

ATTACHMENT TO LICENSE AMENDMENT NO. 118

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

REMOVE

7

INSERT

7

Appendix A to Facility Combined License Nos. NPF-91 and NPF-92

REMOVE

1.1-1

3.3.15-2

3.3.16-3

3.3.16-4

INSERT

1.1-1

3.3.15-2

3.3.16-3

3.3.16-4

(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in FSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 118, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

1.0 USE AND APPLICATION

1.1 Definitions

- NOTE -

The defined terms of this section appear in capitalized type and are applicable throughout these Technical Specifications and Bases.

<u>Term</u>	<u>Definition</u>
ACTIONS	ACTIONS shall be that part of a Specification that prescribes Required Actions to be taken under designated Conditions within specified Completion Times.
ACTUATION LOGIC TEST	An ACTUATION LOGIC TEST shall be the application of various simulated or actual input combinations in conjunction with each possible interlock logic state required for OPERABILITY of a logic circuit and the verification of the required logic output. The ACTUATION LOGIC TEST may be performed by means of any series of sequential, overlapping, or total steps.
ACTUATION LOGIC OUTPUT TEST	An ACTUATION LOGIC OUTPUT TEST shall be the application of simulated or actual logic signals and the verification of the required component actuation output signals up to, but not including, the actuated device. The ACTUATION LOGIC OUTPUT TEST may be performed by means of any series of sequential, overlapping, or total steps.
AXIAL FLUX DIFFERENCE (AFD)	AFD shall be the difference in normalized flux signals between the top and bottom halves of a two-section excore neutron detector.
CHANNEL CALIBRATION	<p>A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds within the necessary range and accuracy to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass all devices in the channel required for OPERABILITY.</p> <p>Calibration of instrument channels with resistance temperature detector (RTD) or thermocouple sensors may consist of an in-place qualitative assessment of sensor behavior and normal calibration of the remaining adjustable devices in the channel. The CHANNEL CALIBRATION may be performed by means of any series of sequential, overlapping, or total channel steps.</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.15.1	Perform ACTUATION LOGIC TEST on ESF Coincidence Logic.	92 days on a STAGGERED TEST BASIS
SR 3.3.15.2	Perform ACTUATION LOGIC OUTPUT TEST on ESF Actuation.	24 months
SR 3.3.15.3	<p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODE 4 above the P-19 (RCS Pressure) interlock with the RCS not being cooled by RNS.</p> <p>-----</p> <p>Verify pressurizer heater circuit breakers trip open on an actual or simulated actuation signal.</p>	24 months
SR 3.3.15.4	Verify reactor coolant pump breakers trip open on an actual or simulated actuation signal.	24 months
SR 3.3.15.5	Verify main feedwater and startup feedwater pump breakers trip open on an actual or simulated actuation signal.	24 months
SR 3.3.15.6	<p>-----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODES 1 and 2.</p> <p>-----</p> <p>Verify auxiliary spray and purification line isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p>	24 months

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>D. Required Action and associated Completion Time of Condition A not met during movement of irradiated fuel assemblies.</p> <p><u>OR</u></p> <p>One or more Functions within two or more required divisions inoperable during movement of irradiated fuel assemblies.</p>	<p>D.1 Suspend movement of irradiated fuel assemblies.</p>	<p>Immediately</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.3.16.1 Perform ACTUATION LOGIC TEST on ESF Coincidence Logic.</p>	<p>92 days on a STAGGERED TEST BASIS</p>
<p>SR 3.3.16.2 Perform ACTUATION LOGIC OUTPUT TEST on ESF Actuation.</p>	<p>24 months</p>
<p>SR 3.3.16.3 -----</p> <p style="text-align: center;">- NOTE -</p> <p style="text-align: center;">Only required to be met in MODE 5.</p> <p style="text-align: center;">-----</p> <p>Verify reactor coolant pump breakers trip open on an actual or simulated actuation signal.</p>	<p>24 months</p>

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.16.4 -----</p> <p style="text-align: center;">- NOTES -</p> <ol style="list-style-type: none"> 1. Not required to be met in MODE 5 above the P-12 (Pressurizer Level) interlock. 2. Not required to be met in MODE 6 with water level \geq 23 feet above the top of the reactor vessel flange. <p>-----</p> <p>Verify CVS letdown isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p>	<p>24 months</p>
<p>SR 3.3.16.5 -----</p> <p style="text-align: center;">- NOTE -</p> <p>Only required to be met in MODE 6.</p> <p>-----</p> <p>Verify Spent Fuel Pool Cooling System containment isolation valves actuate to the isolation position on an actual or simulated actuation signal.</p>	<p>24 months</p>