actual problem is, we estimate that correcting the problem would require between a one week and two week outage. This is based on bringing the reactor to cold shutdown and possibly having to remove the missile shield to get to the coil stack of the 2D5 control rod (center of core).

Relief Requested

Technical Specification 3.0.4 does not permit entry into an OPERATIONAL MODE while under an ACTION statement. We are requesting relief on this specification to permit entry into an operational mode while under action statement 3.1.3.2.A.1 until the start of the refueling outage, which is scheduled to commence on March 31, 1979.

Method of Reactor Startup

If the reactor is started with an inoperable rod position indicator the following steps will be followed to assure reactor safety is not jeopardized:

- 1. D bank control rods will be left fully inserted in the core prior to entering MODE 1.
- 2. After entering MODE 1 action statement 3.1.3.2.A.1 will be implemented to confirm alignment of control rod 2D5 as D bank control rods are withdrawn from the reactor core.

Safety Evaluation

Changing of reactor modes will not be affected by the 2D5 rod position indicator being inoperable because the D bank control rods will be left fully inserted in the core prior to entering MODE 1. - 2 -

movable incore detectors as allowed by Technical Specification Action Statement 3.1.3.2.A.1. Since this method is currently addressed in the technical specifications, no unreviewed safety questions are involved in this proposed change.