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Docket No. 50-255

Mr. David Bixel
Nuclear Licensing Administrator APR 16 1979
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Dear Mr. Bixel:

This letter is to advise you of topics which have been deleted from the Systematic Evaluation Program (SEP) review of your facility. Enclosure 1 is a list of topics being deleted because the same safety issues are being reviewed as part of ongoing generic issues. As indicated at the onset of the SEP many generic issues would not be evaluated in the SEP; however, the staff will consider any criteria resulting from the completion of the evaluation of a generic issue prior to the integrated assessment of your facility.

Enclosure 2 is a list of topics that have been deleted from the review of your plant because they are not applicable to your facility or site.

Please review and identify any errors in these lists and provide your comments within 30 days of the date you receive this letter. If no response is received within that time, we will assume that you have no comments or corrections.

Sincerely,

Original signed by
Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

TRAY
CCP

Enclosures:
As Stated

cc w/enclosures:
See next page

As of 11/79 (for corrections to lists) 7905090029

OFFICE >	DOR:ORB #2	DOR:ORB #2	DOR:ORB #2	DOR:SEP	
SURNAME >	RDSilver:ah	HSmith	DLZiemann	PADiBenedetto	
DATE >	3/22/79	3/22/79	3/16/79	4/16/79	

Mr. David Bixel

- 2 -

APR 16 1979

cc w/enclosures:

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PALISADESSEP TOPICS NOT REQUIRING REVIEW DUE TO
ONGOING GENERIC LICENSING ACTIVITIES

- TOPIC V-1: COMPLIANCE WITH CODES AND STANDARDS
- COMMENTS: The Engineering Branch, DOR, is performing a review of all plants for compliance with inspection requirements of 10 CFR 50.55a(g).
- TOPIC V-3: OVERPRESSURE PROTECTION
- COMMENTS: Listed as TASK A-26 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Generic letters issued to PWR facilities in August 1976. Specific criteria has been identified to be used in the design of modifications intended to preclude exceeding the limits of 10 CFR 50, Appendix. G. Palisades Plant modifications and procedural changes implemented in response to an August 1976 NRC letter are under review.
- TOPIC V-7: REACTOR COOLANT PUMP OVERSPEED
- COMMENTS: Listed as TASK B-68 in NUREG-0371, dated November 1978. At this time no criteria has been developed for this task.
- TOPIC V-8: STEAM GENERATOR INTEGRITY
- COMMENTS: Listed as TASK A-3, 4, 5 of NUREG-0317, dated November 1978. The TASK ACTION PLAN completely envelopes the scope of this topic review.
- TOPIC V-13: WATER HAMMER
- COMMENTS: Listed as TASK A-1 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published dated November 1978. Generic letters sent to this facility May 1975 initiated an ongoing review for Palisades.
- TOPIC VI-2-B: SUBCOMPARTMENT ANALYSIS
- COMMENTS: Listed as TASK A-2 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published dated November 1978. Guidance letters issued to PWR licensees October 1975.

April 16, 1979

TOPIC VII-1-B: TRIP UNCERTAINTY

COMMENTS: Staff evaluations, issued August 17, 1978, state that review is being pursued independent of SEP to determine impact of instrument error and drift upon safety margins of trip setpoints, and that appropriate actions will be taken to assure that adequate safety margins are maintained.

TOPIC VII-4: FAILURES OF NON-SAFETY SYSTEMS AND SELECTED ESF'S

COMMENTS: The scope of this topic is contained in TASK A-17 of NUREG-0371, November 1978. The staff is developing a method of review. Criteria has not been established for implementation.

TOPIC VII-5: INSTRUMENTS FOR MONITORING ACCIDENTS

COMMENTS: TASK A-34 of NUREG-0371, November 1978. Criteria under development.

TOPIC VII-6: FREQUENCY DECAY

COMMENTS: The scope of this topic is completely addressed in TASK A-35 (2.C-10) of NUREG-0371, November 1978.

TOPIC VIII-1-A: EFFECTS OF DEGRADED GRID VOLTAGE

COMMENTS: The scope of this topic is contained in TASK A-35 (C-2, 3, 4, and 5) of NUREG-0371, November 1978. Initial letters sent to licensee August 1976.

TOPIC IX-6: FIRE PROTECTION

COMMENTS: This topic is being reviewed outside the SEPB as part of the ongoing licensing review implemented as a result of the Browns Ferry incident.

TOPIC XI-1: APPENDIX I

COMMENTS: This topic is part of an ongoing licensing review for all facilities to implement the limits of the Appendix I to 10 CFR 50.

TOPIC XI-2: RADIOLOGICAL MONITORING SYSTEMS

COMMENTS: Listed in NUREG-0371, November 1978, as TASK B-67, criteria and scope of review under development.

TOPIC XIII-2: SAFEGUARDS/INDUSTRIAL SECURITY

COMMENTS: Ongoing licensing review to implement the requirements of 10 CFR 73.55.

TOPIC XV-22: ANTICIPATED TRANSIENTS WITHOUT SCRAM

COMMENTS: TASK A-9 of NUREG-0371, November 1978, criteria for implementation under development.

TOPIC XV-23: STEAM GENERATOR MULTIPLE TUBE FAILURES

COMMENTS: This topic is being considered in conjunction with TASK A-2, 3, 4 of NUREG-0371. No criteria has been established on this topic.

PALISADESSEP TOPICS NOT APPLICABLE TO PALISADES

<u>TOPIC NO.</u>	<u>TITLE</u>	<u>COMMENTS</u>
III-10-C	Surveillance Requirements on BWR Recirculation Pumps	N/A BWR Safety Topic
IV-3	BWR Jet Pump Operating Indications	N/A BWR Safety Topic
V-2	Applicability of Code Cases	N/A at this time. To be reviewed for any future modifications using references to Code Cases.
V-9	Reactor Core Isolation Cooling System	N/A BWR Safety Topic
V-12-A	Water Purity of BWR Primary Coolant	N/A BWR Safety Topic
VI-2-A	Pressure-Suppression BWR Containments	N/A BWR Safety Topic
VI-2-C	Ice Condenser Containment	N/A to this unit containment design
VI-7-A-1	Increased Vessel Head Temperature	N/A this unit NSSS
VI-7-A-2	Upper Plenum Injection	N/A this unit NSSS
VI-7-A-4	Core Spray Nozzle Effectiveness	N/A BWR Safety Topic
VI-7.D	Long Term Cooling Passive Failures	N/A No current criteria
VI-9-A	Main Steam Line Isolation Seal System	N/A BWR Safety Topic
VI-10-B	Shared Engineer Safety Features	N/A to this unit site
VII-7	Acceptability of Swing Bus Design on BWR-4 Plants	N/A BWR Safety Topic
XV-11	Fuel Assembly Misloading	Review on a routine basis as part of reload
XV-13	Rod Drop Accident (BWR)	N/A BWR Safety Topic
Group I (BWR) DBE	All	N/A BWR DBE Review
Group VI DBE	Topic XV-13 (Only) Uncontrolled Rod Assembly Withdrawal at Power, Low Power Startup, Rod Drop Accident	N/A BWR DBE Review