

## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 9, 1995

Mr. Kurt M. Haas Plant Safety and Licensing Director Palisades Plant 27780 Blue Star Memorial Highway Covert, MI 49043

SUBJECT: PALISADES PLANT - JUSTIFICATION FOR CONTINUED OPERATION - POTENTIAL RADIOACTIVE RELEASE PATH (TAC NO. M80402)

Dear Mr. Haas:

By submittal dated January 10, 1992, as supplemented by submittals dated April 21 and July 28, 1992, Consumers Power Company (CPCo) requested our permission for continued operation of the Palisades Plant until the beginning of Fuel Cycle 12 and provided a justification for continued operation (JCO). In the JCO submittals, it was pointed out that the discovery of the existence of a potential leak path for radioactive containment sump water to the safety injection and refueling water (SIRW) tank during the recirculation phase of a postulated loss-of-coolant accident (LOCA) raised an unreviewed safety issue. This is because the tank is vented directly to the atmosphere unmonitored via a 6-inch line, and CPCo had not previously considered the tank vent release as a source term when calculating its control room operator and offsite doses. In the January 10 and April 21, 1992, submittals, CPCo committed to provide by April 30, 1992, a new design basis LOCA analysis (maximum hypothetical accident (MHA)) to account for the additional dose contributions to control room and offsite doses due to SIRW tank vent release and identify the modifications needed to support the analysis. CPCo further added that it would complete all the modifications by the beginning of Fuel Cycle 12.

In the April 21, 1992, submittal, CPCo provided the proposed compensatory measures including limited administrative controls and the interim calculated doses with the associated assumptions. The submittal additionally included responses to our questions based on the review of the January 10, 1992, submittal. In the submittal dated July 28, 1992, CPCo revised the source term assumptions to be consistent with Regulatory Guide 1.4 guidelines and recalculated the interim control room doses and offsite doses for the design basis LOCA.

We have completed our review of the above submittals and enclosed is our safety evaluation. We find the responses to our questions relating to continued plant operation, the compensatory measures, and the assumptions used in the interim dose analyses to be acceptable. Further, we find the calculated interim control room operator doses and offsite doses are within

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the applicable General Design Criterion 19 and 10 CFR Part 100 limits. Based on the above, we conclude that the JCO for Palisades is acceptable for continued operation until completion of the final MHA analysis during Fuel Cycle 12, but no later than January 1996.

## Sincerely,

ORIGINAL SIGNED BY

Marsha Gamberoni, Project Manager Project Directorate III-1 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Docket No. 50-255

Enclosure: Safety Evaluation

cc w/encl: See next page

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cc: Plant Service list

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