

H.A.F.A. INTERNATIONAL, INC.

7545 Central Industrial Drive
Riviera Beach, Florida 33419
(305) 848-5252

October 7, 1987
HII-87-1227
40-40-7
DCC-002 SCOPE

Mr. George Johnson
Nuclear Regulatory Commission
Material Branch
7920 Norfolk Avenue
Mail Stop P-842
Bethesda, MD 20814

SUBJECT: Informal update meeting with NRC Staff Members, Bethesda, MD.

Dear Mr. Johnson:

The following is a recap of the topics that were discussed during our September 30, 1987 meeting.

Attendees:

G. Johnson,	NRC	M. Turnbow,	TVA
W. Hazelton,	NRC	R. Jones,	TVA
D. Smith,	NRC	F. Askwith,	HAFA
G. Gibbs,	TED	H. Askwith,	HAFA
E. Caba,	TED		

The following informational topics where Instrumented Inspection Techniques (IIT) will be utilized, was presented by H.A.F.A. Int'l. Inc.:

A. Main Steam Line Testing.

The testing on the Main Steam Lines will be performed while the plant is coming up to mode 3. Standard HAFA Acoustic Emission Leak Testing will be utilized during this test and will be supplemented with a VT-2 examination. (See Attachment A and Al)

B. Main Steam Blowdown.

The Main Steam Blowdown testing will be performed at Mode 4. Standard HAFA Acoustic Emission Leak Testing will be utilized during this test and will be supplemented with a VT-2 examination. (See Attachment B)

C. Main Feedwater

The Main Feedwater Testing will be performed at Mode 4. Standard HAFA Acoustic Emission Leak Testing will be utilized during this test and will be supplemented with a VT-2 examination. (See Attachment C)

Consultants to the Energy Industry

~~880+200091 ZPP.~~

D. Service Water.

The Service Water System Testing will be performed during normal plant operation utilizing IIT and will be supplemented with a VT-2 examination. (See Attachment D)

E. Component Cooling System.

This test will be performed on the main header during Mode 6, utilizing IIT and will be supplemented with a VT-2 examination and the balance of the system will be performed during normal plant operation. (See Attachment E)

F. Decay Heat System.

The Decay Heat System Testing will be performed during Mode 6, utilizing IIT and will be supplemented with a VT-2 examination. (See Attachment F)

G. Residual Heat Removal System.

The R.H.R. Valve & Piping Testing will be at 500 psig utilizing IIT, and supplemented with a VT-2 examination while the plant is coming up to Mode 3. (See Attachment G)

H. Non Isolation Valves.

Piping which has valves that do not perform an isolation function (e.g. governor valves) will be treated as a opened ended system with no leak testing being performed on those valves. (No sketch necessary, generic statement)

We have attached color coded sketches showing the areas of testing (e.g. CCS main header) that will be performed on the systems listed above. It should also be noted that any applicable minor valve repairs or replacement will also be inspected by Volumetric Examination as required by the construction Code of the respective plant.

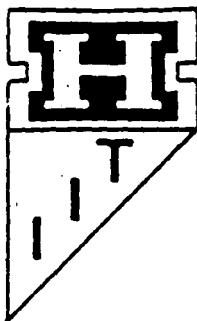
Very truly yours,



H. H. Askwith
Vice President

HHA/dm

cc:	G. Gibbs	TED	W
	E. Caba	TED	W/O
	J. Ewald	TED	W
	D. Weakland	DLC	W
	D. Grabski	DLC	W/O
	K. Grada	DLC	W/O



H.A.F.A. INTERNATIONAL, INC.

7545 Central Industrial Drive
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November 12, 1987
HII-87-1255
DLC-002 Scope

Mr. D. Weakland
Duquesne Light Company
Beaver Valley Nuclear Power Station
Post Office Box 4, Mail Drop BV-SAPS
Shippingport, Pennsylvania 15077

SUBJECT: Trip Report on Meeting with Nuclear Regulatory Commission Staff,
Bethesda, Maryland on November 5, 1987

Dear Mr. Weakland:

A meeting was held with NRC staff members in Bethesda, Maryland on November 5, 1987. Those in attendance were as follows:

NRC	HAFA
G. Johnson	F. Askwith
W. Hazelton	H. Askwith

The topics of discussion were reduction of test holding time on the Main Steam, Main Feedwater and Blowdown systems from two hours to ten minutes. The NRC staff members in the meeting agreed that the holding time of two hours was not warranted when performing IIT on the above systems.

Since the Safety Evaluation Report in Topical Report HAFA 135 (P-A) states a two hour holding time, DLC will have to file a letter to the NRC staff with full justification for the holding time reduction.

HAFA will draft the letter for DLC to forward to the NRC on your letterhead justifying the ten minute hold as an acceptable alternative to the two hour hold. This task assignment was discussed and confirmed by Mr. D. Grabski.

Sincerely,

H. H. Askwith
Vice President

HHA/er

cc: Mr. G. Johnson, NRC Mr. D. Grabski
 Mr. W. Hazelton, NRC Mr. K. Grada



A.A.F.A. INTERNATIONAL.

7645 Central Industrial Drive
Riviera Beach, Florida 33419
(305) 848-5252

REF. 5

MEMORANDUM

February 6, 1986
III-86-1134-T
PO# 026-Q-93872A-MB

TO: J. Lingenfelter
FROM: Jon Hallen *Hallen Jr.*
SUBJECT: Meeting with the NRC, Bethesda, Md., February 4, 1986

IN ATTENDENCE:

Mr. George Johnson - NRC
Mr. Horace Shaw - NRC
Dr. C. Y. Cheng - NRC
Mr. Herbert Askwith - HAFA
Mr. Jon Hallen - HAFA

The purpose of the meeting was to discuss the implementation of IIT to the Main Steam supply of the Auxiliary Feedwater turbines.

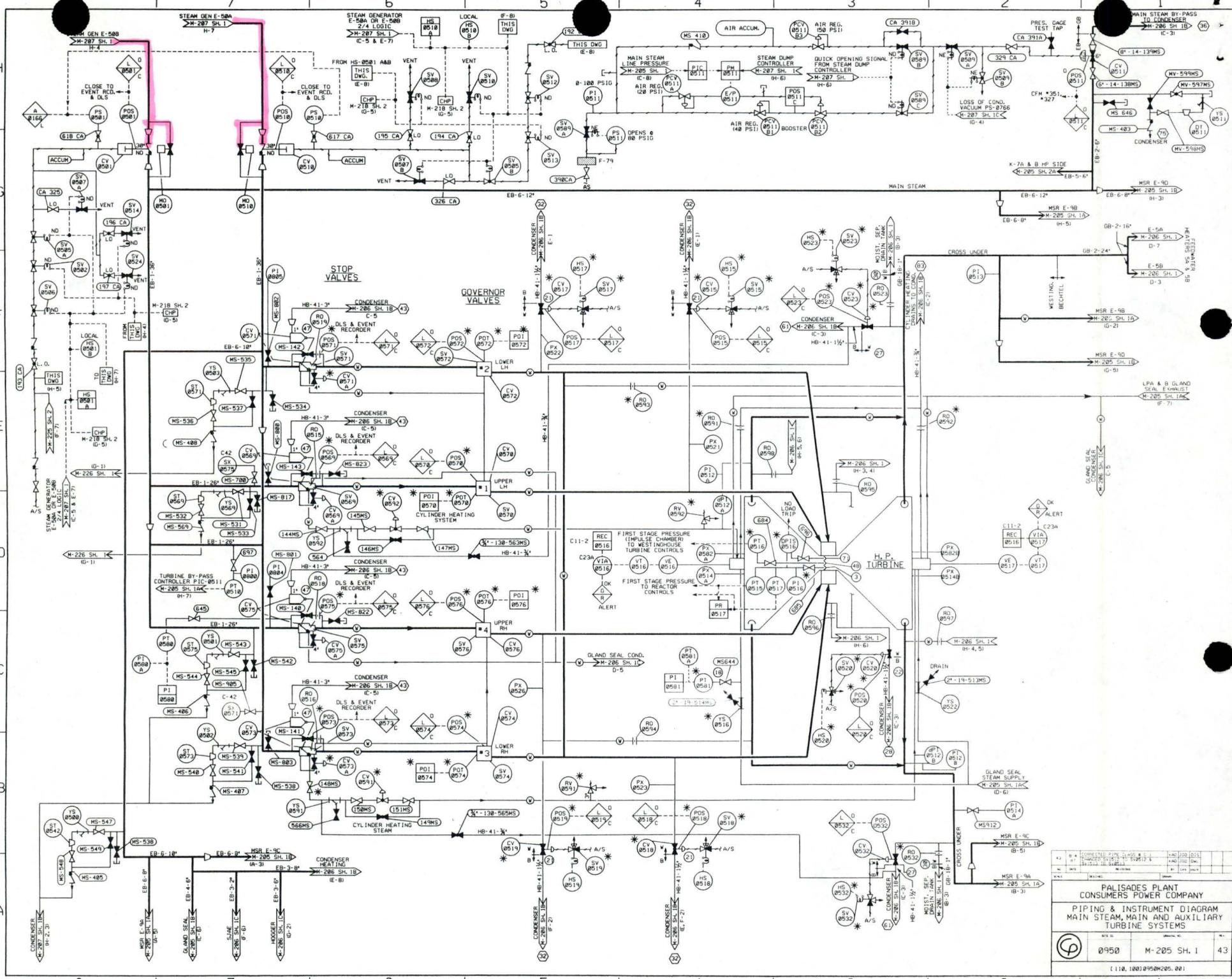
Mr. Askwith was concerned that the HAFA Topical Report 135 (P)(A) did not address the use of IIT for the testing of the replacements and modifications completed on the Main Steam supply of the Auxiliary Feedwater turbines. This testing would involve two (2) phases. The first would check the leak tightness of the check valves in the subsystem. This would be performed by injecting water into the subsystem at a pressure of 1050 psi and measuring the leakage past the check valves. The second phase would use the steam from the Steam Generator during Mode 3 (Hot Stand By) to pressurize the subsystem to normal operating pressure. This would be followed by using IIT and VT-2 examination to test the system piping.

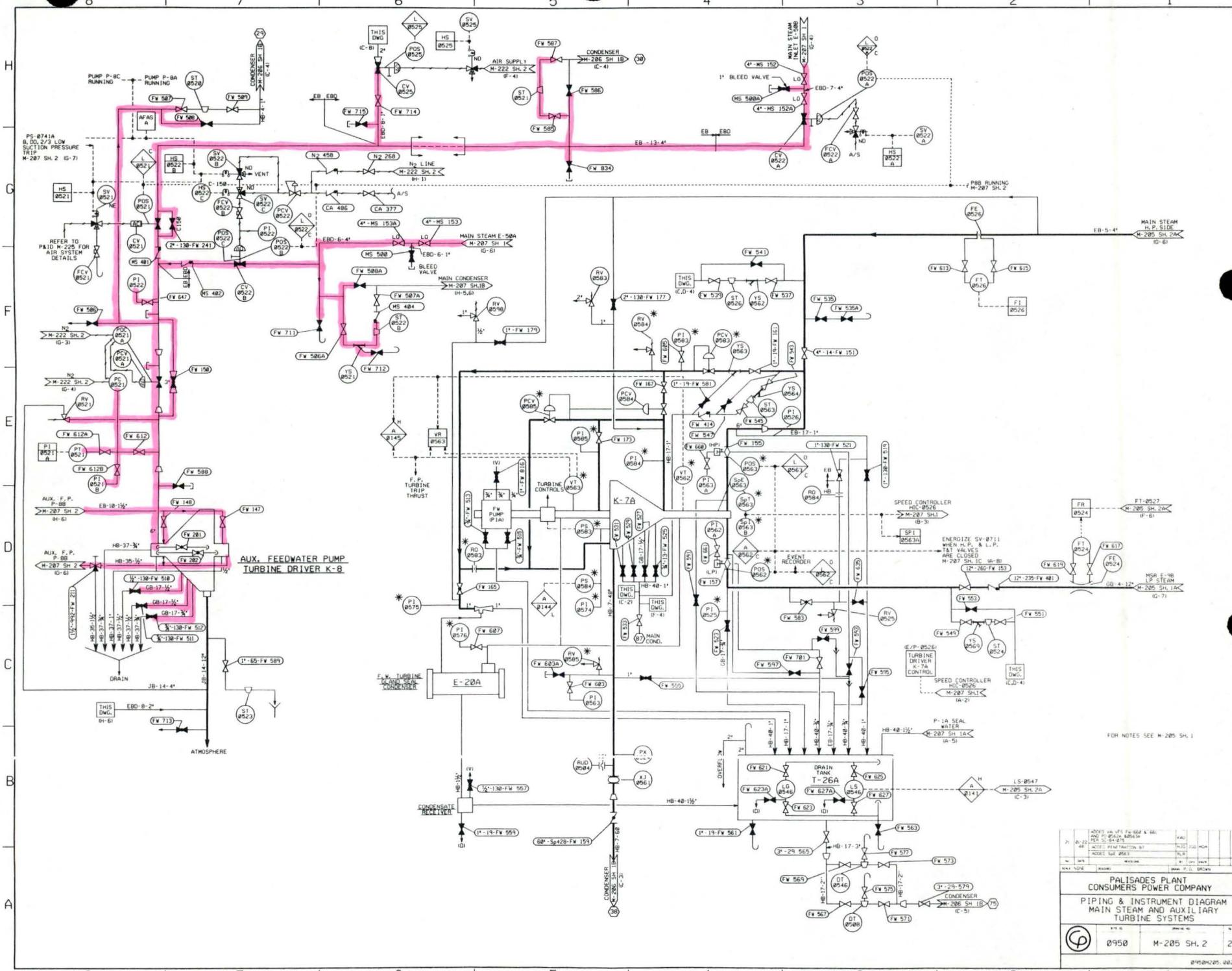
In answer to Mr. Askwith's concerns, Mr. Johnson stated that the HAFA Topical Report 135 (P)(A) covered the ASME Section XI requirements for pressurizing the subsystem since the replacements and modifications are required to be pressure tested. He also suggested that Toledo Edison write an information letter to the NRC describing the methods of testing that will be used for this test.

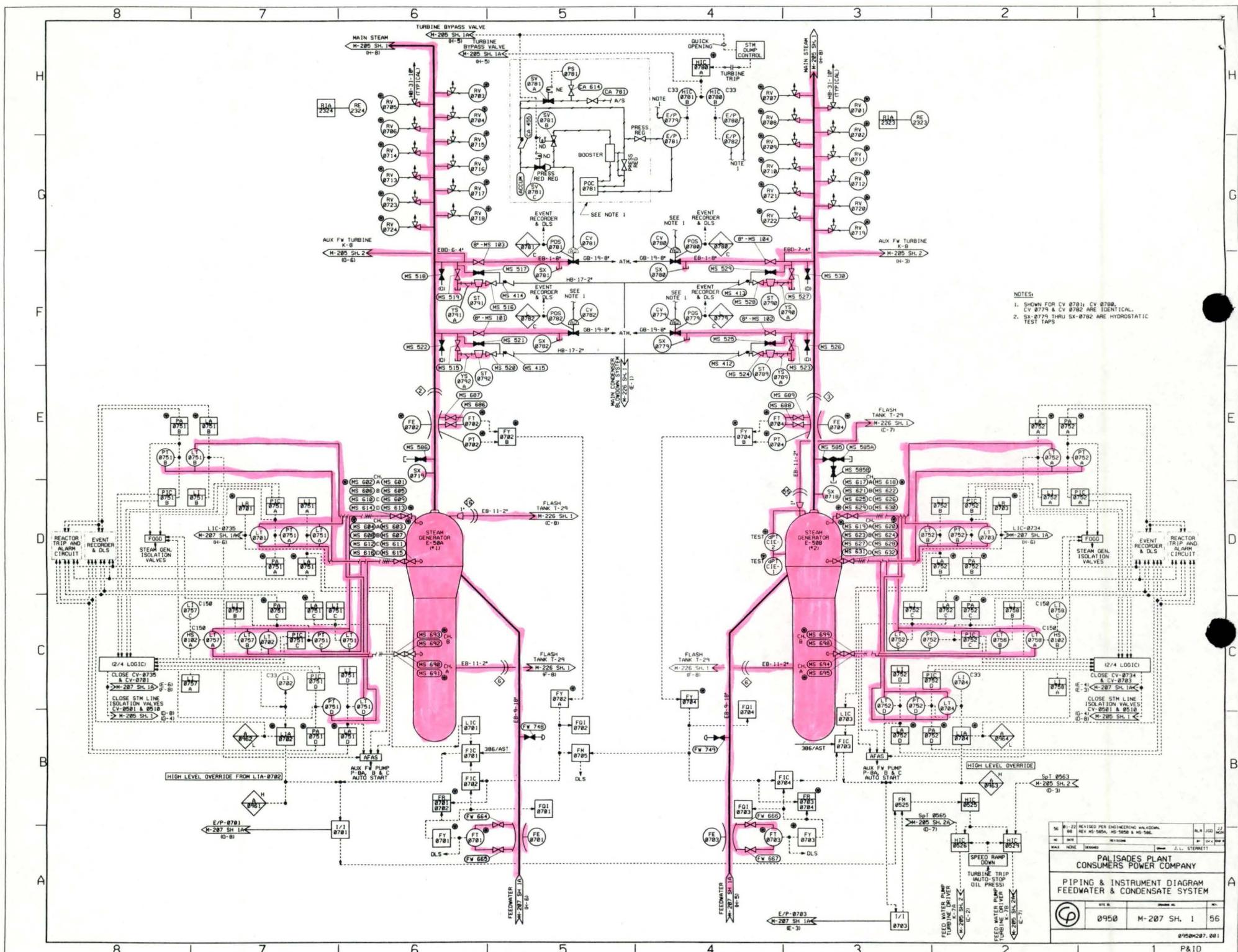
The NRC also suggested that they want a three (3) hour seminar to be held with HAFA and their reviewing staff to discuss code requirements versus practical testing of pumps, valves, and systems concerning pressure testing and leakage limits of LWR's. HAFA will notify the NRC as to the date of this seminar.

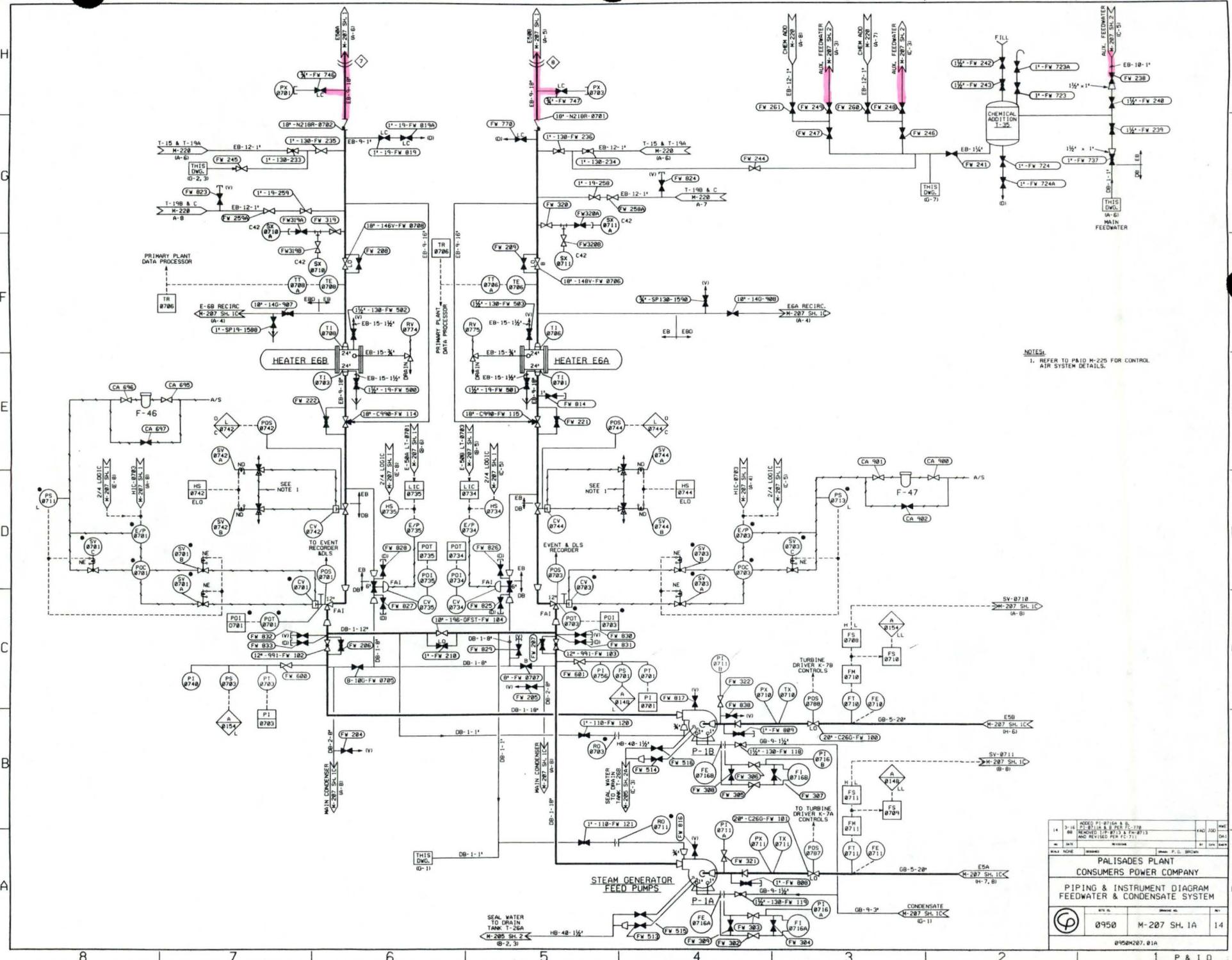
JH/mw

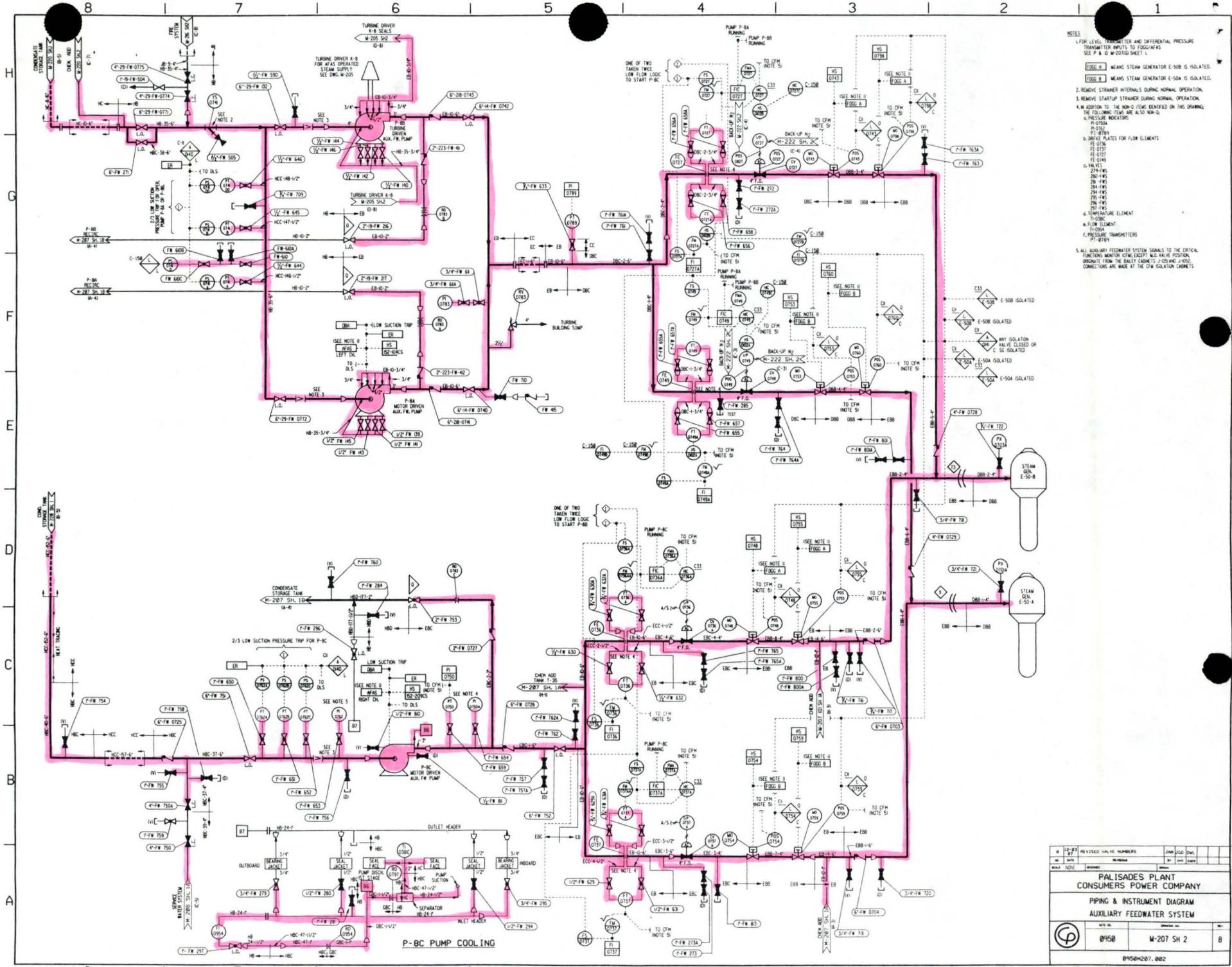
cc: Mr. D. Danielson, NRC Region III
Mr. W. Culdeanmont, NRC Region III
Mr. E. Caba, TED
Mr. J. Ewald, TED
Mr. S. Quenzer, TED
Mr. J. Williams, Jr., TED
Mr. C. Daft, TED
Mr. R. Flood, TED
Mr. C. Ackerman, TED
Mr. T. Bloom, TED
Mr. G. Johnson, NRC
Mr. H. Shaw, NRC
Dr. C. Y. Cheng, NRC











NOTES
6. FOR LEVEL TRANSMITTER AND DIFFERENTIAL PRESSURE
TRANSMITTER INPUTS TO FOGG/AFAS
SEE P & G M-2010 SHEET L

- FIGURE 3-6 MEANS STEAM GENERATOR L-504 IS ISOLATED.**

 - 2. REMOVE STRAINER ELEMENTS DURING NORMAL OPERATION.
 - 3. REMOVE STARTUP STRAINER DURING NORMAL OPERATION.
 - 4. IN ADDITION TO THE NO. 9 AND NO. 10 STRAINERS, THE FOLLOWING ARE ALSO IDENTIFIED:
 - a. PRESSURE INDICATORS
 - b. PI-2704A
 - c. PI-2705A
 - d. DRAIN VALVES FOR FLOW ELEMENTS
 - e. FE-0116
 - f. FE-0117
 - g. FE-0121
 - h. FE-0124
 - i. NUVIS
 - j. 274-FMS
 - k. 280-FMS
 - l. 281-FMS
 - m. 284-FMS
 - n. 285-FMS
 - o. 295-FMS
 - p. 297-FMS
 - q. 298-FMS
 - d. TEMPERATURE ELEMENT
 - 1. THERMOCOUPLE
 - 2. THERMISTOR
 - 3. THERMOCOUPLE
 - 4. THERMISTOR
 - e. LIQUID LEVEL
 - f. PI-2706A
 - g. PRESSURE TRANSMITTERS
 - h. PT-2789A

5. ALL AUXILIARY FEEDWATER SYSTEM ISOLATION VALVE IS LOCATED ON THE CRITICAL DRAIN LINE. THIS IS A FULL BLOWDOWN POSITION. ISOLATION IS MADE FROM THE FAULTY CAVITIES J-054 AND J-055. CONNECTIONS ARE MADE AT THE CTW ISOLATION CAVITIES.

5. ALL AUXILIARY FEEDWATER SYSTEM SIGNALS TO THE CRITICAL FUNCTIONS MONITOR (CFM), EXCEPT M.D. VALVE POSITION, ORIGINATE FROM THE BAILEY CABINETS J-103&14 AND J-1052. CONNECTIONS ARE MADE AT THE CFM ISOLATION CABINETS

B-123		REVISED VALVE NUMBERS	JAN 150 D/W
DATE		REVISION	BY DATE
NOV 1964		JAN 1965	
PALISADES PLANT CONSUMERS POWER COMPANY			
PIPING & INSTRUMENT DIAGRAM			
AUXILIARY FEEDWATER SYSTEM			
	ITEM NO.	PRINTING NO.	REV.
	0950	M-207 SH 2	8
0950M-207, 002			

