

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8706170167 DOC. DATE: 87/06/10 NOTARIZED: YES DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv 05000387
 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv 05000388
 AUTH. NAME AUTHOR AFFILIATION
 KEISER, H. W. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION
 BUTLER, W. R. Project Directorate I-2

SUBJECT: Forwards application for proposed Amends 98 & 51 to Licenses
 DPR-14 & DPR-22, respectively, revising Tech Spec
 4.8.2.1.d.2.b to increase load profiles for Batteries 1D612
 & 1D622.

DISTRIBUTION CODE: A001D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 244
 TITLE: OR Submittal: General Distribution

NOTES: 1cy NMSS/FCAF/PM. LPDR 2cys Transcripts. 05000387
 1cy NMSS/FCAF/PM. LPDR 2cys Transcripts. 05000388

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD1-2 LA THADANI, M	1 0 1 1	PD1-2 PD	5 5
INTERNAL:	ACRS	6 6	ARM/DAF/LFMB	1 0
	NRR/DEST/ADE	1 1	NRR/DEST/ADS	1 1
	NRR/DOEA/TSB	1 1	NRR/PHAS/ILRB	1 1
	OGC/HDS2	1 0	<u>REG FILE</u> 01	1 1
EXTERNAL:	EG&G BRUSKE, S	1 1	LPDR	2 2
	NRC PDR	1 1	NSIC	1 1
NOTES:		3 3		

Read w/out check



12-15-53

1

1

12-15-53 12-15-53 12-15-53



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

JUN 10 1987

Harold W. Keiser
Vice President-Nuclear Operations
215/770-7502

Director of Nuclear Reactor Regulation
Attention: Dr. W. R. Butler, Project Director
Project Directorate I-2
Division of Reactor Projects
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
PROPOSED AMENDMENT NO. 98 TO NPF-14 AND
PROPOSED AMENDMENT NO. 51 TO NPF-22
125V D.C. LOAD PROFILES
PLA-2867 FILES A17-20A, R41-2, S002

Dear Dr. Butler:

Pursuant to 10CFR50.90, Pennsylvania Power & Light Co. requests amendments in the form of Technical Specification changes, to Operating Licenses NPF-14 and NPF-22 for Susquehanna Steam Electric Station Units 1 and 2.

Specifically, we request Specification 4.8.2.1.d.2.b be modified to increase the load profiles for batteries 1D612 and 1D622. This is necessary to accommodate the installation of ATWS Alternate Rod Injection solenoid valves. Marked-up copies of the affected technical specification pages are enclosed.

The proposed changes do not:

- (1) Involve an increase in the probability or consequences of an accident previously evaluated. FSAR Section 8.3.2.1.1.4 states the station batteries have sufficient capacity without the charger, to independently supply the required loads for 4 hours. The Technical Specifications require the batteries be surveilled to a dummy load which is greater than the design loads. To demonstrate the batteries have adequate capacity to supply the proposed load profile, a calculation was performed by our Engineering department. This calculation - in part - is based on IEEE 485. In particular, cell sizing worksheets were prepared for the new loads. These worksheets determine the number of positive plates the specific battery must have to supply the given load profile. For the proposed profile, and considering worst case which is a battery electrolyte temperature of 60°F, the 1D612 and 1D622 batteries were demonstrated to have adequate capacity.
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated. As stated in Part(1), the batteries have sufficient capacity for the proposed load profile, thus enabling them to

8706170167 870610
PDR ADDCK 05000387
PDR

Rec'd w/out charge

Acc'd
1/1

THE UNIVERSITY OF CHICAGO
DIVISION OF THE PHYSICAL SCIENCES
DEPARTMENT OF CHEMISTRY
5780 SOUTH CAMPUS DRIVE
CHICAGO, ILLINOIS 60637

PROFESSOR [Name]
[Address]
[City, State, Zip]

Dear Professor [Name]:

I am writing to you regarding the [topic] of your [document].

Sincerely,
[Name]

[Detailed body text, mostly illegible due to low contrast and scan quality.]

Very truly yours,
[Name]

JUN 10 1987

2

FILES A17-20,R41-2 PLA-2867
Dr. W. R. Butler

perform their intended function. Any postulated accident resulting from this change is bound by previous analysis.

- (3) Involve a reduction in the margin of safety. In accordance with IEEE 450, the rated battery capacity is 25 percent greater than required. This margin allows replacement of the battery when its capacity has decreased to 80 percent of its rated capacity (100 percent of design load). This margin has been maintained and included as part of the calculation referred to in Part(1).

We request these amendments be approved and made effective at the conclusion of the Unit 1 3rd refueling and inspection outage. (Currently scheduled to commence September 12, 1987 and end November 13, 1987).

Pursuant to 10CFR170.22, the appropriate fee is enclosed.

If you have any questions, please contact D. J. Walters at (215)770-6536.

Very truly yours,



H. W. Keiser
Vice President - Nuclear Operations

cc: ~~NRC Document Control Desk (original)~~
NRC Region I
Mr. L. R. Plisco, NRC Resident Inspector
Mr. M. C. Thadani, NRC Project Manager

1950

...

...

...

...

...

...

...