

NOTICE OF VIOLATION

Tennessee Valley Authority
Browns Ferry Units 2 and 3

Docket Nos. 50-260, 50-296
License Nos. DPR-52, DPR-68

During an NRC inspection conducted on March 17 - April 27, 1996, three violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG 1600, the violations are listed below:

- A. 10 CFR 50, Appendix R, Criterion III.G.2 requires where cables that could prevent operation or cause maloperation due to hot shorts, open circuits, or shorts to ground, of redundant trains of systems necessary to achieve and maintain hot shutdown conditions are located within the same fire area outside of primary containment, one of the redundant trains shall be ensured to remain free of fire damage by use of a fire barrier enclosure or physical separation.

Contrary to the above, fire barrier enclosure or physical separation was not applied to cables associated with the following equipment required to achieve and maintain hot shutdown conditions: 2-LI-3-58A (reactor water level indication), 2-PI-3-74 (reactor pressure indication), 2-TI-64-161 (suppression chamber temperature indication), 2-FCV-73-16 (high pressure coolant injection system steam supply valve). These conditions have existed since the restart of Unit 2 in May, 1991.

This is a Severity Level IV violation (Supplement I), applicable to Unit 2 only.

- B. Technical Specification 3.10.A.1 requires that the required refueling equipment interlocks shall be operable during in-vessel fuel movement with the equipment associated with the interlocks except as specified in 3.10.A.6 and 3.10.A.7.

Contrary to the above, on March 23, 1996, the refueling equipment interlocks were not operable during in-vessel fuel movement nor were conditions specified in 3.10.A.6 or 3.10.A.7 met.

This is a Severity Level IV violation (Supplement I), applicable to Unit 2 only.

- C. TS 6.8.1.1.a requires that written procedures shall be established, implemented, and maintained covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February, 1978. Paragraph 9 of Appendix A of Regulatory Guide 1.33 recommends procedures for performing maintenance that can affect safety related equipment.

Enclosure 1



0-TI-264, Scaffolding and Temporary Platforms, step 7.3.15.1, states that no scaffolding/scaffolding access and egress points or platforms shall be erected closer than 3 feet from any access point of electrical boards, electrical panels or panels with electrical devices which will block entrance into the panel or possibly cause inadequate protection of switches from being repositioned, unless specifically permitted by the Shift Operations Supervisor as indicated by signature on the Scaffold Erection Checklist.

Contrary to the above, on February 26, 1996, plant procedure 0-TI-264 was not properly implemented when scaffolding was erected less than 3 feet from the 3C Emergency Diesel Generator field flashing breaker. On February 29, 1996, this resulted in the 3C Emergency Diesel Generator being inoperable due to an inadvertent mispositioning of the field flashing breaker.

This is a Severity Level IV violation (Supplement I), applicable to Unit 3 only.

In reference to violations B and C, pursuant to the provisions of 10 CFR 2.201, Tennessee Valley Authority is hereby required to submit in a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). The responses should include for each violation: (1) the reason for the violation or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be taken. Where good cause is shown, consideration will be given to extending the response time.

In reference to violation A, the NRC has concluded that information regarding the reason for the violation, the corrective actions taken and planned to correct the violation and prevent recurrence and the date full compliance will be achieved is already adequately addressed in LER 260/96001, 10 CFR Part 50 Appendix R Noncompliance Results in Plant Being Outside Its Design Basis and Being In A Condition Not Covered By Plant Operating Instructions. However, you are required to submit a written statement or explanation pursuant to 10 CFR 2.201 if the description therein does not accurately reflect your corrective actions or your position.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be



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3

placed in the PDR, and provide the legal basis to support your request for withholding the information from the public.

Dated at Atlanta, Georgia
this 21st day of May 1996

