

Vogle PEmails

From: Patel, Chandu
Sent: Friday, February 23, 2018 12:00 PM
To: Vogle PEmails
Subject: Final RAI for Alt- 7 for Vogle 3and 4
Attachments: RAI_9393 Final 22318.docx

Hi,

These are final RAI for alternate 7 (ALT 7) for Vogle 3 and 4 as we discussed in a meeting held on February 15, 2018. Please respond to these RAI as soon as possible to support our review schedule.

Sincerely,

Chandu Patel, Senior Project Manager
U.S. NRC, Office of New Reactors
NRC/NRO/DNRL/LB4,
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Request for Additional Information 1

Issue Date: 02/23/2018

Application Title: VEGP Units 3 and 4 - LARs

Operating Company: Southern Nuclear Operating Co.

Docket No. 52-025 and 52-026

Review Section: NONE - NO SRP SECTION

Application Section:

QUESTIONS

NONE-11

In RAI Question 1.a, the NRC staff requested confirmation that the alternative examinations will be performed in accordance with the ASME Code as conditioned in § 50.55a of Title 10 of the *Code of Federal Regulations* (10 CFR). The NRC staff requests confirmation that the proposed remote VT-1 visual examination will be performed in accordance with ASME Code, Section XI, as conditioned in 10 CFR 50.55a, which requires that a visual examination performed instead of an ultrasonic examination has a magnification that has a resolution sensitivity to resolve 0.044 inch (1.1 mm) lower case characters without an ascender or descender (e.g., a, e, n, v).

NONE-12

In RAI Question 1.b.ii, the NRC staff requested a description of the alternative examinations. The NRC staff requests confirmation that the proposed remote VT-1 visual examination will cover essentially 100 percent of the examination volume for the inlet, outlet, and DVI nozzles as defined in ASME Code, Section XI, Figure IWB-2500-7(b). If not, then provide sketches or some other means detailing the examination volume for the inlet, outlet, and DVI nozzles that will be covered by the proposed remote VT-1 visual examination.

NONE-13

In the letter dated December 8, 2017, SNC stated that the stresses at the nozzle corner region for the Advanced Passive 1000 (AP1000) are similar to the operating fleet. In order to determine whether the operational experience from the current operating fleet applies, the NRC staff requests the following information:

- a. The design stress state and operating stresses at the nozzle corner region from the AP1000 reactor pressure vessel design report for each inlet, outlet, and DVI nozzle. In addition, these stresses should specifically include any thermal cycling and stratification loading for each inlet, outlet, and DVI nozzle.
- b. An evaluation that compares the design stress state and operating stresses at the nozzle corner region for each inlet, outlet, and DVI nozzle of the AP1000 provided in (a) above to the design stress state and operating stresses in the nozzle corner region of similar designed nozzles for the current operating fleet. This should include a discussion of why each inlet, outlet, and DVI nozzle is similar to the operating fleet nozzle designs.

