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Enclosure 1

August 21, 1974

Mr. James G. Keppler
Regional Director
Directorate of Regulatory
Operations - Region III
U.S. Atomic Energy Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Dresden Station Semi-Annual Report,
AEC Dkts 50-10, 50-237 and 50-249.

Dear Mr. Keppler:

Attached is the Semi-Annual Report for Dresden Station as required by the Technical Specifications for Units 1, 2 and 3 for the period of January 1, 1974 through June 30, 1974.

One copy of this report is provided for your use, and 39 copies are being transmitted by this letter to the Acting Director, Directorate of Licensing.

Very truly yours,

J S Abel

J. S. Abel
Nuclear Licensing Administrator
Boiling Water Reactors

Att.

cc: Mr. Edson G. Case (w/att.)

Dupe
8008200221

4. Surveillance

All licensed required surveillance for the six month period January 1, 1974 to June 30, 1974 were satisfactorily completed. The unit was shutdown for refueling operations during the entire reporting period. Required refueling surveillances were continued. These surveillances included such major tests as the primary containment integrated leak rate test, core spray logic and flow tests, simulated automatic actuation test of the core spray system, station battery load discharge test, recirculation of the high pressure boron tank through the low pressure boron tank, automatic actuation test of the primary isolation valves, control rod drive timing and latching, friction and scram tests, and the required instrument calibrations. In addition, the inservice inspection program was satisfactorily completed as well as the other tests. A description of the primary containment integrated leak test and inservice inspection program is given below.

INTEGRATED PRIMARY CONTAINMENT LEAK TEST

A second primary containment integrated leak rate test was performed during the 8th partial refueling outage from January 23 through January 28. The test was conducted in accordance with Technical Specification Section 4.7.A.1.g. This section requires a primary containment integrated leak test be conducted at least three times in a ten (10) year period.

After the containment had been pressurized to 20 psig (Pa) on January 25, 1974, the conditions were allowed to stabilize and leakage data was taken. After five (5) hours of data taking, the average leakage was determined to be 8.15 %/day. Investigation revealed that there was excessive leakage through an unused 2" decontamination line from the "A" cleanup system to radwaste, exiting the sphere through penetration H-45. The line was blind flanged at a spool piece connection outside the sphere in the radwaste pipeway. Additionally, leakage was noted through the secondary steam generator sample drain valve, FCV-510. Attempts to isolate this leakage failed. The sphere pressure was then increased to 20 psig and the test resumed.

After the leakage test was concluded, the verification test was conducted in accordance with ANSI N45.4 - 1972. The verification test lasted for approximately 8 hours with the leakage calculation of .2633 $\pm$ .034 %/day. The verification test met the acceptability criteria of 10CFR50, Appendix J which requires the verification test results to be within 0.25 La.

Since there were two known leakage paths from the sphere during the integrated sphere test (2" decon line blind flange after installation and valve FCV-510) it was decided to do an "as found" local test on the 2" decontamination line and the secondary steam generator sample drain valve, FCV-510. After repair of the penetrations, a second local test was performed and the difference subtracted from the leakage determined from the integrated leak test. The "as found" leakage through the 2" decontamination line was found to be .0009 %/day and through the secondary steam generator sample drain, FCV-510, .019 %/day. After repairs were made to FCV-510, the calculated leakage was .0000144 %/day. After the 2" decontamination line was seal welded, the calculated leakage was 0.00 %/day. The total difference between the "as found" and the repaired leakage is .019 %/day, thereby resulting in a primary containment integrated leakage of .2495 $\pm$ .034 %/day at a pressure of 20 psig.

The error bound of $\pm$ 0.034 %/day is a 2σ value and by definition represents a 95% confidence level bound. When the primary containment integrated leakage is evaluated at the upper bound, the result is 0.2835 %/day (0.2495 + 0.034). Since the 95% confidence level upper bound leakage (0.2835 %/day) is less than the Technical Specification 4.7.A.1.e requirement (0.30 %/day) for plant startup, the test is successful.

INSERVICE INSPECTION PROGRAM

The 1973 refueling outage inservice inspection program was completed on June 18, 1974 in accordance with Section XI of the ASME Code (1971 Summer Addenda). During this period, approximately 220 components were examined by either volumetric, surface and/or visual methods. Of the indications noted, none were evaluated to be of any significance.

On June 18, 1974 the reactor vessel and associated primary system piping were hydrostatically tested at 1060 psig and 280°F. At this time two pin-hole leaks were detected in the same pipe section in the north steam supply line to the emergency condenser. The defective pipe section was replaced according to code specifications and was satisfactorily hydrostatically tested at about 1600 psig.

5. Results of Periodic Containment Leak Rate Tests

Table 5 shows the results of the periodic containment leak rate tests performed during the period from January 1, 1974 to June 30, 1974.

6. Changes, Tests and Experiments Requiring Authorization From the Commission

No changes, tests, or experiments requiring commission authorization were performed during the period from January 1, 1974 to June 30, 1974.

7. Key Changes in Plant Operating Organization

The following key changes in plant operating personnel occurred during the period from January 1, 1974 to June 30, 1974.
Station Superintendent - Ben B. Stephenson
Operating Engineer - Eugene Budzichowski