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U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-255/80-03

Docket No. 50-255

License No. DPR-20

Licensee: Consumers Power Company
212 West Michigan Avenue
Jackson, MI 49201

Facility Name: Palisades Nuclear Generating Plant

Inspection At: Palisades Site, Covert, MI

Inspection Conducted: February 4-8,11,12,14,15,19-22 and 25-29, 1980

Inspectors: *for* B. L. Jorgensen 3-20-80

for J. K. Heller 3-20-80

Approved By: D. C. Boyd, Chief 3-20-80
Reactor Projects, Section 3-1

Inspection Summary

Inspection during February, 1980 (Report No. 50-255/80-03)

Areas Inspected: Routine resident inspection program activities during February, 1980, including: inspection during long-term shutdown; reportable occurrences; and actions on IE Bulletins and NUREG-0578 items. The inspection involved 78 inspector-hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified in the areas inspected.

Results: No items of noncompliance or deviations were identified in the areas inspected.

DETAILS

1. Persons-Contacted

*J. G. Lewis, General Manager, Palisades *H. J. Palmer, Technical Superintendent
B. L. Schaner, Operations Supervisor
B. L. Harshe, Senior Engineer
R. A. Vincent, Senior Engineer
R. L. Ritt, Senior Technical Analyst *G. H. R. Petitjean, Technical Engineer
G. W. Ford, General Engineer
C. E. Smith, Shift Supervisor
A. S. Kanicki, Shift Supervisor
R. Bennett, Plant Instructor

Several other members of the plant operations and technical staffs were also briefly contacted.

*Denotes those present at management interview on February 29, 1980.

2. Activities During Long Term Shutdown

The inspectors reviewed selected licensee activities being carried out during the current shutdown condition to ascertain compliance to applicable requirements.

a. Control Room

A number of visits to the control room verified adherence to limiting conditions for operation at cold shutdown, proper shift manning, adherence to selected procedures, and instrument/recorder operation. Selected operations logs were reviewed to maintain current knowledge of plant and system status.

b. Facility Tours

Tours of selected accessible plant areas, including the containment building, were conducted to assess equipment and plant conditions, radiological controls, security, and general safety. A few questions or minor problems were identified and discussed with licensee personnel, including: storage of the thermal shield on pedestals near the reactor cavity; spark control from a welding operation on the "B" steam generator feedline; positive access controls for a potential entrance/exit of the radiation controlled area; and numerous apparently burned-out indicator lights on the C-33 (shutdown)

panel. Where appropriate, the licensee initiated corrective measures responsive to the indicated minor problems.

No items of noncompliance or deviations were identified.

3. Reportable Events

The inspectors reviewed the licensee's actions for the following event reports to verify proper review and evaluation, corrective action, and compliance with associated license requirements. Records and personnel interviews were used in the review.

- a. LER 79-38: Service water pump P-7c discharge check valve failed to open. Disassembly and examination of the valve internals during the current outage revealed a generally deteriorated condition, more significant than the expected hinge pin corrosion. All three service water pump discharge check valves were ultimately removed in sequence, returned to the manufacturer for remanufacture/overhaul, reinstalled, and satisfactorily tested. This item is closed.
- b. LER 79-41: Safety analysis nonconservative in that excessive steam-generator feed by the condensate pumps, following a main steamline break without loss of offsite power, was not considered. A facility modification was performed to isolate the main feed flowpaths, including condensate, on low steam generator pressure. The inspectors discussed the modification with plant engineering staff, reviewed the facility change package, and witnessed successful pre-op testing of the modified system. This item is closed. An outstanding inspection item concerning the same subject, identified during a previous inspection ^{1/} is also considered closed based on the review discussed above.

Note: IE Bulletin 80-04 addresses similar concerns, but with a broader scope. The licensee's actions on Bulletin 80-04 will be reviewed during a future inspection.

- c. LER 79-42: Lift settings of two pressurizer code safety valves out of specification. All three safety valves were tested on steam during the current outage because the licensee received information that his standard test using cold nitrogen could be insufficiently accurate. The valves were reset and retested within specification. Future testing will be done on steam. The inspectors reviewed the test report and the documentation covering procurement of the testing services. This item is closed.

No items of noncompliance or deviations were identified.

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4. IE Bulletins and NUREG 0578

This inspection included a review of licensee activities relating to requirements of IE Bulletin based on the accident at the Three Mile Island Nuclear Plant and of NUREG 0578 "TMI-2 Lessons Learned Taskforce Status Report and Short-term Recommendations." Proper completion of many of these items constitutes a prerequisite to plant startup following the current outage.

The items can be categorized as facility items (modifications to systems and components) and procedure items. Procedure items will be reviewed in detail during a future special inspection.

- a. IE Bulletin 79-06B: Many of the specific requirements of this Bulletin were reviewed and reported on during a previous inspection. ^{2/} One facility item, relating to containment automatic isolation in NUREG 0578. This inspection included discussion with the project engineer concerning possible modifications. Final review of facility items relating to automatic containment isolation will be completed during a future inspection.

The several remaining procedure items will also be reviewed during a future inspection. One item, however, relating to notification of the NRC within one hour of uncontrolled/unexpected plant conditions, has been superseded by a Commission rulemaking noticed in the Federal Register (February 29, 1980) which specifies for all power reactor licensees new one-hour notification requirements, including uncontrolled/unexpected plant conditions. For the purpose of IE Bulletin 79-06B, this item is closed.

- b. NUREG 0578: The following facility items were discussed with the assigned project engineer(s) and an initial review of the modification packages was performed during this inspection:

- (1) Provision of emergency power to pressurizer heaters.
- (2) Provision of direct pressurizer relief and safety valve position indication
- (3) Provision of instrumentation for identifying inadequate core cooling
- (4) Leak testing of systems which could contain highly contaminated fluids
- (5) Provision of auto-start system for auxiliary feedwater

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- (6) Provision of safety-grade auxiliary feedwater flow instrumentation
- (7) Provision of qualified power supply for pressurizer relief valves and the associated block valves

Concerning item (5) above, the modification has been designed and is being installed with the exception of final connections, pending further safety reviews. Concerning item (7), the subject valves, except PORV 1043B, have always been on vital power. The modification to supply 1043B, have always been on vital power. The modification to supply 1043B on vital power is complete and was reviewed by the inspector, including satisfactory pre-operational testing. Item (7) above is considered closed.

No item of noncompliance or deviations were identified.

5. Management Interview

A management interview, attended as noted in Paragraph 1 above, was conducted following the conclusion of the inspection on February 29, 1980. The inspectors discussed the scope and findings of the inspection as described in these Details.