



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 16, 2018

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT 2 – ISSUANCE OF
AMENDMENT NO. 167 TO REVISE TECHNICAL SPECIFICATION 2.1.1.2
SAFETY LIMIT MINIMUM CRITICAL POWER RATIO (EPID L-2017-LLA-0373)

Dear Mr. Hanson:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 167 to Renewed Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station, Unit 2 (Nine Mile Point 2). The amendment consists of changes to the Nine Mile Point 2 Technical Specifications in response to your application dated November 3, 2017 (Agencywide Documents Access and Management System Accession No. ML17307A019).

The amendment changes the safety limit minimum critical power ratio numeric values for Operating Cycle 17 in the Nine Mile Point 2 Technical Specifications. Specifically, the change increases the numeric values of the safety limit minimum critical power ratio for Nine Mile Point 2 from ≥ 1.15 to ≥ 1.17 for two recirculation loop operation, and from ≥ 1.15 to ≥ 1.17 for single recirculation loop operation.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "Michael L. Marshall, Jr." with a stylized flourish at the end.

Michael L. Marshall, Jr., Senior Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosures:

1. Amendment No. 167 to NPF-69
2. Safety Evaluation

cc w/enclosures: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NINE MILE POINT NUCLEAR STATION, LLC

LONG ISLAND LIGHTING COMPANY

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 167
Renewed License No. NPF-69

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (Exelon, the licensee) dated November 3, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

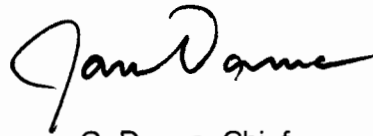
1. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 167, are hereby incorporated into this license. Exelon Generation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

2. This license amendment is effective as of the date of its issuance and shall be implemented prior to startup from the next refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION



James G. Danna, Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License No. NPF-69
and Technical Specifications

Date of Issuance: March 16, 2018

ATTACHMENT TO LICENSE AMENDMENT NO. 167
NINE MILE POINT NUCLEAR STATION, UNIT 2
RENEWED FACILITY OPERATING LICENSE NO. NPF-69
DOCKET NO. 50-410

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page
4

Insert Page
4

Replace the following page of Appendix A, Technical Specifications, with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Pages
2.0-1

Insert Pages
2.0-1

(1) Maximum Power Level

Exelon Generation is authorized to operate the facility at reactor core power levels not in excess of 3988 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 167, are hereby incorporated into this license. Exelon Generation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Fuel Storage and Handling (Section 9.1.SSER 4)*

- a. Fuel assemblies, when stored in their shipping containers, shall be stacked no more than three containers high.
- b. When not in the reactor vessel, no more than three fuel assemblies shall be allowed outside of their shipping containers or storage racks in the New Fuel Vault or Spent Fuel Storage Facility.
- c. The above three fuel assemblies shall maintain a minimum edge-to-edge spacing of twelve (12) inches from the shipping container array and approved storage rack locations.
- d. The New Fuel Storage Vault shall have no more than ten fresh fuel assemblies uncovered at any one time.

(4) Turbine System Maintenance Program (Section 3.5.1.3.10 SER)

The operating licensee shall submit for NRC approval by October 31, 1989, a turbine system maintenance program based on the manufacturer's calculations of missile generation probabilities. (Submitted by NMPC letter dated October 30, 1989 from C.D. Terry and approved by NRC letter dated March 16, 1990 from Robert Martin to Mr. Lawrence Burkhardt, III).

* The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report (SER) and/or its supplements wherein the license condition is discussed.

2.0 SAFETY LIMITS (SLs)

2.1 SLs

2.1.1 Reactor Core SLs

2.1.1.1 With the reactor steam dome pressure < 700 psia or core flow < 10% rated core flow:

THERMAL POWER shall be $\leq 23\%$ RTP.

2.1.1.2 With the reactor steam dome pressure ≥ 700 psia and core flow $\geq 10\%$ rated core flow:

MCPR shall be ≥ 1.17 for two recirculation loop operation or ≥ 1.17 for single recirculation loop operation.

2.1.1.3 Reactor vessel water level shall be greater than the top of active irradiated fuel.

2.1.2 Reactor Coolant System Pressure SL

Reactor steam dome pressure shall be ≤ 1325 psig.

2.2 SL Violations

With any SL violation, the following actions shall be completed within 2 hours:

2.2.1 Restore compliance with all SLs; and

2.2.2 Insert all insertable control rods.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 167

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-69

NINE MILE POINT NUCLEAR STATION, LLC

LONG ISLAND LIGHTING COMPANY

EXELON GENERATION COMPANY, LLC

NINE MILE POINT NUCLEAR STATION, UNIT 2

DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated November 3, 2017 (Reference 1), Exelon Generation Company, LLC (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit 2 (Nine Mile Point 2), Technical Specifications (TSs). The requested changes would change the safety limit minimum critical power ratio (SLMCPR) numeric values for Operating Cycle 17. Specifically, the changes would increase the numeric values of SLMCPR from ≥ 1.15 to ≥ 1.17 for two recirculation loop operation (TLO), and from ≥ 1.15 to ≥ 1.17 for single recirculation loop operation (SLO). The SLMCPR is contained in Section 2.1.1.2 of the Nine Mile Point 2 TSs and it can vary from cycle to cycle.

2.0 REGULATORY EVALUATION

2.1 Proposed Changes

Based on the analysis that was attached to the letter dated November 3, 2017, which used U.S. Nuclear Regulatory Commission (NRC)-approved methodologies, the licensee has proposed to amend the Nine Mile Point 2 TSs to revise the SLMCPR for Operating Cycle 17. Specifically, the licensee requested to change the calculated SLMCPR in Section 2.1.1.2 of the Nine Mile Point 2 TSs from ≥ 1.15 to ≥ 1.17 for TLO and from ≥ 1.15 to ≥ 1.17 for SLO.

2.2 Regulations and Guidance

The regulatory requirements and guidance documents that the NRC staff considered in its review of the proposed amendment included the following:

- The NRC's regulatory requirements related to the content of the TSs are contained in Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR), which requires that TSs include items in the following categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in the TSs.

The regulations in 10 CFR 50.36(c)(1)(i)(A) state, in part, that:

Safety limits for nuclear reactors are limits upon important process variables that are found to be necessary to reasonably protect the integrity of certain of the physical barriers that guard against the uncontrolled release of radioactivity.

The fuel cladding is one of the physical barriers that separate the radioactive materials from the environment. The SLMCPR is a safety limit that is required to be in the TSs to ensure that fuel design limits are not exceeded.

- General Design Criterion (GDC) 10, "Reactor design," of Appendix A to 10 CFR Part 50 states that the reactor core and associated coolant, control, and protection systems shall be designed with appropriate margin to assure that specified acceptable fuel design limits (SAFDLs) are not exceeded. The purpose of the SLMCPR is to ensure that the SAFDLs are not exceeded, during steady state operation and analyzed transients.
- Guidance on the acceptability of the reactivity control systems, the reactor core, and fuel system design is provided in Section 4.2, "Fuel System Design," of the Standard Review Plan (SRP) (Reference 2). Specifically, Section 4.2 specifies all fuel damage criteria for evaluation to determine whether fuel designs meet the SAFDLs. In SRP Section 4.4, "Thermal and Hydraulic Design" (Reference 3), guidance is provided on the review of thermal-hydraulic design in meeting the requirement of GDC 10 and the fuel design criteria established in SRP Section 4.2. Section 4.4 states that an acceptable approach is establishing the critical power ratio such that at least 99.9 percent of fuel rods in the core would not experience a departure from nucleate boiling or boiling transition during normal operation or anticipated operational occurrences.

3.0 TECHNICAL EVALUATION

The SLMCPR numeric values in Section 2.1.1.2 of the Nine Mile Point 2 TSs are safety limits. The SLMCPR limit is established such that at least 99.9 percent of fuel rods in the core would not experience the onset of transition boiling as a result of normal operation and transients, which in turn ensures that fuel cladding damage would not occur. The SLMCPR limit is established such that fuel design limits are not exceeded during steady state operation, normal operational transients, and abnormal operational transients. As such, fuel damage is calculated not to occur if the limit is not violated. However, because fuel damage is not directly observable, a step-back approach is used to establish corresponding operating limits. The

operating limit minimum critical power ratio (OLMCPR) is established by summing the cycle-specific core reload transient analyses adders and the calculated SLMCPR values. The OLMCPR is required to be established and documented in the core operating limits report (COLR) for each reload cycle by Nine Mile Point 2 TS 5.6.5, "Core Operating Limits Report (COLR)." The absolute value of the SLMCPR tends to vary cycle-to-cycle, typically due to the introduction of improved fuel bundle types, changes in fuel vendors or applicable computer codes, and changes in core loading pattern. Following the determination of the cycle-specific SLMCPR values, the OLMCPR values are derived.

The Nine Mile Point 2 Cycle 17 SLMCPR analysis attached to the letter dated November 3, 2017 is consistent with NRC-approved methodologies and uncertainties, as documented in References 4, 5, 6, 10, 11, and 12. These methodologies were used for the Nine Mile Point 2 Cycle 16 and Cycle 17 SLMCPR calculations. The NRC staff reviewed the proposed changes to ensure that the generic methods were appropriately applied to the Nine Mile Point 2 Cycle 17 SLMCPR analysis. The Nine Mile Point 2 Cycle 17 core consists of 616 GNF2 and 148 GE14 legacy fuel assemblies. No plant hardware or operational changes are required with this proposed change. The NEDC-32505P-A report (Reference 4) is the generic R-Factor methodology report that describes the changed methodology that was adopted after part length rods were introduced. The NRC staff's safety evaluation for NEDC-32505P-A has a restriction that the applicability of the R-Factor methodology is confirmed when a new fuel type is introduced. The FLN-2007-011 report (Reference 5) confirmed that the R-factor methodology of NEDC-32505P-A is applicable to GNF2, and that all of the criteria defined in NEDE-24011-P-A (Reference 6), have been met for the GNF2 fuel design.

Comparing Nine Mile Point 2 Cycle 17 to Cycle 16, the Cycle 17 core has a much flatter core bundle-by-bundle minimum critical power ratio (MCPR) distribution. The bundle pin-by-pin power/R-Factor distribution for the Cycle 17 core is not much different than for the Cycle 16 core. Greater flatness in either parameter yields more rods susceptible to boiling transition and thus a higher calculated SLMCPR. Therefore, the calculated SLMCPR may change whenever there are changes to the core configuration or to the fresh fuel designs. The plant-cycle specific SLMCPR methodology accounts for these factors. Another factor besides core MCPR distribution or bundle R-factor distribution that significantly impacts the SLMCPR is the expansion of the analysis domain that comes with the application of Maximum Extended Load Line Limit Plus (MELLLA+) (References 7 and 8). The rated power/minimum core flow point is analyzed at a lower core flow (than without MELLLA+) using increased uncertainties (see Section 2.2.1.1 of Reference 9) that tend to increase the SLMCPR. The combination of a combined higher uncertainty and the application of MELLLA+ is sufficient to explain the increase in SLMCPR.

Consistent with NEDC-33173P-A, Revision 4 (Reference 10), a 0.02 SLMCPR penalty was added for operation in the MELLLA+ region. This information regarding requested changes to the Nine Mile Point 2 TS SLMCPR is based on and is for the core rated power of 3,988 megawatt thermal, and at minimum core flow of 85 percent at rated power.

The current required SLMCPR values in the Nine Mile Point 2 TS are ≥ 1.15 for TLO and ≥ 1.15 for SLO. The Nine Mile Point 2 Cycle 17 SLMCPR analysis resulted in minimum calculated SLMCPR values of ≥ 1.17 for TLO and ≥ 1.17 for SLO. For Cycle 17, the minimum core flow SLMCPR calculation performed at 85 percent core flow and rated core power condition was limiting as compared to the rated core flow and rated core power condition. The analysis of the revised plant-specific SLMCPR numeric values for Nine Mile Point 2 Cycle 17 was performed as part of the reload licensing analysis for Nine Mile Point 2 Cycle 17, and is based upon

NRC-approved methods. The NRC staff verified that no departures from NRC-approved methodologies, or deviations from NRC-approved calculational uncertainties, were identified in the Nine Mile Point 2, Cycle 17, SLMCPR calculations. All calculated uncertainties for Nine Mile Point 2, Cycle 17, SLMCPR calculations were conservative relative to NRC-approved values.

The NRC staff further verified that the proposed changes would continue to meet the applicable regulations and requirements, and that the analysis performed to calculate the Nine Mile Point 2 Cycle 17 SLMCPR numeric values was based upon NRC-approved methodologies. The NRC staff concludes that the SLMCPR will continue to provide assurance that at least 99.9 percent of the fuel rods in the core will not experience the onset of transition boiling, and that fuel cladding integrity will be maintained under conditions of normal operation and with appropriate margin for anticipated operational occurrences.

The NRC staff finds that the licensee's proposed amendment to update the TSs to include cycle-specific SLMCPR numeric values is based on NRC-approved methodologies that have been approved for use with GNF2 and GE14 fuels. The proposed amendment is consistent with the regulatory requirements and guidance as discussed in Section 2.0 of this safety evaluation. The NRC staff determined that the proposed changes do not require any exemptions or relief from regulatory requirements. Defense-in-depth and sufficient safety margins will continue to be maintained. Based on the above considerations, the proposed changes are acceptable because the Cycle 17 SLMCPR values will ensure that the SAFDLs are not exceeded during steady state operation and analyzed transients.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION

The NRC's regulations in 10 CFR 50.92 state that the NRC may make a final determination that a license amendment involves no significant hazards consideration if operation of the facility in accordance with the amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

As required by 10 CFR 50.91(a), the licensee, in its submittal, provided its analysis of the issue of no significant hazards consideration, which is presented below.

1. Does the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The derivation of the cycle specific Safety Limit Minimum Critical Power Ratios (SLMCPRs) for incorporation into the Technical Specifications (TS), and their use to determine cycle specific thermal limits, has been performed using the methodology discussed in NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel," Revision 25.

The basis of the SLMCPR calculation is to ensure that during normal operation and during abnormal operational transients, at least 99.9% of all fuel rods in the core do not experience transition boiling if the limit is not violated. The new SLMCPRs preserve the existing margin to transition boiling.

The MCPR safety limit is reevaluated for each reload using NRC-approved methodologies. The analyses for NMP [Nine Mile Point 2], Cycle 17, have concluded that a two-recirculation loop MCPR safety limit of ≥ 1.17 , based on the application of Global Nuclear Fuel's NRC-approved MCPR safety limit methodology, will ensure that this acceptance criterion is met. For single recirculation loop operation, a MCPR safety limit of ≥ 1.17 also ensures that this acceptance criterion is met. The MCPR operating limits are presented and controlled in accordance with the NMP2 Core Operating Limits Report (COLR).

The requested TS changes do not involve any plant modifications or operational changes that could affect system reliability or performance or that could affect the probability of operator error. The requested changes do not affect any postulated accident precursors, do not affect any accident mitigating systems, and do not introduce any new accident initiation mechanisms. Therefore, the proposed TS changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The SLMCPR is a TS numerical value, calculated to ensure that during normal operation and during abnormal operational transients, at least 99.9% of all fuel rods in the core do not experience transition boiling if the limit is not violated. The new SLMCPRs are calculated using NRC-approved methodology discussed in NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel," Revision 25. The proposed changes do not involve any new modes of operation, any changes to setpoints, or any plant modifications. The proposed revised MCPR safety limits have been shown to be acceptable for Cycle 17 operation. The core operating limits will continue to be developed using NRC-approved methods. The proposed MCPR safety limits or methods for establishing the core operating limits do not result in the creation of any new precursors to an accident. Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed amendment involve a significant reduction in a margin of safety?

Response: No.

There is no significant reduction in the margin of safety previously approved by the NRC as a result of the proposed change to the SLMCPRs. The new SLMCPRs are calculated using methodology discussed in NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel," Revision 25. The SLMCPRs ensure that during normal

operation and during abnormal operational transients, at least 99.9% of all fuel rods in the core do not experience transition boiling if the limit is not violated, thereby preserving the fuel cladding integrity. Therefore, the proposed TS changes do not involve a significant reduction in the margin of safety previously approved by the NRC.

The NRC staff has reviewed the licensee's analysis and, based on this review, concludes that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff has made a final determination that no significant hazards consideration is involved for the proposed amendment and that the amendment should be issued as allowed by the criteria contained in 10 CFR 50.91.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment on February 2, 2018. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding dated February 6, 2018 (83 FR 5280). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

8.0 REFERENCES

- (1) James Barstow, Exelon Generation Company, LLC, letter to U.S. Nuclear Regulatory Commission, November 3, 2017 (ADAMS Accession No. ML17307A019).
- (2) U.S. Nuclear Regulatory Commission, "Standard Review Plan (SRP) for the Review of Safety Analysis Reports for Nuclear Power Plants," NUREG-0800, Chapter 4, Section 4.2, Revision 3, "Fuel System Design," March 2007 (ADAMS Accession No. ML070740002).

- (3) U.S. Nuclear Regulatory Commission, "Standard Review Plan (SRP) for the Review of Safety Analysis Reports for Nuclear Power Plants," NUREG-0800, Chapter 4, Section 4.4, Revision 2, "Thermal and Hydraulic Design," March 2007 (ADAMS Accession No. ML070550060).
- (4) GE Nuclear Energy, "R-Factor Calculation Method for GE11, GE12 and GE13 Fuel," NEDC-32505-A, Revision 1, July 1999 (ADAMS Accession No. ML060520636).
- (5) Global Nuclear Fuel – Americas, LLC, "GNF2 Advantage Generic Compliance with NEDE-24011-P-A (GESTAR II)," NEDC-33270P (FLN-2007-011), March 2007 (ADAMS Accession No. ML070780337).
- (6) Global Nuclear Fuel – Americas, LLC, "General Electric Standard Application for Reactor Fuel (GESTAR II, Main)," NEDE-24011-P-A, Revision 25, August 2017 (Proprietary; not publicly available).
- (7) Bhalchandra Vaidya, U.S. Nuclear Regulatory Commission, letter to Peter Orphanos, Exelon Generation Company, LLC, September 2, 2015 (ADAMS Accession No. ML15096A076).
- (8) GE Hitachi Nuclear Energy, "Maximum Extended Load Line Limit Analysis Plus," NEDC-33006P-A, Revision 3, June 2009 (Proprietary; not publicly available).
- (9) U.S. Nuclear Regulatory Commission, "Safety Evaluation by the Office of Nuclear Reactor Regulation [of] Licensing Topical Report NEDC-33006, 'General Electric Boiling Water Reactor Maximum Extended Load Line Limit Analysis Plus,'" October 2008 (ADAMS Accession No. ML081130008).
- (10) GE Hitachi Nuclear Energy, "Applicability of GE Methods to Expanded Operating Domains," NEDC-33173P-A, Revision 4, November 2012 (Proprietary; not publicly available).
- (11) GE Nuclear Energy, "Methodology and Uncertainties for Safety Limit MCPR Evaluations," NEDC-32601-A, Revision 0, August 1999 (ADAMS Accession No. ML14093A216).
- (12) GE Nuclear Energy, "Power Distribution Uncertainties for Safety Limit MCPR Evaluations," NEDC-32694P-A, Revision 0, August 1999 (Proprietary; not publicly available).

Principal Contributor: Muhammad M. Razzaque

Date: March 16, 2018.

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT 2 - ISSUANCE OF
AMENDMENT NO. 167 TO REVISE TECHNICAL SPECIFICATION 2.1.1.2
SAFETY LIMIT MINIMUM CRITICAL POWER RATIO (EPID L-2017-LLA-0373)
DATED MARCH 16, 2018

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