

NRR-DMPSPeM Resource

From: Rankin, Jennivine
Sent: Friday, February 09, 2018 10:36 AM
To: 'Helen L Levendosky'; 'Danielle M Burgoyne'
Subject: DC Cook, Unit 2 - Request for Additional Information Regarding Reactor Vessel Internals Aging Management Program (EPID L-2017-LRO-0068)
Attachments: Final RAI.docx

Mrs. Levendosky,

By letter dated December 8, 2017 (Agencywide Documents Access and Management System Accession No. ML17346A683), Indiana Michigan Power Company (the licensee) submitted supplemental information to the U.S. Nuclear Regulatory Commission (NRC) regarding the reactor vessel internals aging management program. Specifically, the submittal focuses on 5 degraded baffle-edge bolts identified during the Donald C. Cook Nuclear Plant, Unit 2 cycle 23 refueling outage.

The NRC staff has reviewed the information provided and determined that additional information is required in order to complete its review. A draft request for additional information (RAI) was transmitted on January 30, 2018, and a clarification call was held on February 9, 2018. As agreed upon, please submit your response to the RAI by March 30, 2018. If you wish to alter the date of your response, please contact me at (301) 415-1530.

Please treat this e-mail as formal transmittal of the RAIs.

Thanks,
Jennie

Jennie Rankin, Project Manager
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Hearing Identifier: NRR_DMPS
Email Number: 154

Mail Envelope Properties (DM2PR09MB042964C7950FD97686293A4998F20)

Subject: DC Cook, Unit 2 - Request for Additional Information Regarding Reactor Vessel Internals Again Management Program (EPID L-2017-LRO-0068)
Sent Date: 2/9/2018 10:35:47 AM
Received Date: 2/9/2018 10:35:48 AM
From: Rankin, Jennivine

Created By: Jennivine.Rankin@nrc.gov

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Tracking Status: None
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Tracking Status: None

Post Office: DM2PR09MB0429.namprd09.prod.outlook.com

Files	Size	Date & Time
MESSAGE	1128	2/9/2018 10:35:48 AM
Final RAI.docx	27155	

Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received:

REQUEST FOR ADDITIONAL INFORMATION REGARDING
SUPPLEMENTAL INFORMATION REGARDING THE
REACTOR VESSEL INTERNALS AGING MANAGEMENT PROGRAM
DONALD C. COOK NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-316

EPID L-2017-LRO-0068

By letter dated December 8, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17346A683), Indiana Michigan Power Company (the licensee) submitted supplemental information to the U.S. Nuclear Regulatory Commission (NRC) regarding the reactor vessel internals aging management program. Specifically, the submittal focuses on 5 degraded baffle-edge bolts identified during the Donald C. Cook Nuclear Plant (CNP), Unit 2 cycle 23 refueling outage.

RAI-1

Condition 3 of the NRC staff's final safety evaluation (SE) of WCAP-17096-NP-A, Revision 2, "Reactor Internals Acceptance Criteria, Methodology and Data Requirements" (ADAMS Accession No. ML16061A187) requires licensees to submit to the NRC the plant specific failure modes and effects analysis (FMEA) within one year after the inspection that detects relevant conditions. In Enclosure 2 of the December 8, 2017, submittal, the licensee provided a proprietary FMEA of the baffle-edge bolts for CNP, Unit 2. Specifically, the licensee provided sections relevant to baffle-edge bolts from WCAP-18131-P, Revision 1, "Background and Technical Basis Supporting Engineering Flaw Acceptance Criteria for D.C. Cook Unit 2 Reactor Vessel Internals MRP-227-A Primary and Expansion Components."

The methodology and analysis section for baffle-edge bolts as described in WCAP-17096-NP-A (ADAMS Accession No. ML16279A320) states, in part, "[o]bservation of relevant conditions in the baffle edge bolts or locking bars would require a FMEA on the observed condition."

The NRC staff notes that the information provided in the December 8, 2017, submittal describes the process for baffle-edge bolt FMEA and the generic results of the FMEA, but does not take into account the specific "observed condition" of the baffle-edge bolts at CNP, Unit No. 2 as described in WCAP-17096-NP-A. Please provide the results of the baffle-edge bolt FMEA considering the as-found condition at CNP, Unit No. 2.