

ATTACHMENT TO LICENSE AMENDMENT NO. 117

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

| <u>REMOVE</u> | <u>INSERT</u> |
|---------------|---------------|
| 7 | 7 |

Appendix A to Facility Combined License Nos. NPF-91 and NPF-92

| <u>REMOVE</u> | <u>INSERT</u> |
|---------------|---------------|
| 1.1-6 | 1.1-6 |
| 3.3.8-3 | 3.3.8-3 |
| 3.3.8-6 | 3.3.8-6 |
| 3.3.8-7 | 3.3.8-7 |
| 3.3.9-2 | 3.3.9-2 |
| 3.3.9-3 | 3.3.9-3 |
| 3.3.9-5 | 3.3.9-5 |
| 3.3.10-2 | 3.3.10-2 |
| 3.3.10-3 | 3.3.10-3 |
| 3.3.10-5 | 3.3.10-5 |
| 3.3.15-2 | 3.3.15-2 |
| 3.3.16-4 | 3.3.16-4 |
| 3.4.11-1 | 3.4.11-1 |
| 3.4.11-2 | 3.4.11-2 |
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| 3.4.12-1 | 3.4.12-1 |
| 3.4.12-2 | 3.4.12-2 |
| 3.4.13-1 | 3.4.13-1 |
| 3.4.13-2 | 3.4.13-2 |
| --- | 3.4.13-3 |
| 3.5.3-1 | 3.5.3-1 |
| 3.5.3-2 | 3.5.3-2 |

Appendix C to Facility Combined License No. NPF-91

| <u>REMOVE</u> | <u>INSERT</u> |
|---------------|---------------|
| C-409 | C-409 |
| C-426 | C-426 |

(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in FSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 117, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

1.1 Definitions

TRIP ACTUATING DEVICE
OPERATIONAL TEST
(TADOT)

A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the necessary accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps.

VENTED

VENTED shall be the condition when all required flow paths in ADS stage 1, 2, and 3, or alternative flow path with equivalent area, required by LCO 3.4.13, "Automatic Depressurization System (ADS) - Shutdown, RCS Open," are open.

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|--|---|
| J. As required by Required Action C.1 and referenced in Table 3.3.8-1. | J.1 Be in MODE 5. <u>AND</u> J.2 Initiate action to establish RCS VENTED. | 37 hours with three or more inoperable channels <u>AND</u> 180 hours 180 hours |
| K. As required by Required Action C.1 and referenced in Table 3.3.8-1. | K.1 Suspend positive reactivity additions. <u>AND</u> K.2 Initiate action to open RCS pressure boundary and establish $\geq 20\%$ pressurizer level. | Immediately Immediately |
| L. As required by Required Action C.1 and referenced in Table 3.3.8-1. | L.1 Suspend positive reactivity additions. <u>AND</u> L.2 Initiate action to remove the upper internals. | Immediately Immediately |
| M. As required by Required Action C.1 and referenced in Table 3.3.8-1. | M.1 Suspend positive reactivity additions. <u>AND</u> M.2 Be in MODE 5. <u>AND</u> M.3 Initiate action to establish a pressurizer level $\geq 20\%$ with the RCS pressure boundary intact. | Immediately 12 hours 12 hours |

Table 3.3.8-1 (page 1 of 2)
Engineered Safeguards Actuation System Instrumentation

| FUNCTION | APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS | REQUIRED CHANNELS | CONDITIONS |
|--|--|----------------------|------------|
| 1. Containment Pressure | | | |
| a. – Low | 1,2,3,4,5 ^(a) ,6 ^(a) | 4 | P |
| b. – Low 2 | 1,2,3,4,5 ^(a) ,6 ^(a) | 4 | P |
| 2. Containment Pressure – High 2 | 1,2,3,4 | 4 | H |
| 3. Containment Radioactivity – High | 1,2,3,4 ^(b) | 4 | I |
| 4. Containment Radioactivity – High 2 | 1,2,3 | 4 | I |
| 5. Pressurizer Pressure – Low 3 | 1,2,3 ^(c) | 4 | E |
| 6. Pressurizer Water Level – Low | 1,2 | 4 | D |
| 7. Pressurizer Water Level – Low 2 | 1,2,3,4 ^(b) | 4 | F |
| | 4 ^(d) ,5 ^(e) | 4 | J |
| 8. Pressurizer Water Level – High | 1,2,3 | 4 | I |
| 9. Pressurizer Water Level – High 2 | 1,2,3,4 ^(f) | 4 | I |
| 10. Pressurizer Water Level, High 3 | 1,2,3,4 ^(f) | 4 | F |
| 11. RCS Cold Leg Temperature (T _{cold}) – Low 2 | 1,2,3 ^(c) | 4 per loop | E |
| 12. Reactor Coolant Average Temperature (T _{avg}) – Low | 1,2 | 4 | D |
| 13. Reactor Coolant Average Temperature (T _{avg}) – Low 2 | 1,2 | 4 | D |
| 14. RCS Wide Range Pressure – Low | 1,2,3,4 | 4 | H |
| | 5 | 4 | K |
| | 6 ^(g) | 4 | L |

- (a) Without an open containment air flow path ≥ 6 inches in diameter.
- (b) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
- (c) Above the P-11 (Pressurizer Pressure) interlock, when the RCS boron concentration is below that necessary to meet the SDM requirements at an RCS temperature of 200°F.
- (d) With the RCS being cooled by the RNS.
- (e) With RCS not VENTED and CMT actuation on Pressurizer Water Level - Low 2 not blocked.
- (f) Above the P-19 (RCS Pressure) interlock with the RCS not being cooled by RNS.
- (g) With upper internals in place.

Table 3.3.8-1 (page 2 of 2)
Engineered Safeguards Actuation System Instrumentation

| FUNCTION | APPLICABLE MODES OR | | REQUIRED CHANNELS | CONDITIONS |
|---|--|--|---------------------|------------|
| | OTHER SPECIFIED CONDITIONS | | | |
| 15. Core Makeup Tank (CMT) Level – Low 3 | 1,2,3,4 ^(b) | | 4 per tank | F |
| | 4 ^(d) ,5 ^(h) | | 4 per OPERABLE tank | J |
| 16. CMT Level – Low 6 | 1,2,3,4 ^(b) | | 4 per tank | F |
| | 4 ^(d) ,5 ^(h) | | 4 per OPERABLE tank | J |
| 17. Source Range Neutron Flux Doubling | 2 ⁽ⁱ⁾ ,3 ⁽ⁱ⁾ ,4 ⁽ⁱ⁾ | | 4 | I |
| | 5 ⁽ⁱ⁾ | | 4 | I |
| 18. IRWST Level – Low 3 | 1,2,3,4 ^(b) | | 4 | F |
| | 4 ^(d) ,5 | | 4 | M |
| | 6 ^(g) | | 4 | N |
| 19. Reactor Coolant Pump Bearing Water Temperature – High 2 | 1,2,3,4 | | 4 per RCP | O |
| 20. SG Narrow Range Water Level – Low | 1,2,3,4 ^(b) | | 4 per SG | F |
| 21. SG Wide Range Water Level – Low | 1,2,3,4 ^(b) | | 4 per SG | F |
| 22. SG Narrow Range Water Level High | 1,2,3,4 | | 4 per SG | I |
| 23. SG Narrow Range Water Level – High 3 | 1,2 | | 4 per SG | D |
| | 3,4 | | 4 per SG | I |
| 24. Steam Line Pressure – Low 2 | 1,2,3,4 ^(b) | | 4 per steam line | G |
| 25. Steam Line Pressure – Negative Rate – High | 3 ^(k) | | 4 per steam line | I |

(b) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(d) With the RCS being cooled by the RNS.

(g) With upper internals in place.

(h) With RCS not VENTED.

(i) With unborated water source flow paths not isolated except when critical or except during intentional approach to criticality.

(j) With unborated water source flow paths not isolated.

(k) Below the P-11 (Pressurizer Pressure) interlock.

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|---------------------------------|
| <p>C. Required Action and associated Completion Time of Condition A or B not met.</p> <p><u>OR</u></p> <p>One or more Functions with two channels inoperable.</p> | <p>C.1 Enter the Condition referenced in Table 3.3.9-1 for the channel(s).</p> | <p>Immediately</p> |
| <p>D. As required by Required Action C.1 and referenced in Table 3.3.9-1.</p> | <p>D.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>D.2 Be in MODE 4 with the Reactor Coolant System (RCS) cooling provided by the Normal Residual Heat Removal System (RNS).</p> | <p>6 hours</p> <p>24 hours</p> |
| <p>E. As required by Required Action C.1 and referenced in Table 3.3.9-1.</p> | <p>E.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>E.2 Be in MODE 5.</p> | <p>6 hours</p> <p>36 hours</p> |
| <p>F. As required by Required Action C.1 and referenced in Table 3.3.9-1.</p> | <p>F.1 Declare affected isolation valve(s) inoperable.</p> | <p>Immediately</p> |
| <p>G. As required by Required Action C.1 and referenced in Table 3.3.9-1.</p> | <p>G.1 Be in MODE 5.</p> <p><u>AND</u></p> <p>G.2 Initiate action to establish RCS VENTED.</p> | <p>12 hours</p> <p>12 hours</p> |

ACTIONS (continued)

| CONDITION | | REQUIRED ACTION | COMPLETION TIME |
|-----------|---|---|-----------------|
| H. | As required by Required Action C.1 and referenced in Table 3.3.9-1. | H.1 Suspend positive reactivity additions. | Immediately |
| | | <u>AND</u> | |
| | | H.2 Initiate action to establish RCS VENTED and establish $\geq 20\%$ pressurizer level. | Immediately |
| I. | As required by Required Action C.1 and referenced in Table 3.3.9-1. | I.1 Suspend positive reactivity additions. | Immediately |
| | | <u>AND</u> | |
| | | I.2 Initiate action to remove the upper internals. | Immediately |
| J. | As required by Required Action C.1 and referenced in Table 3.3.9-1. | J.1 Suspend positive reactivity additions. | Immediately |
| | | <u>AND</u> | |
| | | J.2 Be in MODE 5. | 12 hours |
| | | <u>AND</u> | |
| | | J.3 Initiate action to establish a pressurizer level $\geq 20\%$ with the RCS pressure boundary intact. | 12 hours |
| K. | As required by Required Action C.1 and referenced in Table 3.3.9-1. | K.1 Suspend positive reactivity additions. | Immediately |
| | | <u>AND</u> | |
| | | K.2 Initiate action to establish water level ≥ 23 feet above the top of the reactor vessel flange. | Immediately |

Table 3.3.9-1 (page 1 of 2)
Engineered Safeguards Actuation System Instrumentation

| FUNCTION | APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS | REQUIRED CHANNELS | CONDITIONS |
|---|--|-------------------|------------|
| 1. Safeguards Actuation - Manual Initiation | 1,2,3,4 | 2 switches | E |
| | 5 | 2 switches | J |
| 2. Core Makeup Tank (CMT) Actuation - Manual Initiation | 1,2,3,4 ^(a) | 2 switches | D |
| | 4 ^(b) , 5 ^(d) | 2 switches | G |
| 3. Containment Isolation - Manual Initiation | 1,2,3,4 | 2 switches | E |
| 4. Steam Line Isolation - Manual Initiation | 1,2,3,4 | 2 switches | F |
| 5. Feedwater Isolation - Manual Initiation | 1,2,3,4 | 2 switches | F |
| 6. ADS Stages 1, 2 & 3 Actuation - Manual Initiation | 1,2,3,4 | 2 switch sets | E |
| | 5 ^(d) | 2 switch sets | H |
| 7. ADS Stage 4 Actuation - Manual Initiation | 1,2,3,4 | 2 switch sets | E |
| | 5 | 2 switch sets | H |
| | 6 ^(e) | 2 switch sets | I |
| 8. Passive Containment Cooling Actuation - Manual Initiation | 1,2,3,4 | 2 switches | E |
| | 5 ^(f) | 2 switches | J |
| | 6 ^(f) | 2 switches | K |
| 9. Passive Residual Heat Removal Heat Exchanger Actuation - Manual Initiation | 1,2,3,4 | 2 Switches | E |
| | 5 ^(c) | 2 switches | G |
| 10. Chemical and Volume Control System Makeup Isolation - Manual Initiation | 1,2,3,4 ^(a) | 2 switches | F |
| 11. Normal Residual Heat Removal System Isolation - Manual Initiation | 1,2,3 | 2 switch sets | F |

(a) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(b) With the RCS being cooled by the RNS.

(c) With the RCS pressure boundary intact.

(d) With RCS not VENTED.

(e) With upper internals in place.

(f) With decay heat > 6.0 MWt.

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|--|--|
| <p>C. As required by Required Action B.1 and referenced in Table 3.3.10-1.</p> | <p>C.1 Suspend positive reactivity additions.</p> <p><u>AND</u></p> <p>C.2 Initiate action to establish a pressurizer level above the P-12 (Pressurizer Level) interlock.</p> | <p>Immediately</p> <p>Immediately</p> |
| <p>D. As required by Required Action B.1 and referenced in Table 3.3.10-1.</p> | <p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Flow path(s) may be unisolated intermittently under administrative controls.</p> <p>-----</p> <p>D.1.1 Isolate the affected flow path(s).</p> <p><u>AND</u></p> <p>D.1.2.1 Isolate the affected flow path(s) by use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.</p> <p><u>OR</u></p> <p>D.1.2.2 Verify the affected flow path is isolated</p> <p><u>OR</u></p> <p>D.2 Initiate action to establish a pressurizer level above the P-12 (Pressurizer Level) interlock.</p> | <p>24 hours</p> <p>7 days</p> <p>Once per 7 days</p> <p>12 hours</p> |

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|-----------------|
| E. As required by Required Action B.1 and referenced in Table 3.3.10-1. | E.1 Initiate action to establish water level \geq 23 feet above the top of the reactor vessel flange. | Immediately |

Table 3.3.10-1 (page 1 of 1)
Engineered Safeguards Actuation System Instrumentation

| FUNCTION | APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS | REQUIRED CHANNELS | CONDITIONS |
|--------------------------|--|----------------------|------------|
| 1. Hot Leg Level – Low 4 | 5 ^(a) ,6 ^(b) | 1 per loop | C |
| 2. Hot Leg Level – Low 2 | 5 ^(c) | 1 per loop | D |
| | 6 ^(d) | 1 per loop | E |

(a) With CMT actuation on Pressurizer Water Level - Low 2 blocked.

(b) With upper internals in place and with CMT actuation on Pressurizer Water Level - Low 2 blocked.

(c) Below the P-12 (Pressurizer Level) interlock.

(d) With the water level < 23 feet above the top of the reactor vessel flange.

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------------------------------|
| SR 3.3.15.1 | Perform ACTUATION LOGIC TEST. | 92 days on a STAGGERED TEST BASIS |
| SR 3.3.15.2 | <p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODE 4 above the P-19 (RCS Pressure) interlock with the RCS not being cooled by RNS.</p> <p>-----</p> <p>Verify pressurizer heater circuit breakers trip open on an actual or simulated actuation signal.</p> | 24 months |
| SR 3.3.15.3 | Verify reactor coolant pump breakers trip open on an actual or simulated actuation signal. | 24 months |
| SR 3.3.15.4 | Verify main feedwater and startup feedwater pump breakers trip open on an actual or simulated actuation signal. | 24 months |
| SR 3.3.15.5 | <p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODES 1 and 2.</p> <p>-----</p> <p>Verify auxiliary spray and purification line isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p> | 24 months |

SURVEILLANCE REQUIREMENTS (continued)

| SURVEILLANCE | FREQUENCY |
|--|------------------|
| <p>SR 3.3.16.3 -----</p> <p style="text-align: center;">- NOTES -</p> <ol style="list-style-type: none"> 1. Not required to be met in MODE 5 above the P-12 (Pressurizer Level) interlock. 2. Not required to be met in MODE 6 with water level \geq 23 feet above the top of the reactor vessel flange. <p>-----</p> <p>Verify CVS letdown isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p> | <p>24 months</p> |
| <p>SR 3.3.16.4 -----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODE 6.</p> <p>-----</p> <p>Verify Spent Fuel Pool Cooling System containment isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p> | <p>24 months</p> |

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.11 Automatic Depressurization System (ADS) – Operating

LCO 3.4.11 Ten ADS flow paths shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|---|-----------------|
| A. One flow path in ADS stage 1, 2, or 3 inoperable. | A.1 Restore flow path to OPERABLE status. | 7 days |
| B. One flow path in ADS stage 4 inoperable. | B.1 Restore flow path to OPERABLE status. | 72 hours |
| C. One flow path in ADS stage 1 inoperable and one flow path in ADS stage 2 or 3 inoperable. <u>OR</u> Two flow paths in ADS stage 1 inoperable. | C.1 Restore one flow path to OPERABLE status. | 72 hours |

SURVEILLANCE REQUIREMENTS (continued)

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------|
| SR 3.4.11.4 | Verify each stage 1, 2, and 3 ADS valve actuates to the open position on an actual or simulated actuation signal. | 24 months |
| SR 3.4.11.5 | ----- - NOTE - Squib actuation may be excluded. ----- Verify continuity of the circuit from the Protection Logic Cabinets to each stage 4 ADS valve. | 24 months |

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12 Automatic Depressurization System (ADS) – Shutdown, RCS Intact

- LCO 3.4.12
- A. With reactor subcritical for < 28 hrs:
 - 1. Five flow paths in ADS stage 1, 2, and 3 shall be OPERABLE; and
 - 2. Four flow paths in ADS stage 4 shall be OPERABLE
 - B. With reactor subcritical for ≥ 28 hrs:
 - 1. Three flow paths in ADS stage 1, 2, and 3, with a minimum of two flow paths in ADS stage 2 or 3, shall be OPERABLE; and
 - 2. Three flow paths in ADS stage 4 shall be OPERABLE

APPLICABILITY: MODE 5 with RCS pressure boundary intact and pressurizer level ≥ 20%.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|--|-----------------|
| A. One required flow path in ADS stage 1, 2, or 3 inoperable. | A.1 Restore required flow path to OPERABLE status. | 7 days |
| B. One required flow path in ADS stage 4 inoperable. | B.1 Restore required flow path to OPERABLE status. | 72 hours |
| C. One required flow path in ADS stage 1 inoperable and one required flow path in ADS stage 2 or 3 inoperable. <u>OR</u> Two required flow paths in ADS stage 1 inoperable. | C.1 Restore one required flow path to OPERABLE status. | 72 hours |

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|---|--------------------|
| <p>D. Required Action and associated Completion Time of Condition A, B, or C not met.</p> <p><u>OR</u></p> <p>Condition A and Condition B entered concurrently.</p> <p><u>OR</u></p> <p>Three or more flow paths in ADS stage 1, 2, and 3 inoperable.</p> <p><u>OR</u></p> <p>LCO not met for reasons other than Condition A, B, or C.</p> | <p>D.1 Initiate action to open the RCS pressure boundary.</p> | <p>Immediately</p> |

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | FREQUENCY |
|---|--|
| <p>SR 3.4.12.1 For flow paths required to be OPERABLE, the SRs of LCO 3.4.11, "Automatic Depressurization System (ADS) – Operating" are applicable.</p> | <p>In accordance with applicable SRs</p> |

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.13 Automatic Depressurization System (ADS) – Shutdown, RCS Open

- LCO 3.4.13
- A. With reactor subcritical for < 28 hrs:
 - 1. Five flow paths in ADS stage 1, 2, and 3 shall be open; and
 - 2. Four flow paths in ADS stage 4 shall be OPERABLE.
 - B. With reactor subcritical for ≥ 28 hrs:
 - 1. Three flow paths in ADS stage 1, 2, and 3, with a minimum of two flow paths in ADS stage 2 or 3, shall be open; and
 - 2. Three flow paths in ADS stage 4 shall be OPERABLE.

- NOTE -

In MODE 5, required flow paths in ADS stage 1, 2, and 3 may be closed provided they meet OPERABILITY requirements of LCO 3.4.12, ADS – Shutdown, RCS Intact, for the following:

- a. To facilitate RCS vacuum fill operations until a pressurizer level of ≥ 20% is established; or
 - b. To facilitate LCO compliance during transitions between LCO 3.4.12 and LCO 3.4.13.
-

APPLICABILITY: MODE 5 with pressurizer level < 20%,
MODE 5 with RCS pressure boundary open,
MODE 6 with upper internals in place.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|---|--|
| <p>A. One required flow path in ADS stage 1, 2, and 3 not open.</p> | <p>A.1 Restore required flow path in ADS stage 1, 2, and 3 to open status.</p> <p><u>OR</u></p> <p>A.2 Open alternative flow path(s) with an equivalent area.</p> | <p>72 hours</p> <p>72 hours</p> |
| <p>B. One required flow path in ADS stage 4 inoperable.</p> | <p>B.1 Restore required flow path in ADS stage 4 to OPERABLE status.</p> <p><u>OR</u></p> <p>B.2 Open an alternative flow path with an equivalent area.</p> | <p>36 hours</p> <p>36 hours</p> |
| <p>C. Required Action and associated Completion Time of Condition A or B not met in MODE 5.</p> <p><u>OR</u></p> <p>Condition A and Condition B entered concurrently in MODE 5.</p> <p><u>OR</u></p> <p>LCO not met for reasons other than Condition A or B in MODE 5.</p> | <p>C.1 Initiate action to fill the RCS to establish $\geq 20\%$ pressurizer level.</p> <p><u>AND</u></p> <p>C.2 Suspend positive reactivity additions.</p> <p><u>AND</u></p> <p>C.3 Initiate action to establish RCS VENTED condition.</p> | <p>Immediately</p> <p>Immediately</p> <p>Immediately</p> |

ACTIONS (continued)

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|--|-----------------|
| D. Required Action and associated Completion Time of Condition A or B not met in MODE 6. <u>OR</u> Condition A and Condition B entered concurrently in MODE 6. <u>OR</u> LCO not met for reasons other than Condition A or B in MODE 6. | D.1 Initiate action to remove the upper internals. | Immediately |
| | <u>AND</u> D.2 Suspend positive reactivity additions. | Immediately |

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | FREQUENCY |
|--|-----------------------------------|
| SR 3.4.13.1 Verify each required ADS stage 1, 2, and 3 valve is in the open position. | 12 hours |
| SR 3.4.13.2 For each required flow path in ADS stage 4, the following SRs are applicable: SR 3.4.11.1 SR 3.4.11.3 SR 3.4.11.5 | In accordance with applicable SRs |

3.5 PASSIVE CORE COOLING SYSTEM (PXS)

3.5.3 Core Makeup Tanks (CMTs) – Shutdown, Reactor Coolant System (RCS) Intact

LCO 3.5.3 One CMT shall be OPERABLE.

APPLICABILITY: MODE 4 with the RCS cooling provided by the Normal Residual Heat Removal System (RNS).
MODE 5 with the RCS not VENTED.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|-----------------|
| A. Required CMT inoperable due to one outlet isolation valve inoperable. | A.1 Restore required isolation valve to OPERABLE status. | 72 hours |
| B. Required CMT inoperable due to water temperature or boron concentration not within limits. | B.1 Restore water temperature and boron concentration to within limits. | 72 hours |
| C. Required CMT inlet line noncondensable gas volume not within limit in MODE 4. | C.1 Restore CMT inlet line noncondensable gas volume to within limit. | 24 hours |
| | <u>OR</u> C.2 Be in MODE 5. | 24 hours |
| D. Required CMT inoperable for reasons other than Condition A, B, or C. | D.1 Restore required CMT to OPERABLE status. | 8 hours |
| E. Required Action and associated Completion Time of Condition A, B, or D not met. | E.1 Initiate action to be in MODE 5 with RCS VENTED. | Immediately |

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | | FREQUENCY |
|--------------|---|-----------------------------------|
| SR 3.5.3.1 | <p>For the CMT required to be OPERABLE, the following SRs are applicable:</p> <p>SR 3.5.2.1</p> <p>SR 3.5.2.2</p> <p>SR 3.5.2.3</p> <p>SR 3.5.2.5</p> <p>SR 3.5.2.6</p> <p>SR 3.5.2.7</p> <p>SR 3.5.2.8</p> | In accordance with applicable SRs |
| SR 3.5.3.2 | <p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODE 4 with RCS cooling provided by the Normal Residual Heat Removal System (RNS).</p> <p>-----</p> <p>For the CMT required to be OPERABLE, the following SR is applicable:</p> <p>SR 3.5.2.4</p> | In accordance with applicable SR |

1. The physical arrangement of the nuclear island structures, the annex building, and the turbine building is as described in the Design Description of this Section 3.3, and as shown on Figures 3.3-1 through 3.3-14. The physical arrangement of the radwaste building and the diesel generator building is as described in the Design Description of this Section 3.3.
2.
 - a) The nuclear island structures, including the critical sections listed in Table 3.3-7, are seismic Category I and are designed and constructed to withstand design basis loads, as specified in the Design Description, without loss of structural integrity and the safety-related functions. The design bases loads are those loads associated with:
 - Normal plant operation (including dead loads, live loads, lateral earth pressure loads, and equipment loads, including hydrodynamic loads, temperature and equipment vibration);
 - External events (including rain, snow, flood, tornado, tornado generated missiles and earthquake); and
 - Internal events (including flood, pipe rupture, equipment failure, and equipment failure generated missiles).
 - b) Site grade level is located relative to floor elevation 100'-0" per Table 3.3-5. Floor elevation 100'-0" is defined as the elevation of the floor at design plant grade.
 - c) The containment and its penetrations are designed and constructed to ASME Code Section III, Class MC.⁽¹⁾
 - d) The containment and its penetrations retain their pressure boundary integrity associated with the design pressure.
 - e) The containment and its penetrations maintain the containment leakage rate less than the maximum allowable leakage rate associated with the peak containment pressure for the design basis accident.
 - f) The key dimensions of the nuclear island structures are as defined on Table 3.3-5.
 - g) The containment vessel greater than 7 feet above the operating deck provides a heat transfer surface. A free volume exists inside the containment shell above the operating deck.
 - h) The containment free volume below elevation 107.68' provides containment floodup during a postulated loss-of-coolant accident.
3. Walls and floors of the nuclear island structures as defined on Table 3.3-1, except for designed openings and penetrations, provide shielding during normal operations.
4.
 - a) Walls and floors of the annex building as defined on Table 3.3-1, except for designed openings and penetrations, provide shielding during normal operations.
 - b) The walls on the outside of the waste accumulation room in the radwaste building provide shielding from accumulated waste.

1. Containment isolation devices are addressed in subsection 2.2.1, Containment System.

Table 3.3-6
Inspections, Tests, Analyses, and Acceptance Criteria

| No. | ITAAC No. | Design Commitment | Inspections, Tests, Analyses | Acceptance Criteria |
|-----|------------|--|--|---|
| 772 | 3.3.00.02d | Not used per Amendment No. 84 | | |
| 773 | 3.3.00.02e | Not used per Amendment No. 84 | | |
| 774 | 3.3.00.02f | 2.f) The key dimensions of nuclear island structures are defined on Table 3.3-5. | An inspection will be performed of the as-built configuration of the nuclear island structures. | A report exists and concludes that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on Table 3.3-5. |
| 775 | 3.3.00.02g | 2.g) The containment vessel greater than 7 feet above the operating deck provides a heat transfer surface. A free volume exists inside the containment shell above the operating deck. | The maximum containment vessel inside height from the operating deck is measured and the inner radius below the spring line is measured at two orthogonal radial directions at one elevation. | The containment vessel maximum inside height from the operating deck is 146'-7" (with tolerance of +12", -6"), and the inside diameter is 130 feet nominal (with tolerance of +12", -6"). |
| 776 | 3.3.00.02h | 2.h) The free volume in the containment allows for floodup to support long-term core cooling for postulated loss-of-coolant accidents. | An inspection will be performed of the as-built containment structures and equipment. The portions of the containment included in this inspection are the volumes that flood with a loss-of-coolant accident in passive core cooling system valve/equipment room B (11207). The in-containment refueling water storage tank volume is excluded from this inspection. | A report exists and concludes that the floodup volume of this portion of the containment is less than 71,960 ft ³ to an elevation of 107.68'. |