AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272
UNIT Salem #1
DATE November 9, 1978
COMPLETED BY L. K. Miller
TELEPHONE 609-365-7000 X507

MONTH October, 1978

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	495	17	0
2	524	18	0
3	561	19	0
4	613	20	0
5	488	21	0
6	516	22	0
7	607	23	0
8	761	24	0
9	904	25	0
10	741	26	0
11	639	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

OPERATING DATA REPORT

DOCKET NO.: 50-272
DATE: November 9, 1978
COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000 X507

OPERATING STATUS

1. Unit Name: Salem #1
2. Reporting Period: October, 1978
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1135
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
NONE

Notes:

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons For Restrictions, If Any: NONE

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	745	7,296	11,737
12. Number Of Hours Reactor Was Critical	258.4	4,179.8	6,723.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	258.4	3,985.3	6,415.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	564,691.2	11,711,663.8	18,406,883.8
17. Gross Electrical Energy Generated (MWH)	174,850	3,919,360	6,105,670
18. Net Electrical Energy Generated (MWH)	159,265	3,708,825	5,767,043
19. Unit Service Factor	34.7	54.6	54.7
20. Unit Availability Factor	34.7	54.6	54.7
21. Unit Capacity Factor (Using MDC Net)	19.8	47.1	45.5
22. Unit Capacity Factor (Using DER Net)	19.6	46.6	45.1
23. Unit Forced Outage Rate	62.2	45.0	36.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Unit Refueling 3/31/79 - 6/23/79

25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/5/78
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	9/30/76	12/11/76
INITIAL ELECTRICITY	11/1/76	12/25/76
COMMERCIAL OPERATION	12/20/76	6/30/77

REPORT MONTH October

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: November 9, 1978
COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-235	10/3/78	F	0.0	A	4A	- - -	HH	HTEXCH	11B Condenser Tube Leak
78-236	10/4/78	F	0.0	A	4A	- - -	HH	PUMPXX	11 SGFP Inboard Journal Bearing Overtemperature
78-237	10/4/78	F	0.0	A	4A	- - -	HH	PUMPXX	11 SGFP Inspect & Repair Inboard Journal Bearing
78-238	10/4/78	F	0.0	A	4A	- - -	HH	PUMPXX	11 SGFP Bearing Inspection & 11 Vac. Pump Cooler Replacement
78-241	10/6/78	F	0.0	B	4A	- - -	HH	PUMPXX	11 SGFP Bearing Inspection
78-243	10/7/78	F	0.0	B	4A	- - -	HH	PUMPXX	11 Cond. Pump Lower Motor Bearing Inspection
78-245	10/8/78	F	0.0	A	4A	- - -	HF	FILTER	12B Traveling Screen Repair
78-246	10/10/78	F	0.0	A	4A	- - -	HH	PUMPXX	12 Cond. Pump - Install Ground Brush & Uncouple Motor

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure(Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error(Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER) File (NUREG-0161)

5
Exhibit 1-Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: November 9, 1978
COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-249	10/11/78	F	111.6	A	2	- - -	HH	HTEXCH	11A & 11B Condenser Tube Leak & Secondary Chemistry Problems
78-250	10/16/78	F	72.0	A	4B	- - -	PC	VALVEX	Repair CV78 and Inspection of Steam Generators
78-251	10/18/78	F	65.0	A	4B	- - -	SF	PUMPXX	Replace 12 Safety Injection Pump
78-252	10/21/78	F	248.0	A	4B	- - -	CB	PUMPXX	Failure of Seals on 13 RCP

MAJOR PLANT MODIFICATIONS
REPORT MONTH October 1978

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: November 9, 1978
COMPLETED BY: L. K. Miller
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*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1ED-0316	Service Air	Place No. 2 SAC In Service
1ED-0320	Circ Water	Modify Waterbox Gratings
1ED-0356	Fuel Handling	Modify Manipulator Crane Control Circuit
1ED-0364-12	Service Water	Install Modified No.12 S.W. Pump
1EC-0375	Gland Seal	Remove Test Orifice
1EC-0397	Circ Water	Modify Diff. Indicator
1EC-0405	RMS 1-R-22	Setpoint change 2.5mR/Hr. to 25mR/Hr.
1MD-0054	25kV	Modify Gen. Potential Ind.
1MD-0093-B	Condensate	Install Modified Bearing in No. 13 Condensate Pump
1PD-0036	Aux. Annunciator	Nomenclature Change for Points 573 & 594
1PD-0159	Feed & Condensate	Install Test & Vent Valves in SGFP Turbine Gov. Actuators

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS
REPORT MONTH October, 1978

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: November 9, 1978
COMPLETED BY: L. K. Miller
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*DCR NO.	10CFR50.59 SAFETY EVALUATION
1ED-0316	Compressor #2 was jumpered out because only the No.'s 1 & 3 air compressors were turned over with Unit #1. The DCR completes the original design (i.e. servicing of three Station Air Compressors). None of the conditions for an unreviewed safety question exist.
1ED-0320	This does not alter any approved design criteria, procedures, or set points. Safety related equipment will not be affected.
1ED-0356	It will decrease the margin of safety now specified in the Technical Specification.
1ED-0364-12	A revised seismic analysis has been done on the new aluminum bronze column pipe. The impeller modification produced a flatter head-capacity curve without affecting design flow point, (see SGS/M-DM-077). System still performs original design function. Probability of accidents has not increased, nor has the occurance of new accidents been made possible. The Tech. Spec. bases are unaffected.
1EC-0375	This design change does not affect any presently performed safety analysis, nor does it create any new safety hazards. The basis of the Technical Specifications are not affected. System is Non-Nuclear, and Non-Safety Related.
1EC-0397	This system is not safety related; the function affected by this change does not impact a safety related system, nor create conditions requiring a safety analysis.
1EC-0405	This monitor and its setting does not affect the safe operation or shutdown of the plant. It will also not invalidate any safety analysis or require that a new one be performed.
1MD-0054	Safety related equipment will not be affected.
1MD-0093-B	This design change does not affect any safety related systems or components.
1PD-0036	Safety related equipment will not be affected.
1PD-0159	The design change will improve system operation. No functional system change is involved.

* DESIGN CHANGE REQUEST

DOCKET NO.: 50-272
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DATE: November 9, 1978
COMPLETED BY: L. K. Miller
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MMARY OF SAFETY RELATED MAINTENANCE
DEPARTMENT Performance
REPORT MONTH October, 1978

ORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
PD-5021	RMS R11A	Perform Channel Calibration Check.	Completed Cal. Check.
PD-5033	1FT-445	Erratic Setpoint.	Tightened loose connection
PD-5031	1PC456A	Trips High-Output unstable.	Replaced module.
PD-5035	1B13E	Drawer out of spec.	Replaced meter.
PD-5036	1FM425	No flow indication.	Replaced.
PD-5023	Channel N41	Channel N41 out of spec.	Calibrated.
PD-5084	1XD581QCB.	Blue Ribbon connector broken	Replaced module.
PD-4803	1TM412P, 1TM412Q, 1TM505C.	Faulty capacitors.	Replaced capacitors & calibrated.
PD-5091	1XD581CB	Module dropped.	Replaced connectors & switch.
OD-10109	Control Room Area Vent System	1CAA1 Damper will not position properly.	Adjusted limit switch.
OD-10098	13 Aux. Feed Pump	1MS132 Failed to open - Pump inoperable.	Tightened loose clamp & diaphragm nut.
OD-10226	Valve 1CC215	No open indication.	Extended actuator.
OD-10660	1FA1550Z	Inoperable liquid release flow recorder	Replaced transmitter.
PD-5125	SV-913, SV-915	Leaking SV for 12 & 14 Reactor Nozzle Support Fan.	Replaced internals.
PD-5126	SV-910, SV-911.	Leaking SV for 11 & 12 Reactor Shield Vent Fan.	Rebuilt valves.

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: November 9, 1978

COMPLETED BY: L.K. Miller

TELEPHONE: 609-365-7000 X507

SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Performance

REPORT MONTH October, 1978

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
EC-1465	Valve, 1LW15	Do not have open & closed indication.	Tightened packing nut & adjusted valve stem.
OD-10095	Solid state train B - NIS relays	Relay 27A Malfunctions	Replaced relay 27A & input card
PD-4970	14 Reactor Coolant Loop ΔT/Tavg.	Recalibrate to new valves.	Calibrated.
PD-5098	Valve, 12SW058	No open indication.	Adjusted open limit switch.
PD-5100	1FA3172, 1FA3542	No flow indication.	Vented inlet & outlet flow transmitters.
PD-5096	1FC510A/B	1BS510A out of spec.	Module setpoints adjusted.
OD-10245	Valve, 14SW223	Inoperable.	Vented 1FA3543Z & 1FA3176Z
OD-10231	Valve, 13SW057, 13SW058, 13SW072	Valve will not maintain flow.	Vented 1FA3169Z.
RE-0179	Channel N44	ΔI Meter Calibration drift.	Replaced detector "B" current meter and selector switch.
OD-10286	13 Fan coil unit	Investigate Low Flow Alarm.	Blew down inlet & outlet transmitters.
PD-5014	11 Reactor Coolant Loop ΔT/Tavg.	Comparator Tripping Low.	Replaced ITC41G/H
PD-4968	11 Reactor Coolant Loop ΔT/Tavg	Recalibrate to new valves.	Calibrated.
PD-4928	Channel N44	Recorder pen oscillates.	Tightened loose connection.
PD-4991	Valve, 11MS169	Restroke Valve.	Stroked valve.

DOCKET NO.: 50-272

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SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Performance

REPORT MONTH October, 1978

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-10350	1LI459A, 1LI460A, 1LI461A.	All channels read differently.	Channels I, II, & III calibrated.
OD-10731	Channel N32	Meter pegged High.	Tightened loose connector & replaced pen.
OD-10741	1FC116	Bezel Alarm - 13 RCP Flow is zero.	Replaced module.
OD-10729	Valve, 12SW122	Valve inoperable.	Blew down transmitter & recalibrated.
PD-5096	1FC510A/B	1BS510A out of spec.	Calibrated.
PD-4991	Valve, 11MS169	Faulty operation-control air leak.	Repaired leak.
PD-5008	Channel N44	Out of spec.	Adjusted 1NM307 Gain.
MD-3079	1BD627A	Blown output diode.	Replaced diode.
PD-5124	Valve, 11BF019	Restroke valve.	Valve stroked.
PD-5103	1FA3172Z-1, 1FA3572Z-1, 1FA3542Z-1.	Low Flow Indication.	Recalibrated.
PD-5114	Channel N42	Adjust Isolation Amplifiers.	Adjusted I/A 306 & 307.
PD-5119	1PM403D, 1PM405B.	Out of spec.	Replaced 1PM405B & repaired 1PM403D.
PD-5110	1FM543C	1BS541B out of spec.	Replaced.

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: November 9, 1978

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MONTHLY SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Performance

REPORT MONTH October, 1978

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
CD-10414	RMS 1R13E	Out of spec.	Calibrated.
PD-5140	1TC107A/B	No output from 1TC107A/B	Replaced module.
PD-5136	1LT-459	Equalize transmitter.	Completed.
PD-5123	Valve, 11MS169	Restroke valve.	Completed.
PD-5098	12 Fan Coil Unit	Adjust limit switches on valve 12SW058.	Adjusted open limit.

MARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Maintenance

REPORT MONTH October, 1978

UNIT NAME: Salem #1

DATE: November 9, 1978

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

ORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
MD-3040	Valve, 13MS10	Packing leak.	Tightened packing gland.
OD-10453	Valve, 13CV78	Bonnet leak.	Replaced bonnet.
MD-3037	Valve, 11MS10	Packing leak, nut missing.	Repacked-replaced nut.
OD-10485	13 Containment fan coil unit.	Motor cooler inoperable	Replaced motor cooler.
OD-10487	IC Diesel	Compressor will not maintain tank pressure.	Tightened packing glands.
OD-10484	Valve, 14SW223	Valve leaks.	Welded temporary patch.
OD-10497	Panels 241-1, 235, 234	NRC Inspection - Do Not have proper door seals	Replaced door seals.
OD-10531	Valve, 12CV157	Diaphragm failed.	Replaced diaphragm.
OD-10588	14 Service Water Pump Strainer	Will not start in "Auto" mode.	Recalibrated pressure switch.
OD-10585	14 RHR Sump Pump	Will not shut down automatically.	Adjusted motor control relay contacts.
OD-10599	IC Diesel	Failed pressure switch.	Replaced switch.
OD-10542	Valve, 12SW024	Diaphragm failed.	Replaced diaphragm.
OD-10581	16 Service Water Pump	Flex Seal Water Supply line leaking.	Replaced with copper tubing.
OD-10457	Valve, 12CV162	Will not stroke.	Replaced diaphragm.

SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Maintenance

REPORT MONTH October, 1978

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: November 9, 1978

COMPLETED BY: L. K. Miller

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WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
MD-2901	Valve, 1MS168	Packing leak.	Repacked valve.
OD-10527	Batch Tank	Drains plugged.	Valve 1CV144 diaphragm replaced & Valve 1CV145 cleaned.
MD-1393	Westinghouse BFD Relays	Perform DCR ED-0009 Replace BFD Type Relays	Completed.
OD-7431	14 RHR Sump Pump	False indication.	Relaced motor, pump & magnatrol.
OD-10303	Valve, 10SW240	Inoperable	Cleaned & greased.
MD-3092	Valve, 1PS010	Valve leaks.	Replaced valve.
MD-3088	100' Elevation air lock.	Bent hinge shaft & worn seal.	Repaired hinge & replaced seal.
MD-3086	100' Elevation air lock.	Excessive air flow.	Adjusted door.
OD-10178	Valve, 12CS002	Packing leak.	Repacked valve.
OP-0366	11 Component Cooler	Tube leaks.	Pulled a tube & installed plugs.
MD-2997	Valve, 11MS169	Inoperable.	Reworked stem, replaced gaskets & repacked.
OD-10614	Valve, 13SJ024	Packing leak.	Tightened packing.
OD-10503	100' Containment air lock	Excessive air flow.	Adjusted inner door latch.
OD-9279	14 Service Water Pump Strainer	High ΔP indication.	Replaced local gauges.

MARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Maintenance

REPORT MONTH October, 1978

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: November 9, 1978

COMPLETED BY: L. K. Miller

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ORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-9286	Valve, 12SW022	Motor operator failed.	Replaced motor.
MD-2902	Valve, 11MS168	Packing & bonnet leak.	Bonnet gasket & packing replaced.
OP-0487	Valve, 1CV075	Leaking.	Tightened packing gland.
MD-3054	1FE499A	Leaking.	Replaced gaskets.
OD-10612	14 Steam Gen. Pressure tap	Leaking.	Repacked.
OD-10726	Valve, 12SJ045	Failed stroke time.	Re-timed valve.
OD-10727	Valve, 10SW026	Failed stroke time.	Re-timed valve.
OD-10603	13 Containment fan coil unit	Torn rubber at fan outlet plenum.	Patched all holes in plenum
OD-10606	15 Containment fan coil unit	Leak.	Sealed cooling coil housing.
OD-10604	12 Containment fan coil unit.	Torn rubber at fan outlet plenum.	Repaired all holes in plenum
OD-10718	11 Service Water Pump strainer	Broken shear pin.	Replaced 2 new lower shoes & shear key.
OD-10476	12 Boric Acid transfer pump	Inboard pump seal leaks.	Installed new seal.
OD-10602	14 Containment fan coil unit.	Torn rubber at fan outlet plenum.	Repaired all holes in plenum
MD-2697	12 Reactor coolant pump.	Remove 0.1" Shim from No. 3 Seal & test run.	Tested & reinstalled 0.1" Shim.

DOCKET NO.: 50-272

UNIT NAME: Salem #1

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SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT Maintenance

REPORT MONTH October, 1978

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
-9566	14 Steam Gen. Flow Trap	Valve packing leaks.	Adjusted packing.
-10760	Valve, 11CS001	Valve position indicator disconnected.	Replaced.
-8700	Valve, 14SW20	Inoperable valve operator.	Replace broken gear on hand crank.
-2800	Valve 1S25	Does not pass flow.	Replaced valve.

10-1 A load increase was commenced at 0025 from 50% power to 55% power at 0110. The unit remained at 55% power for the remainder of the day.

10-2 The unit load was held at 55% power for the entire day. Unit load is being held at 55% while maintenance is being performed on #11 Steam Generator Feed Pump.

10-3 Unit load was held at 55% until 1605 when a power escalation was initiated at 3% per hour in order to evaluate #11 Steam Generator Feed Pump operation. Unit load was leveled off at 74% at 2440. Load reduction was commenced at 2310 from 74% to 67% at 2335.

10-4 Unit load was held at 67% to evaluate #11 Steam Generator Feed Pump operation. Load was reduced at 0310 from 67% to 60% by 0420. A load increase was initiated at 0505 from 60% at 3% per hour to 70% by 0800. At 0900, load increase was commenced at 3% per hour to 75% at 1020. Load was held at 75% to evaluate the operation of #11 Steam Generator Feed Pump. Load decrease was commenced at 1255 from 75% at a rate of 5% per hour. Load was stabilized at 54% at 1710 and remained at 54% for the remainder of the day.

10-5 thru 10-6 Unit load remained at 55% while maintenance was being performed on #11 Steam Generator Feed Pump.

10-7 Load was held at 55% until 0605 when a load increase at 4% per hour was initiated. Load was stabilized at 69% at 0910. Load was held at 69% until 1620 when load was rapidly reduced to 51% after #11 Steam Generator Feed Pump tripped. Load was held at 51% until 1805 when a load increase was initiated at 4% per hour to 69% at 2220. Load was held

10-7 (cont) at 69% for the remainder of the day.

10-8 Unit load was held at 69% until 1440 when a load increase was commenced at 5% per hour to 92% at 1940. Unit load remained at 92% until 2135, when load was reduced to 86% by 2225, due to two circulators being out of service on #12 Condenser Shell. Unit load remained at 86% for the remainder of the day.

10-9 Unit load remained at 86% for the day.

10-10 Unit load remained at 86% until 0355 when a load decrease was initiated at 5% per hour due to Bearing trouble on #12 Condensate Pump. Load was stabilized at 70% at 0650. Load was maintained at 70% for the remainder of the day.

10-11 Load increase was commenced at 0001 at 5% per hour and stabilized at 89% at 0405. Load was held at 89% due to circulators 12A and 12B being out of service. At 0740 a load decrease was initiated at 5% per hour to 82% power at 0820. Load was decreased to take a third circulator out of service. Load remained at 82% until 1300. Load was gradually increased to 84% by 1500. At 1545 a load increase was initiated at 3% per hour to 91% at 1710. Load was held at 91% until condenser tube leaks required load to be decreased at 1720. At 1825 the reactor was tripped from 77% due to excessive condenser tube leakage. The unit remained shutdown for the remainder of the day.

10-12 thru 10-31 On Saturday, October 21, the plant was being heated up and pressurized in preparation for Start-Up when at 2130, #13 RCP Seal failed causing the RCS to depressurize from 850 PSIA. The remainder of the month, the unit was shutdown for #13 RCP Seal replacement and containment deconing.

REFUELING INFORMATION

DOCKET NO.: 50-272
UNIT: Salem#1
DATE: November 9, 19
COMPLETED BY: L. K. Miller
TELEPHONE: 609-365-7000
Ext-507

MONTH: October, 1978

1. Refueling information has changed from last month:

   YES X NO

2. Scheduled date of next refueling: March 31, 1979
3. Scheduled date for restart following refueling: June 23, 1979
4. A. Will Technical Specification changes or other license amendments be required? YES NO

   NOT DETERMINED TO-DATE October, 1978

   B. Has the reload fuel design been reviewed by the Station Operatin Review Committee? YES NO X

   If no, when is it scheduled? January, 1979
5. Scheduled date(s) for submitting proposed licensing action:

   February, 1979 If Required
6. Important licensing considerations associated with refueling:

   NONE
7. Number of Fuel Assemblies:

   A. In-Core 193

   B. In Spent Fuel Storage 0
8. Present licensed spent fuel storage capacity: 264

   Future spent fuel storage capacity: 1,170
9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: April, 1982