

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Palisades Nuclear Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 5 5	PAGE (3) 1 OF 0 3
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TITLE (4)  
IMPROPER VALVE LINEUP RESULTS IN REACTOR BEING CRITICAL AT LESS THAN 525 DEGREES F

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	1 2	8 7	8 7	0 2 2	0 0				N/A		0 5 0 0 0
									N/A		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											
POWER LEVEL (10) 0 0 0	20.402(b)			20.408(c)			80.73(a)(2)(iv)			73.71(b)		
	20.408(a)(1)(i)			80.38(a)(1)			80.73(a)(2)(v)			73.71(a)		
	20.408(a)(1)(ii)			80.38(a)(2)			80.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)		
	20.408(a)(1)(iii)			80.73(a)(2)(i)			80.73(a)(2)(vii)(A)					
	20.408(a)(1)(iv)			80.73(a)(2)(ii)			80.73(a)(2)(vii)(B)					
20.408(a)(1)(v)			80.73(a)(2)(iii)			80.73(a)(2)(viii)						

LICENSEE CONTACT FOR THIS LER (12)

NAME JGLewis, Technical Director, Palisades	TELEPHONE NUMBER
	AREA CODE: 6 1 6      7 1 6 4 - 8 9 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
A									
B									

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)       NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 12, 1987 at approximately 2210, with the reactor critical, the primary coolant system temperature dropped below 525 degrees F. The reactor was taken critical at 2018. This occurrence, which is contrary to Palisades Technical Specification 3.1.3(c) resulted from an improper valve position.

The decrease in primary coolant system (PCS) temperature to 524.8 degrees F for 15 seconds was the result of main feedwater regulating valve CV-0701 being placed in the open position when it is normally closed prior to turbine synchronization. The starting of main feedwater pump P-1B with CV-0701 in the open position caused continued feed to steam generator E-50A and the subsequent PCS temperature drop. After discovery of the mispositioned valve, it was placed in the required position (ie, closed) and PCS temperature returned to normal.

Plant operating procedures have been changed to include a sign-off after operator verification that main feedwater regulating valves CV-0701 and CV-0703 are in the closed position. The Control Room operator involved in the occurrence has been counseled regarding required standards for control panel awareness.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		87	022	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description

On July 12, 1987 at approximately 2210, with the reactor critical, the primary coolant system temperature dropped below 525 degrees F. The reactor was taken critical at 2018. This occurrence, which is contrary to Palisades Technical Specification 3.1.3(c) resulted from an improper valve position.

While returning the Plant to service, Operations personnel started main feedwater pump P-1B [SJ;P] in accordance with General Operating Procedure (GOP) 4. Immediately thereafter, Control Room operators noticed steam generator, E-50A [AB;SG] level increasing. The Control Room operator then identified main feedwater water regulating valve, CV-0701 [SJ;FCV] to be in the open position. The regulating valve was promptly returned to its normal closed position and steam generator level stabilized.

CV-0701 was in the open position and P-1B was running during the 15 second period when the primary coolant system when temperature dropped to 524.8 degrees F. This drop below the administrative Technical Specification limit of 525 degrees F was the result of excessive heat removal due to the continued feeding of steam generator, E-50A.

Cause Of The Event

The drop in primary coolant system temperature to 524.8 degrees F for 15 seconds was the result of main feedwater regulating valve CV-0701 being in the open position when normal Plant line-up prior to turbine synchronization for the valve is closed. This line-up resulted in continued feeding of steam generator E-50A and the subsequent primary coolant system temperature decrease.

The Control Room operator on-shift at the time of the event was not aware that CV-0701 was in the open position prior to starting main feedwater pump P-1B.

Corrective Action

General Operating Procedure 4, "Turbine Generator Startup From Hot Standby" has been changed to include a sign-off for verification that main feedwater regulating valves CV-0701 and CV-0703 are closed and that their associated controllers FIC-0701 and FIC-0703 are in the manual mode. This line-up is a standard operating practice which has been performed in the past without procedural guidance.

The Control Room operator involved with this occurrence has been counseled by the Operations Superintendent regarding required standards for control panel awareness.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Analysis Of The Event

As primary coolant system temperature was below 525 degrees F (ie; 524.8) for only 15 seconds and prompt operator action mitigated the event causing the temperature decrease, no threat to the general public was imposed. Also, since an over cooling event resulting from continued feedwater flow with the reactor critical is an analyzed condition, no safety hazard is believed to exist.

The event is being reported per 10CFR50.73 (a)(2)(i) as an operational condition prohibited by Plant Technical Specifications.

Additional Information

For additional information regarding occurrences of the reactor being critical with the primary coolant system less than 525 degrees F, reference Licensee Event Reports 84-014 and 87-018.



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General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

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Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 - PALISADES PLANT -  
LICENSEE EVENT REPORT 87-022 - IMPROPER VALVE LINEUP  
IN REACTOR BEING CRITICAL AT LESS THAN 525 DEGREES F

Licensee Event Report (LER) 87-022, (Improper Valve Lineup in Reactor Being  
Critical at Less than 525 Degrees F) is attached. This event is reportable to  
the NRC per 10CFR50.73(a)(2)(i).

Brian D Johnson  
Staff Licensing Engineer

CC Administrator, Region III, USNRC  
NRC Resident Inspector - Palisades

Attachment

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