



Palisades Nuclear Plant: 27780 Blue Star Memorial Highway, Covert, MI 49043

G B Slade
General Manager

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Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

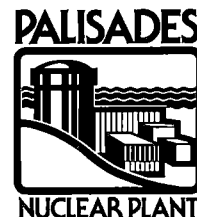
DOCKET 50-255 - LICENSE DPR-20 - PALISADES PLANT - TECHNICAL SPECIFICATIONS
CHANGE REQUEST - INSTRUMENTATION AND CONTROL - REVISED PAGES.

A complete set of revised pages for our November 15, 1991 Technical Specifications change request, "Instrumentation and Control" are attached. The changes made by the revised pages are the result of suggestions and comments made by NRR reviewers, Doris Liao and Iqbal Ahmed. Answers to questions posed by Mr. Ahmed, during his review, have been attached. Revised basis pages have also been attached. In addition to updating the changes proposed in our November 15, 1991 submittal, four other changes are proposed:

- 1) Proposed page 4-6 has been updated to reflect changes proposed for the Palisades Technical Specifications (TS) in response to Generic Letter (GL) 90-06. Parts 4.1a, and 4.1b of our November 15, 1991 change request have been replaced by parts 4.1.1.1 through 4.1.1.4 of the February 2, 1994 submittal for GL 90-06. The technical content is nearly identical, and the GL 90-06 TS wording is the result of detailed discussions with the NRC staff (refer to our June 25, 1993 letter on Technical Specifications for Overpressure Protection).
- 2) A revised page 3-1, containing a proposed section 3.0.5, and a new page 3-1ac have been included in the revised pages. The proposed Specification 3.0.5 and its Basis are taken from NUREG 1432, Standard Technical Specifications, Combustion Engineering Plants. This new specification is necessary to allow testing and repair of equipment taken out of service in accordance with requirements of the proposed action statements, the same reason for which it was added to the new standard technical specifications.

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- 3) A revised page 2-2, containing the Reactor Protective System trip settings, and an associated revised Basis page B 2-5, have been included. These pages propose changing the existing Low Steam Generator Level reactor trip setting. This change is to co-ordinate the settings for reactor trip (page 2-2) and for Auxiliary Feedwater (AFW) initiation (page 3-63). Both signals are initiated by the same instrument channels, and both signals have the same minimum setting. The setpoint for both signals are proposed as " $\geq 25.9\%$ " rather than the current reactor trip setting of "Above the feedwater ring center line." The AFW pump start setpoint is not currently specified in the Technical Specifications. As discussed in the supplied Basis page, the proposed 25.9% indicated level corresponds to the physical position of the feedwater ring center line. This change is proposed because, typically, TS limits are specified in the units indicated on control room monitors.
- 4) The explicit requirements for operability and surveillance of the Safety Injection Tank level and pressure instrument channels, items 3 and 4 of Tables 3.17.6 and 4.17.6 of the November 15, 1991 proposed pages, have been deleted in accordance with the suggestions of GL 93-05, item 7.4.

The Description of Changes section of our November 15, 1991 letter describe how the new, re-numbered pages fit into the Technical Specifications. Attachment 4 contains suggested instructions for page replacement.

Attachment 1 contains the revised proposed Technical Specifications pages.

Attachment 2 contains the Bases for the proposed Technical Specifications.

Attachment 3 contains the answers to questions raised during Mr. Ahmed's review.

Attachment 4 contains suggested instructions for page replacement.

The changes proposed in this letter do not alter the conclusions of the No Significant Hazards Analysis contained in our November 15, 1991 letter on this subject. A copy of this letter has been sent to the State of Michigan.



For Gerald B Slade
General Manager

CC Administrator, Region III, USNRC
Resident Inspector, Palisades

Attachments