

Docket No. 50-255

OCTOBER 01 1980

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Mr. David P. Hoffman  
Nuclear Licensing Administrator  
Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

Dear Mr. Hoffman:

By letter dated November 16, 1979, we sent you the list of topics which would not be reviewed by your plant in the Systematic Evaluation Program (SEP). These topics were deleted from the SEP review because they were not applicable to your plant or they were being reviewed as part of an ongoing generic issue outside the SEP. The following topics should be added to the November 16 listing. Removal of these additional topics from the SEP review results from 1) related generic NRC activities and 2) non-applicability of topics to certain facilities. The following is a list of such topics and indicates their disposition:

<u>Topic</u>	<u>Categorization</u>	<u>Disposition</u>
II-2.B	Generic	Compliance with Appendix E is being evaluated under the Emergency Preparedness Program Office (EPP) effort. Compliance with Appendix I is being evaluated as part of the NRC Appendix I review effort.
II-2.D	Generic	Compliance is being evaluated under the EPP effort.
III-8.B	Deleted	This topic was originally intended for BWR's.
VI-8	Generic	Will be reviewed by NRC as part of TMI Task Action Plan (NUREG-0660) as specified in May 7, 1980 letter to all operating reactor licensees.
XV-11	Deleted	This item applies specifically to BWR's.
XV-18	Deleted	This item applies specifically to BWR's. The radiological consequences of steam line breaks for PWR's is part of Topic XV-2.

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OFFICE  
SURNAME  
DATE

OCTOBER 01 1980

The following two topics will not be reviewed by SEP in their entirety due to overlap with other NRC activities:

<u>Topic</u>	<u>Disposition</u>
II-1.B	Those portions dealing with plant emergency plans will be evaluated as part of the EPPO effort.
X	Those portions of the auxiliary systems being reviewed as part of Lessons Learned, the TMI Action Plan and IE Circular 80-04 will not be reviewed by SEP.

In addition, the original DBE review list indicated that Group VI events, "Uncontrolled Rod Withdrawal at Power" and "Uncontrolled Rod Withdrawal - Low Power Startup" were part of Topic XV-13 which does not apply to PWR's. The actual topic reference should be Topic XV-8, "Control Rod Misoperation" which is applicable to PWR's. The new topic reference list for Topic XV-8 should include Standard Review Plans 15.4.1 and 15.4.2.

Enclosure 1 to this letter includes a revised listing of the DBE Accident and Transient Groups applicable to PWR's. Of note should be the addition of Group XII, "Steam Generator Tube Failure and the inclusion of Fuel Handling Accidents in Group VIII.

Sincerely,

Original signed by  
Dennis M. Crutchfield

Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

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Enclosure:  
As stated

cc w/enclosure:  
See next page

OFFICE	DL ORB #5	DL <i>DM</i> #5			
SURNAME	TWambach:cc	DCrutchfield			
DATE	9/29/80	10/1/80			



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 1, 1980

Docket No. 50-255

Mr. David P. Hoffman  
Nuclear Licensing Administrator  
Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

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<u>Topic</u>	<u>Categorization</u>	<u>Disposition</u>
II-2.B	Generic	Compliance with Appendix E is being evaluated under the Emergency Preparedness Program Office (EPP0) effort. Compliance with Appendix I is being evaluated as part of the NRC Appendix I review effort.
II-2.D	Generic	Compliance is being evaluated under the EPP0 effort.
III-8.B	Deleted	This topic was originally intended for BWR's.
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October 1, 1980

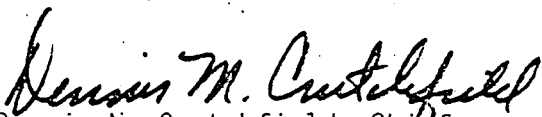
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In addition, the original DBE review list indicated that Group VI events, "Uncontrolled Rod Withdrawal at Power" and "Uncontrolled Rod Withdrawal - Low Power Startup" were part of Topic XV-13 which does not apply to PWR's. The actual topic reference should be Topic XV-8, "Control Rod Misoperation" which is applicable to PWR's. The new topic reference list for Topic XV-8 should include Standard Review Plans 15.4.1 and 15.4.2.

Enclosure 1 to this letter includes a revised listing of the DBE Accident and Transient Groups applicable to PWR's. Of note should be the addition of Group XII, "Steam Generator Tube Failure and the inclusion of Fuel Handling Accidents in Group VIII.

Sincerely,

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

Enclosure:  
As stated

cc w/enclosure:  
See next page

October 1, 1980

cc w/enclosure:

M. I. Miller, Esquire  
Isham, Lincoln & Beale  
Suite 4200  
One First National Plaza  
Chicago, Illinois 60670

Mr. Paul A. Perry, Secretary  
Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

Judd L. Bacon, Esquire  
Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

Myron M. Cherry, Esquire  
Suite 4501  
One IBM Plaza  
Chicago, Illinois 60611

Ms. Mary P. Sinclair  
Great Lakes Energy Alliance  
5711<sup>st</sup> Summerset Drive  
Midland, Michigan 48640

Kalamazoo Public Library  
315 South Rose Street  
Kalamazoo, Michigan 49006

Township Supervisor  
Covert Township  
Route 1, Box 10  
Van Buren County, Michigan 49043

Office of the Governor (2)  
Room 1 - Capitol Building  
Lansing, Michigan 48913

Director, Technical Assessment  
Division  
Office of Radiation Programs  
(AW-459)  
U. S. Environmental Protection  
Agency  
Crystal Mall #2  
Arlington, Virginia 20460

U. S. Environmental Protection  
Agency  
Federal Activities Branch  
Region V Office  
ATTN: EIS COORDINATOR  
230 South Dearborn Street  
Chicago, Illinois 60604

Charles Bechhoefer, Esq., Chairman  
Atomic Safety and Licensing Board  
Panel  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dr. George C. Anderson  
Department of Oceanography  
University of Washington  
Seattle, Washington 98195

Dr. M. Stanley Livingston  
1005 Calle Largo  
Santa Fe, New Mexico 87501

Resident Inspector  
c/o U. S. NRC  
P. O. Box 87  
South Haven, Michigan 49090

Palisades Plant  
ATTN: Mr. J. G. Lewis  
Plant Manager  
Covert, Michigan 49043

William J. Scanlon, Esquire  
2034 Pauline Boulevard  
Ann Arbor, Michigan 48103

Mr. Richard E. Schaffstall,  
KMC, Incorporated  
1747 Pennsylvania Avenue, N. W.  
Washington, D. C. 20006

PWR DBE TOPIC LIST

Accident and Transient Groups

	<u>RELATED TOPIC</u>	<u>DESIGN BASIS EVENT</u>	<u>SRP</u>
<u>Group I</u> <u>(PWR)</u>	XV-1	Decrease in feedwater temperature	15.1.1
	XV-1	Increase in feedwater flow	15.1.2
	XV-1	Increase in steam flow	15.1.3
	XV-1	Inadvertent opening of steam generator relief/safety valve	15.1.4
	XV-9	Start-up of inactive loop	15.4.4
	XV-10	System malfunction causing decrease in boron concentration	15.4.6
<u>Group II</u>	XV-3	Loss of external load	15.2.1
	XV-3	Turbine trip	15.2.2
	XV-3	Loss of condenser vacuum	15.2.3
	XV-3	Steam pressure regulatory failure	15.2.5
	XV-5	Loss of feedwater flow	15.2.7
	XV-6	Feedwater system pipe break	15.2.8
<u>Group III</u>	XV-2	Steam line break inside containment	15.1.5
	XV-2	Steam line break outside containment	15.1.5
<u>Group IV</u>	XV-4	Loss of AC power to station auxiliaries	15.2.6
	XV-24	Loss of all AC power	-
<u>Group V</u>	XV-7	Loss of forced coolant flow	15.3.1
	XV-7	Primary pump rotor seizure	15.3.3
	XV-7	Primary pump shaft break	15.3.4
<u>Group VI</u>	XV-8	Uncontrolled rod assembly withdrawal at power	15.4.2
	XV-8	Uncontrolled rod assembly withdrawal low power start-up	15.4.1
	XV-8	Control rod misoperation	15.4.3
	XV-12	Spectrum of rod ejection accidents	15.4.8
<u>Group VII</u>	XV-19	Spectrum of loss of coolant accidents	15.6.5
<u>Group VIII</u>	XV-21	Drop of cask or heavy equipment	15.7.5
	XV-20	Radiological Consequences of Fuel Damaging Accidents (inside and outside containment)	15.7.4
<u>Group IX</u>	XV-15	Inadvertent opening of PWR pressurizer relief valve	15.6.1
<u>Group X</u>	XV-14	Inadvertent operation of ECCS or	15.5.1
		CVCS malfunction that causes an increase in coolant inventory	15.5.2

Enclosure 6

-2-

	<u>RELATED TOPIC</u>	<u>DESIGN BASIS EVENT</u>	<u>SRP</u>
<u>Group XI</u>		Not applicable to PWRs	
<u>Group XII</u>	XV-17	Steam Generator Tube Failure	15.6.3