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August 20, 1979

Mr James G Keppler Office of Inspection and Enforcement Region III US Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

DOCKET 50-255 - LICESNE DPR-20 - PALISADES PLANT - REV 1 TO LER's 79-025 and 79-031

Revision 1 to Licensee Event Reports 79-025 and 79-031 which are reportable per Technical Specifications 6.9.2.a(9) and 6.9.2.b(4) respectively are attached.

David P Hoffman

Assistant Nuclear Licensing Administrator

CC: Director, Office of Nuclear Reactor Regulation Director, Office of Inspection and Enforcement

AF 1908 270389

PALISADES

NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

	CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	M IP AL 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5 6 5 ELICENSE CODE 14 15 CAT 58 6
CON'T 0 1 7 8	REPORT L 6 0 5 10 0 0 2 5 5 7 0 6 1 9 7 9 8 0 8 2 0 7 9 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
0 2	During part of Cycle II, the heat transfer coefficients for the CCW heat
0 3	exchangers were less than the design values used in the FSAR accident
0 4	analyses. Event reportable per Tech Spec 6.9.2.a(9). No threat to public
0 5	health or safety existed. An evaluation of the 1977 test data was completed
0 6	on January 2, 1979 and determined reportable on June 19, 1979.
0 7	L
0 8 7 8	
0 9	CODE SUBCODE S
	17 LER/RO REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 32
	ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT NPRD-4 PRIME COMP. COMPONENT TAKEN ACTION ON PLANT METHOD HOURS 22 ATTACHMENT FORM SUB. SUPPLIER MANUFACTURER LY 18 19 2 20 2 21 0 0 0 0 0 1 2 23 19 24 2 43 25 10 5 5 26 26 27 20 27 27 27 28 28 29 29 29 29 29 29 29 29 29 29 29 29 29
10	The tube sides of the CCW heat exchangers became excessively fouled due
11	to natural fouling during 1977. The tubes were cleaned. The heat
1 2	transfer coefficients have since been checked in 1979 after the evaluation
1 3	of the 1977 data and found to be acceptable. Tubes will be cleaned more
1 4	frequently in the future.
1 5	ECILITY % POWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 DISC
1 6 R	ELEASED OF RELEASE N/A AMOUNT OF ACTIVITY (35) 2 (33) Z (34) N/A LOCATION OF RELEASE (36) 9 10 11 44 45 80 PERSONNEL EXPOSURES
1 7	NUMBER TYPE DESCRIPTION (39) O O O O (37) Z (38) N/A
1 8	9 PERSONNEL INJURIES 13 80 NUMBER DESCRIPTION (41) 0 0 (40) N / A
	9 11 12 80 LOSS OF OR DAMAGE TO FACILITY 43 TYPEDESCRIPTION
1 9	Z 42 N/A
	PUBLICITY SSUED DESCRIPTION (45) NRC USE ONLY NRC USE ONLY
2 0	

UPDATE REPORT: PREVIOUS REPORT DATE 07/31/79

PALISADES

NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

	CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	M IP AL 1 2 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 4 5 5 CAT 58
0 1 7 8	REPORT L 6 0 5 0 0 0 2 5 5 7 0 6 2 8 7 9 8 0 18 12 0 7 19 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
0 2	An uncontrolled release of fission gases occurred during power reduction.
0 3	Total release time was 225 minutes. Total release was 1.9 Ci of Xe-133 and
0 4	2.13×10^{-4} Ci of I-131 which is .097% and 1.13%, respectively, of the
0 5	Annual Average Release Rates. Event reportable per TS 6.9.2.b(4). No
0 6	threat to public health or safety existed.
0 7	<u></u>
08	9 80
0 9 7 8	SYSTEM CAUSE CAUSE COMPONENT CODE SUBCODE SUBC
	LER/RO REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 31 32
	ACTION FUTURE ON PLANT METHOD HOURS 22 ATTACHMENT NPRO4 PRIME COMP. COMPONENT MANUFACTURER SUBMITTED FORM SUB. PRIME COMP. COMPONENT MANUFACTURER MA
10	During PCS diversion due to high level in the VCT, uncontrolled release of
11	fission gases occurred to the aux. building atmosphere through an opening
1 2	caused by a relief valve being removed from T-101B (WGDT) for repair.
1 3	Diversion was terminated and air samples taken. Plant personnel were
1 4	notified to stay clear of aux bldg. Relief valve was repaired and replaced.
1 5	STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 DISCO
	ELEASED OF HELEASE AMOUNT OF ACTIVITY (35) G 33 M 34 2.13 X 10 ⁻⁴ Ci of I-131 Relief Valve RV-1161 to Aux Bldg 9 REPROMISE EXPOSURES 44 45 BO 20 10 CATION OF RELEASE (36) Relief Valve RV-1161 to Aux Bldg 80
1 7 7 8	NUMBER 13 TYPE 39 DESCRIPTION (39) 9 11 12 13 13 13 19
1 8	PERSONNEL INJURIES NUMBER DESCRIPTION 41 O O O 40 N/A
7 8	9 11 12 LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION
1 9 7 8	Z 42 N/A 9 PUBLICITY (2) NDC USE ONLY
20	ISSUED DESCRIPTION (45) NRC USE ONLY NRC USE ONLY
, ,	9 10 88 80 90