OPERATING DATA REPORT

DOCKET NO.: 50-220
DATE: 2/13/91
PREPARED BY: D. E. Coleman
TELEPHONE: (315) 349-2558

OPERATING STATUS

1. Unit Name: Nine Mile Point Unit #1
2. Reporting Period: 1/1/91 through 1/31/91
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 642
5. Design Electrical Rating (Net MWe): 625
6. Maximum Dependable Capacity (Gross MWe): 635
7. Maximum Dependable Capacity (Net MWe): 615
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
   Nameplate rating (Gross MWe) corrected to 642 from 645 (755 MVA x .85)

Notes

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	744.0	744.0	187,393.2
12. Number of Hours Reactor Was Critical	549.5	549.5	119,149.6
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,204.2
14. Hours Generator On-Line	533.8	533.8	115,696.3
15. Unit Reserve Shutdown Hours	0.0	0.0	20.4
16. Gross Thermal Energy Generated (MWH)	917,214.0	917,214.0	193,459,892.0
17. Gross Electrical Energy Generated (MWH)	316,069.0	316,069.0	64,118,986.0
18. Net Electrical Energy Generated (MWH)	306,160.0	306,160.0	62,112,000.0
19. Unit Service Factor	71.7	71.7	61.7
20. Unit Availability Factor	71.7	71.7	61.7
21. Unit Capacity Factor (Using MDC Net)	66.9	66.9	54.3
22. Unit Capacity Factor (Using DER Net)	65.8	65.8	53.5
23. Unit Forced Outage Rate	28.3	28.3	25.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
    Midcycle Outage for surveillance testing 3 March 91 through 13 April 91.

25. If Shut Down At End of Report Period, Estimated Date of Startup: _____
26. Unit is Test Status (Prior to Commercial Operation): Forecast Achieved

INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

DOCKET NO.: 50-220
DATE: 2/13/91
PREPARED BY: D. E. Coleman
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MONTH January 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	609
2	0	18	601
3	0	19	605
4	0	20	608
5	0	21	603
6	0	22	606
7	0	23	604
8	0	24	609
9	21	25	565
10	303	26	578
11	501	27	583
12	571	28	593
13	600	29	590
14	598	30	595
15	602	31	605
16	606		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-220
UNIT NAME: NMP#1
DATE: 2/13/91
PREPARED BY: D. E. Coleman
TELEPHONE: (315) 349-2558

REPORT MONTH - January 1991

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
90-04	901229	F	210.2	A	1/3	90-19			Continued - While performing a forced reactor shutdown due to a failure of MSIV 01-02, a full reactor scram occurred. The reactor scram was caused by a spike on IRM 12 (Channel 11 RPS) with a half scram already in on channel 12 RPS due to the failure of MSIV 01-02.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I-Same Source

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT #1

NARRATIVE OF OPERATING EXPERIENCE

The Station operated during the month of January 1991 with a Unit Availability Factor of 71.7% and a Net Design Electrical Capacity Factor of 65.8%. There were no challenges to Electromatic Relief Valves. Reductions in Capacity Factor were due to the forced outage which started on December 29, 1990. While shutting down, due to a failure of MSIV 01-02, a spike occurred on IRM 12 causing the reactor to scram. The unit was synchronized to the grid on January 9, 1991 at 1810. Other reductions in Capacity Factor were due to weekly control rod exercising and the inability to obtain 100% CTP due to turbine steam flow limitation. Also, load was reduced to stabilize Turbine oil pressure.

CLASS I WORK - MECHANICAL MAINTENANCE - January 1991

See attached printout.

CLASS I WORK - INSTRUMENTS AND CONTROLS - January 1991

See attached printout.

CLASS I WORK - ELECTRICAL MAINTENANCE - January 1991

See attached printout.

NINE MILE POINT UNIT ONE
Mechanical Maintenance - COMPLETED SAFETY RELATED WRs FOR JANUARY

02-06-91
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WR Number	EPN Number	TITLE	Description	Corrective Action
W185292	01-01	INBOARD MSIV	THE INBOARD MSIV 01-01 IN THE DRYWELL IS LEAKING - PLEASE ADJUST PACKING REPAIR AS NECESSARY	REMOVED OLD PACKING INSTALLED NEW - 9 RINGS
W185675	01-11	ELECTROHATIC BLOCKING VALVE 01-11	LEAKAGE HAS BEEN DETECTED FROM PACKING AREA VALVE IS LOCATED IN DRYWELL ELEVATION 259	REMOVED OLD PACKING CLEANED STEM AND STUFFING BOX AND REPACKED WITH 9 RINGS OF GRAFOIL TAPE 1 3/4 INCHES X 2 3/4 INCHES X 1/2 INCH
W185672	01-MS9	INSIDE DRYWELL HYDRO TEST LINE VALVE	TRENDING OF DRYWELL LEAKAGE INDICATES THAT MS9 IS LEAKING VALVE IS LOCATED IN DRYWELL ELEVATION 313 SOUTH OF H EAST OF 8 VALVE IS A 2 INCH STAINLESS STEEL GLOBE	UPON INSPECTION OF DRYWELL IT WAS FOUND THAT THIS VALVE WAS NOT LEAKING
W185677	32-NG03B	REACTOR RECIRCULATION PUMP OUTLET IV	LEAKAGE DETECTED FROM PACKING AREA VALVE LOCATED IN DRYWELL ELEVATION 237 AZM 90 DEGREES	ADJUSTED ALL THREE PACKING GLANDS
W185478	38-13	SHUTDOWN COOLING IV	PACKING LEAK ADJUST PACKING	TOOK UP ON PACKING FOLLOWER PER SSS INSTRUCTIONS
W185894	39-12R	EMERGENCY CONDENSER DRAIN LINE ISOLATION VALVE	VALVE IS LEAKING FROM BONNET AREA DISASSEMBLE AND REPLACE GASKET LOCATED IN ECIV ROOM 28 RX BLDG	REPLACED BONNET GASKET 95-36-877 AND TORQUE BONNET TO 90 FOOT POUNDS
W163649	40-12	TOPPING PUMP DISCHARGE VALVE	VALVE PACKING LEAKS REPAIR AS NECESSARY	TIGHTENED PACKING GLAND
W186189	57-A0-202	JET ASSIST VALVE TO COND DEMINS A0-202	VALVE HAS A BLOWN DIAPHRAM LEAKING WATER LOC TB 261 1ST AND BRIDGE SW OVERHEAD	REPLACED DIAPHRAGM WITH 93-39-696
W185179	70-NA	PIPE CAP ON DRAIN LINE DOWNSTREAM OF RCLC 716	CAP IS MISSING ON DRAIN LINE DOWNSTREAM OF RCLC 716 - REPLACE CAP LOCATED IN AUX CLEANUP PUMP ROOM IN REACTOR BUILDING 261	INSTALLED PIPE NIPPLE AND PIPE CAP TO DRAIN VALVE
W185264	201-DRYWELL EMERGENCY HATCH	EMERGENCY AIR LOCK DRYWELL INNER DOOR SEAL	PLEASE REPLACE DRYWELL EMERGENCY AIRLOCK INNER DOOR SEAL - LOCATED AT REACTOR BUILDING 237 AT EMERGENCY AIRLOCK INNER DOOR	OLD GASKET REMOVED SURFACES CLEANED NEW GASKET INSTALLED
W185678	201-DRYWELL HATCHES	DRYWELL PERSONNEL HATCH SEAL REPLACEMENT	REPLACE DOOR SEAL INNER DRYWELL HATCH DOOR - REACTOR BUILDING 237 NORTH	CHANGE RUBBER SEAL
W185182	201.8-96	N2 STORAGE TANK 12 250 POUND RELIEF VALVE	12 N2 STROAGE TANK 250 POUND RELIEF VALVE IS NOT RELIEVING AT 250 PSIG - PLEASE REPAIR LOCATED NORTH OF REACTOR BUILDING AND WEST OF SCREEN HOUSE OUTSIDE	REPLACED N2 STORAGE TANK 250 POUND RELIEF VALVE 201.8-96 SYMBOL NUMBER 93-34-600 WITH NEW VALVE
W168354	210-01	EMERGENCY VENT FAN 11 AND 12 CONTROL RM VENT	WORN BELLOWS ON SUCTION SIDE TB 300	CAN NOT BE PATCHED - MUST BE REPLACED - WR 185645 WRITTEN
W185136	301-38-35	HCU 38-35 SCRAM OUTLET VALVE 127	SCRAM OUTLET VALVE 127 FOR HCU 38-35 IS LEAKING BY AS INDICATED BY INCREASING SCRAM DISCHARGE VOLUME HOLDING TANK LEVEL - LOCATION REACTOR BUILDING 237	NO WORK REQUIRED BY MECHANICAL MAINTENANCE - 910105 - I AND C COMPLETED

NINE MILE POINT UNIT ONE
Mechanical Maintenance - COMPLETED SAFETY RELATED WRs FOR JANUARY

WR Number	EPN Number	TITLE	Description	Corrective Action
W185265	305-125	CRD HCU 46-27 PISTON	CONTROL ROD DRIVE HYDRAULIC CONTROL UNIT 46-27 PISTON IS LEAKING EXCESSIVLY ALARMS ABOUT EVERY 2 1/2 HOURS PLEASE REPAIR OR REPLACE LOCATED REACTOR BUILDING 237 WEST DEF TAG 24314	REPLACE WATER BOTTLE ACCUMULATOR WITH NEW - WR 185623 HAS BEEN GENERATED TO REBUILD OLD ACCUMULATOR

NINE MILE POINT UNIT ONE
Instrument & Controls - COMPLETED SAFETY RELATED WRs FOR JANUARY

WR Number	EPN Number	TITLE	Description	Corrective Action
W186941	32-IA72A	CHANNEL 11 APRM FLOW SUMMER	11 APRM FLOW SUMMER IS READING HIGH REPAIR AS NECESSARY REPLACE FLOW SUMMER	REPLACED FLOW SUMMER WITH ANOTHER SUMMER
W182485	33-95 33-96 33-97	AREA TEMPERATURE MONITORS FOR CLEANUP SYSTEM	IN SUPPORT OF MAINTENANCE SUPPORT PROCEDURE GROUP REMOVE BACK COVER FROM ALL THREE 3 ARE TEMPERATURE MONITORS TM-400 IN ORDER TO PERFORM WIRING INSPECTIONS	REMOVED COVERS TO ALLOW WIRING INSPECTION - REPLACED COVERS WHEN INSPECTIONS COMPLETED
W187850	36-ATWS	RELAY RP12 CONTACTS T4-H3	WHILE PERFORMING N1-ISP-036-M009 A MEASUREMENT WAS TAKEN ACROSS TERMINALS TR-H3 ON RP 12 RELAY OF 1.24 OHMS THIS VALUE IS SUPOSE TO BE 0.0 OHMS OR LESS THAN 1.0 OHMS AS STATED IN THE PROCEDURE - LOCATION AUX CONTROL ROOM CABINET 1S48	REPLACED RELAY RPT-12
W187851	36-ATWS	I/V CONVERTER NEST 500 SLOT 2	WHILE PERFORMING N1-ISP-036-M009 A AS LEFT VOLTAGE OF APPROXIMATELY MINUS 14 VOLTS WAS MEASURED ACROSS 523 PLUS AND 523 MINUS THIS CONVERTERS NORMAL OUTPUT IS 0 TO 10 VOLTS - LOCATION AUX CONTROL ROOM CABINET 1S48	REPLACED IV WITH NEW ONE FROM STORES
W182486	38-62	AREA TEMPERATURE MONITOR FOR SHUTDOWN COOLING SYSTEM	IN SUPPORT OF MAINTENANCE SUPPORT PROCEDURE GROUPREMOVE BACK COVER FROM AREA TEMPERATURE MONITOR TM-400 IN ORDER TO PERFORM WIRING INSPECTIONS	REMOVED COVER TO ALLOW WIRING INSPECTION - REPLACED COVER WHEN INSPECTION COMPLETED
W185480	92-IRM-12	12 IRM DRIVE UNITS	12 IRM DRIVE UNIT LIGHTS ARE NOT ILLUMINATED ON E CONSOLE IN UNIT 1 CONTROL ROOM - TROUBLESHOOT	RESET RFH RELAY
W185481	92-IRM-15	15 IRM DRIVE UNITS	15 DRIVE UNITS LIGHTS ARE NOT ILLUHINATED ON E CONSOLE IN UNIT 1 CONTROL ROOM	RESET RFH RELAY
W187820	92-RH-01G	IRM 17	WHILE PERFORMING N1-ISP-092-V205 STEP 7.7.48 THE CONSOLE E INOPERABLE LAMP FAILED TO ILLUMINATE PLEASE TROUBLESHOOT	TCN TO PROCEDURE - SYSTEM WORKS AS DESIGNED - NO WORK PERFORMED
W187694	201.2-217	11 H2 AUTO CALIBRATION UNIT	THE OUT OF RANGE SPAN LIGHT IS ON PLEASE RECALIBRATE - P AND ID 26939 TURBINE BUILDING 291 WEST	RECALIBRATED H2 MONITOR
W185603	201.2-217A 201.2-218A 201.2-330A 201.2-331A	H2-02 AUTO CALIBRATION	PLEASE SET AUTO CALIBRATION UNITS TO SAMPLE AND HOLD MODE TO PREVENT ANNUNCIATORS FROM COMING IN AFTER AUTO CAL SEQUENCE - C-27003-C C27004-C - TURBINE BUILDING 291 NORTH WALL	MADE ADJUSTMENT TO AUTO CALIBRATION UNIT
W186150	NEU-04-33A	LPRM 04-33A	LPRM 04-33A WOULD NOT PRODUCE A READING EVEN WITH FLUX AMP STRAPPED TO MAXIMUM GAIN DURING N1-RPSP-7 PLEASE PERFORM TDR AND REPAIR - REPLACES W186121	NO WORK PERFORMED TDR SHOWS A BAD DETECTOR AND I/V CURVE SHOWS OPEN - DETECTOR TO BE REPLACE IN FUTRUE AS DIRECTED BY REACTOR ANALYSTS
W185279	NEU-33I/IRM12	IRM 12 POSITION SWITCH - IN DRYWELL	LIMIT SWITCH 33I IRM 12 APPARENTLY HAS FAILED OPEN - PLEASE REPAIR OR REPLACE - PRINT C-19438-C SHEET 5	REPLACED CONNECTOR AT VESSEL END
W187762	NEU-F1	FUSE 10 F1 POWER FOR BYPASS OF 11 SIDE	FUSE BLOWN PLEASE CHECK OUT AND REPLACE FUSE CANNOT BYPASS 11 SIDE NEUTRON INSTRUMENTS	REPLACED FUSE
W187726	PHS-X-5B	NITROGEN PURGE LINE CONNECTION TO PENETRATION X-5B	PLEASE REPLACE CONNECTION NUT AND FERRULE AT PENETRATION X-5B LOCATION REACTOR BUILDING 281 ECIV ROOM	REPLACED NUT AND FERRULE

NINE MILE POINT UNIT ONE
Instrument & Controls - COMPLETED SAFETY RELATED WRs FOR JANUARY

WR Number	EPN Number	TITLE	Description	Corrective Action
W185467	RPS-RE16A AND RE16B	REACTOR 600 PSIG PRESSURE BYPASS CIRCUIT	TROUBLESHOOT REACTOR EQUAL TO OR LESS THAN 600 PSIG PRESSURE BYPASS CIRCUIT	CALIBRATED SWITCHES PER N1-ISP-001-R005 PARTIAL - INDICATOR NOT DONE - MONITORED TRIP RELAYS 11K39 AND 12K39 - NO PROBLEMS NOTED

NINE MILE POINT UNIT ONE
Electrical Maintenance - COMPLETED SAFETY RELATED WRs FOR JANUARY

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WR Number	EPN Number	TITLE	Description	Corrective Action
W185257	01-02	121 MAIN STEAM LINE ISOLATION VALVE 01-02	DURING MAIN STEAM FEEDWATER VALVE TESTING N1-ST-Q26 MAIN STEAM LINE IV 121 01-02 BREAKER TRIPPED LOCATED PB 171B REACTOR BUILDING 261 EAST - TROUBLESHOOT	TROUBLESHOOT SYSTEM PER ATTACHED AP-5.4.2 - MOTOR BURNED OUT REPLACED MOTOR - VALVE WAS STUCK COULD NOT OPERATE MANUALLY
W185683	RPS-FUSES	REACTOR TRIP BUS 131 AND 141 FUSES	REPLACE EXISTING REACTOR TRIP BUS FUSES IN AUX CONT CABINET 1S53 BUS 131 AND AUX CONT CABINET 1S55 BUS 141 WITH GOULD SHAWMUT OT FUSES PER DCR N1-90-001LS368	REPLACED FUSES PER DCR N1-90-001LS368
W185611	RPS-RPS911K3 7B	ROD BLOCK AND RECIRC SCOOP TUBE LOCK HFA RELAY	HFA RELAY 11K37B IS EXTREMELY NOISY ESPECIALLY WHEN COMPARED TO OTHER HFA RELAYS - PLEASE INVESTIGATE - IF PROBLEM FOUND REPAIR OR REPLACE AS REQUIRED - LOCATED IN H PANEL	DID TROUBLESHOOTING AS REQUIRED FOUND LOOSE COIL SCREWS IN RELAY - REFERRED PROBLEM TO METER AND TEST