

UNIT 1
EFFECTIVE PAGE LIST
TECHNICAL SPECIFICATION 306

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- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility."

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- (1) The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal. †



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- (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications. †
- (3) In the operation of the facility, the licensee shall, pursuant to the Federal Water Pollution Control Act Amendments of 1972 (Public Law 92-500), comply with all applicable thermal water quality standards of the State of Alabama and the United States. †
- (4) Deleted. †
- (5) The facility may be modified by plugging the bypass flow holes in the lower core support plate as described in Browns Ferry Nuclear Plant Units 1 and 2 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations (NEDC-21091), October 1975. The reactor shall not be operated with the plugs installed in the lower core support plate bypass flow holes without further authorization by the NRC. †
- (6) The facility may be modified by drilling bypass flow holes in Type 2 and Type 3 fuel assemblies as described in NEDO-21091, "Browns Ferry Nuclear Plant, Units 1 & 2 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations;" and NEDE-21156, "Supplemental Information for Plant Modification to Eliminate Significant In-Core Vibrations," dated January 1976. †



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- (7) The facility may be modified as described in 'Browns Ferry Nuclear Plant Units 1 and 2 Emergency Core Cooling Systems Low Pressure Coolant Injection Modifications for Performance Improvement (December 1975)' submitted by application dated December 1, 1975 and supplements dated February 12, 1976, March 24, 1976, March 30, 1976, May 21, 1976, June 11, 1976, and July 21, 1976. †
- (8) The licensee shall maintain in effect and fully implement all provisions of the Commission approved physical security plan including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plan, which contains information protected under 10 CFR 73.21, is entitled "Browns Ferry Nuclear Plant Physical Security Plan," dated May 15, 1982 (TVA letter dated June 11, 1982) and revisions submitted by TVA letters dated August 31, 1982 and October 19, 1982. †
- Notwithstanding the statement in Section 9.1 of the physical security plan, the licensee shall maintain positive access control over containment in accordance with the requirements of 10 CFR 73.55(d)(8).
- (9) The facility may be modified as described in 'Browns Ferry Nuclear Plant Units 1 and 2 Emergency Core Cooling Systems Low Pressure Coolant Injection Modifications For Performance Improvement (October 1977)' submitted by letter dated December 28, 1977 and supplemented by letter dated December 13, 1978.
- (10) The licensee shall follow all provisions of the NRC approved Guard Training & Qualification Plan, including amendments and changes made pursuant to 10 CFR 50.54(p). The approved Guard Training & Qualification Plan is identified as "Browns Ferry Nuclear Power Station Guard Training & Qualification Plan," dated August 17, 1979, as revised by pages dated January 24, 1980, May 21, 1980, October 1, 1980, and March 9, 1981 and as may subsequently be revised in accordance with 10 CFR 50.54(p). The Guard Training & Qualification Plan shall be followed, in accordance with 10 CFR 73.55(b), 60 days after the date of this amendment.

- (11) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Browns Ferry Physical Security Plan", with revisions submitted through May 24, 1988, and "Browns Ferry Security Personnel Training and Qualification Plan", with revisions submitted through April 16, 1987; and "Browns Ferry Safeguards Contingency Plan", with revisions submitted through June 27, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- (12) The licensee is authorized to temporarily store low-level radioactive waste in an existing covered pavilion that is situated outside the security fence, as presently located, but inside the site exclusion area. The total amount of low-level waste to be stored shall not exceed 1320 curies of total activity. This authorization expires two years from the effective date of this amendment¹ and is subject to all the conditions and restrictions in TVA's application dated January 21, 1980.
- (13) Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for BFN as approved in the SER dated _____ (and Supplements dated _____) subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- D. This amended license is effective as of the date of issuance and shall expire midnight on December 20, 2013.

FOR THE ATOMIC ENERGY COMMISSION

S/ A. Giambusso
A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Date of Issuance: DEC 20 1973

BFN
Unit 1

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3.11/4.11 FIRE PROTECTION SYSTEMS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

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6.2.2 (Cont.)

- d. Two licensed reactor operators shall be in the control room during any cold startups, while shutting down the reactor, and during recovery from unit trip. In addition, a person holding a senior operator license shall be in the control room for that unit whenever it is in an operational mode other than cold shutdown or refueling.
- e. A Health Physics Technician* shall be present at the facility at all times when there is fuel in the reactor.
- f. A person holding a senior operator license or a senior operator license limited to fuel handling, shall be present during alteration of the core to directly supervise the activity and during this time shall not be assigned other duties.
- g. Deleted.

*The Health Physics Technician may be absent from the facility for a period of time not to exceed 2 hours provided immediate action is taken to fill the required position.



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6.5.1.6 (Cont.)

1. Review of reportable events, unusual events, operating anomalies, and abnormal performance of plant equipment.
- m. Investigate reported or suspected incidents involving safety questions or violations of the Technical Specifications.
- n. Review of unit operations to detect potential hazards to nuclear safety. Items that may be included in this review are NRC inspection reports, QA audit, NSRB audit results, American Nuclear Insurer (ANI) inspection results, and significant corrective action reports (CARs).
- o. Performance of special reviews, investigations, or analysis, and report thereon as requested by the Plant Manager or the Nuclear Safety Review Board.
- p. Review of the Fire Protection Program, "Fire Protection Report, Volume-1", and implementing procedures, "Safe Shutdown Instructions", and the submittal of PORC approved changes to Nuclear Safety Review Board.



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UNIT 2
EFFECTIVE PAGE LIST
TECHNICAL SPECIFICATION 306

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- (2) Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time source and special nuclear material as reactor fuel in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility."
- C. This licensee shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal. †
 - (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications. †

- (3) In the operation of the facility, the licensee shall, pursuant to the Federal Water Pollution Control Act Amendments of 1972 (Public Law 92-500), comply with all applicable thermal water quality standards of the State of Alabama and the United States.
- (4) The licensee is hereby granted an exemption from the requirements of General Design Criterion 4 with respect to high energy pipes outside containment in accordance with the condition set forth in the Technical Specifications, Section 3.6.G.2 which requires completion of those items listed in 'Concluding Report on the Effects of Postulated Pipe Failure Outside of Containment for the Browns Ferry Nuclear Plant Units 2 and 3' and related to Unit 2 prior to startup of Unit 2 following the first refueling outage."
- (5) The facility may be modified by plugging the bypass flow holes in the lower core support plate as described in Browns Ferry Nuclear Plant Units 1 and 2 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations (NEDC-21091), October 1975. The reactor shall not be operated with the plugs installed in the lower core support plate bypass flow holes without further authorization by the NRC."
- (6) Deleted. (Numbering changed from 2.C.5.a)
- (7) The facility may be modified by drilling bypass flow holes in Type 2 and Type 3 fuel assemblies as described in NEDO-21091, "Browns Ferry Nuclear Plant, Units 1 & 2 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations; and NEDE-21156, "Supplemental Information for Plant Modification to Eliminate Significant In-Core Vibrations," dated January 1976.
- (8) The facility may be modified as described in 'Browns Ferry Nuclear Plant Units 1 and 2 Emergency Core Cooling Systems Low Pressure Coolant Injection Modifications for Performance Improvement (December 1975)' submitted by application dated December 1, 1975 and supplements dated February 12, 1976, March 24, 1976, March 30, 1976, May 21, 1976, June 11, 1976, and July 21, 1976."



②

- (9) The licensee shall maintain in effect and fully implement all provisions of the Commission approved physical security plan including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plan, which contains information protected under 10 CFR 73.21, is entitled "Browns Ferry Nuclear Plant Physical Security Plan," dated May 15, 1982 (TVA letter dated June 11, 1982) and revisions submitted by TVA letters dated August 31, 1982 and October 19, 1982.

Notwithstanding the statement in Section 9.1 of the physical security plan, the licensee shall maintain positive access control over containment in accordance with the requirements of 10 CFR 73.55(d)(8).

- (10) The facility may be modified as described in 'Browns Ferry Nuclear Plant Units 1 and 2 Emergency Core Cooling Systems Low Pressure Coolant Injection Modifications For Performance Improvement (October 1977)' submitted by letter dated December 28, 1977 and supplemented by letter dated December 13, 1978.
- (11) The licensee shall follow all provisions of the NRC approved Guard Training & Qualification Plan, including amendments and changes made pursuant to 10 CFR 50.54(p). The approved Guard Training & Qualification Plan is identified as "Browns Ferry Nuclear Power Station Guard Training & Qualification Plan," dated August 17, 1979, as revised by pages dated January 24, 1980, May 21, 1980, October 1, 1980, and March 9, 1981 and as may subsequently be revised in accordance with 10 CFR 50.54(p). The Guard Training & Qualification Plan shall be followed, in accordance with 10 CFR 73.55(b), 60 days after the date of this amendment.
- (12) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Browns Ferry Physical Security Plan", with revisions submitted through May 24, 1988, and "Browns Ferry Security Personnel Training and Qualification Plan", with revisions submitted through April 16, 1987; and "Browns Ferry Safeguards Contingency Plan", with revisions submitted through June 27, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- (13) The licensee is authorized to temporarily store low-level radioactive waste in an existing covered pavilion that is situated outside the security fence, as presently located, but inside the site exclusion area. The total amount of low-level waste to be stored shall not exceed 1320 curies of total activity. This authorization expires two years from the effective date of this amendment and is subject to all the conditions and restrictions in TVA's application dated January 21, 1980.
- (14) Commission Order dated March 25, 1983 is modified as follows: in Attachment 1, for item II.F.1.1 and II.F.1.2 change "12/31/84" to "Prior to startup in Cycle 6."
- (15) Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for BFN as approved in the SER dated _____ (and Supplements dated _____) subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

D. This amended license is effective as of the date of issuance and shall expire midnight on June 28, 2014.

FOR THE ATOMIC ENERGY COMMISSION

S/ A. Giambusso
A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:
Appendices A & B - Technical
Specifications

Date of Issuance: JUN 28, 1974

BFN
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E. Spent Fuel Cask	3.10/4.10-9
F. Spent Fuel Cask Handling-Refueling Floor . . .	3.10/4.10-10
3.11/4.11 Deleted.	3.11/4.11-1
5.0 Major Design Features	5.0-1
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6.2.A	Minimum Shift Crew Requirements	6.0-4

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3.11/4.11 FIRE PROTECTION SYSTEMS

LIMITING CONDITIONS FOR OPERATION

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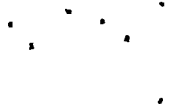
6.2.2 (Cont.)

- d. Two licensed reactor operators shall be in the control room during any cold startups, while shutting down the reactor, and during recovery from unit trip. In addition, a person holding a senior operator license shall be in the control room for that unit whenever it is in an operational mode other than cold shutdown or refueling.
- e. A Health Physics Technician* shall be present at the facility at all times when there is fuel in the reactor.
- f. A person holding a senior operator license or a senior operator license limited to fuel handling, shall be present during alteration of the core to directly supervise the activity and during this time shall not be assigned other duties.
- g. Deleted.

*The Health Physics Technician may be absent from the facility for a period of time not to exceed 2 hours provided immediate action is taken to fill the required position.

6.5.1.6 (Cont.)

- e. Review of all proposed changes to the Technical Specifications.
- f. Review of safety evaluation for proposed tests or experiments to be completed under the provisions of 10 CFR 50.59
- g. Review proposed changes to the Radiological Effluent Manual.
- h. Review adequacy of the Process Control Program and Offsite Dose Calculation Manual at least once every 24 months.
- i. Review changes to the radwaste treatment systems.
- j. Review of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendation, and disposition of the corrective action to prevent recurrence to the Senior Vice President, Nuclear Power, and to the Nuclear Safety Review Board.
- k. Review of all safety evaluations for modifications to structures, systems or components that affect nuclear safety to verify that such actions did not constitute an unreviewed safety question as defined in 10 CFR 50.59, or requires a change to these Technical Specifications.



6.5.1.6 (Cont)

- l. Review of reportable events, unusual events, operating anomalies, and abnormal performance of plant equipment.
- m. Investigate reported or suspected incidents involving safety questions or violations of the Technical Specifications.
- n. Review of unit operations to detect potential hazards to nuclear safety. Items that may be included in this review are NRC inspection reports, QA audit, NSRB audit results, American Nuclear Insurer (ANI) inspection results, and significant corrective action reports (CARs).
- o. Performance of special reviews, investigations, or analysis, and report thereon as requested by the Plant Manager or the Nuclear Safety Review Board.
- p. Review of the Fire Protection Program, "Fire Protection Report, Volume-1", and implementing procedures, "Safe Shutdown Instructions", and the submittal of PORC approved changes to Nuclear Safety Review Board.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire plant staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in site equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months.
- e. The Site Radiological Emergency Plan and implementing procedures at least once every 12 months.
- f. The Plant Physical Security Plan and implementing procedures at least once every 12 months.
- g. Any other area of site operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Power.
- h. The fire protection programmatic controls including the implementing procedures at least once per 24 months.

6.8 PROCEDURES/INSTRUCTIONS AND PROGRAMS

6.8.1 PROCEDURES

6.8.1.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Limitations on the amount of overtime worked by individuals performing safety-related functions in accordance with NRC Policy statement on working hours (Generic Letter No. '82-12).
- c. Surveillance and test activities of safety-related equipment.
- d. Security plan implementation.
- e. Emergency plan implementation.
- f. Fire Protection Program implementation.
- g. Radiological Effluent Manual implementing procedures.
- h. Process Control Program (PCP).
- i. Offsite Dose Calculation Manual.
- j. Administrative procedures which control technical and cross-disciplinary review.

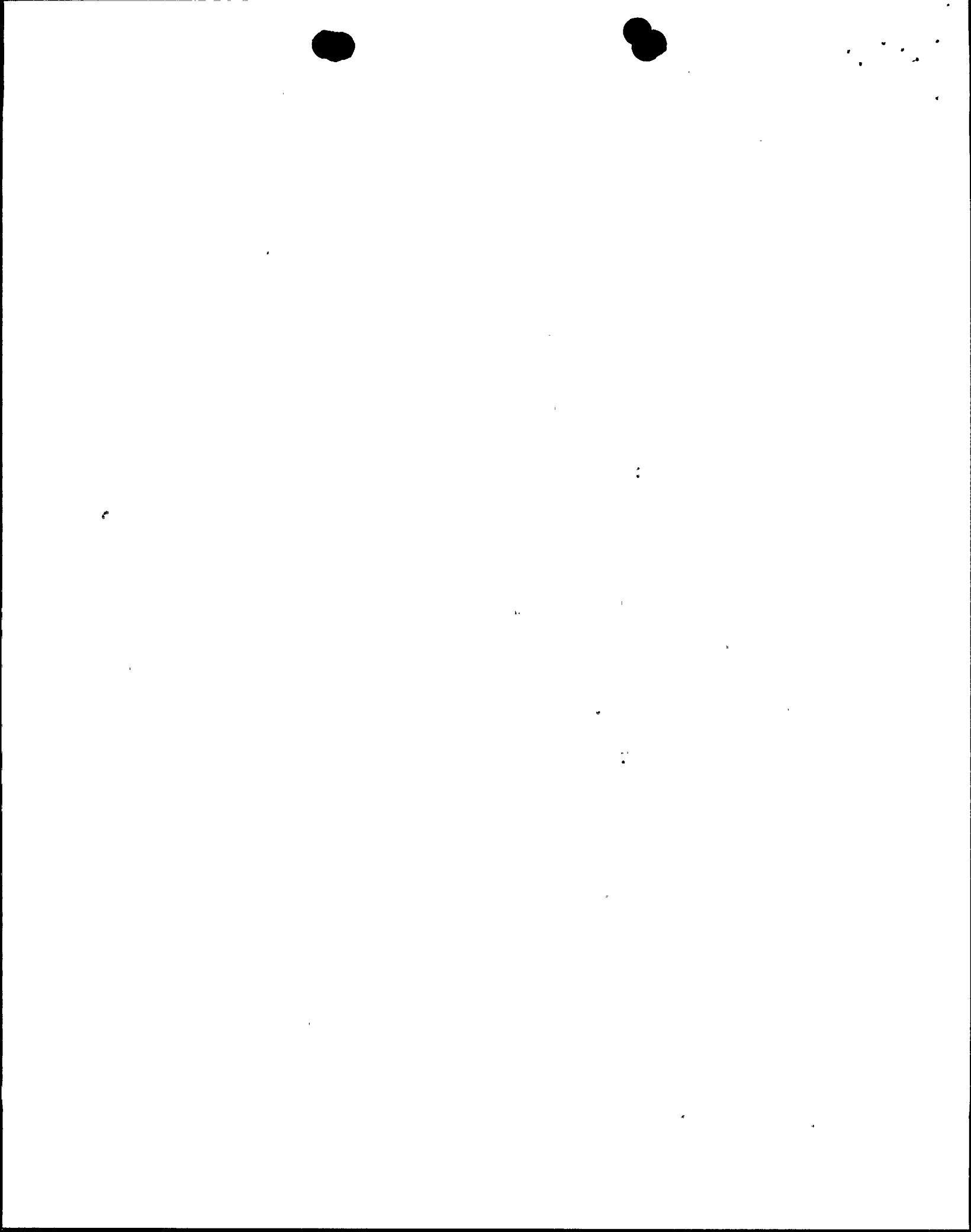
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- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- (1) The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal. †
- (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications. †
- (3) In the operation of the facility, the licensee shall, pursuant to the Federal Water Pollution Control Act Amendments of 1972 (Public Law 92-500), comply with all applicable thermal water quality standards of the State of Alabama and the United States.



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- (4) The licensee shall maintain in effect and fully implement all provisions of the Commission approved physical security plan including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plan, which contains information protected under 10 CFR 73.21, is entitled "Browns Ferry Nuclear Plant Physical Security Plan," dated May 15, 1982 (TVA letter dated June 11, 1982) and revisions submitted by TVA letters dated August 31, 1982 and October 19, 1982.

Notwithstanding the statement in Section 9.1 of the physical security plan, the licensee shall maintain positive access control over containment in accordance with the requirements of 10 CFR 73.55(d)(8).

- (5) The licensee shall follow all provisions of the NRC approved Guard Training & Qualification Plan, including amendments and changes made pursuant to 10 CFR 50.54(p). The approved Guard Training & Qualification Plan is identified as "Browns Ferry Nuclear Power Station Guard Training & Qualification Plan," dated August 17, 1979, as revised by pages dated January 24, 1980, May 21, 1980, October 1, 1980, and March 9, 1981 and as may subsequently be revised in accordance with 10 CFR 50.54(p). The Guard Training & Qualification Plan shall be followed, in accordance with 10 CFR 73.55(b), 60 days after the date of this amendment.
- (6) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Browns Ferry Physical Security Plan", with revisions submitted through May 24, 1988, and "Browns Ferry Security Personnel Training and Qualification Plan", with revisions submitted through April 16, 1987; and "Browns Ferry Safeguards Contingency Plan", with revisions submitted through June 27, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- (7) Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for BFN as approved in the SER dated _____ (and Supplements dated _____) subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.



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LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

Deleted

Deleted

6.2.2 (Cont.)

- d. Two licensed reactor operators shall be in the control room during any cold startups, while shutting down the reactor, and during recovery from unit trip. In addition, a person holding a senior operator license shall be in the control room for that unit whenever it is in an operational mode other than cold shutdown or refueling.
- e. A Health Physics Technician* shall be present at the facility at all times when there is fuel in the reactor.
- f. A person holding a senior operator license or a senior operator license limited to fuel handling, shall be present during alteration of the core to directly supervise the activity and during this time shall not be assigned other duties.
- g. Deleted.

*The Health Physics Technician may be absent from the facility for a period of time not to exceed 2 hours provided immediate action is taken to fill the required position.

6.5.1.6 (Cont.)

- l. Review of reportable events, unusual events, operating anomalies, and abnormal performance of plant equipment.
- m. Investigate reported or suspected incidents involving safety questions or violations of the Technical Specifications.
- n. Review of unit operations to detect potential hazards to nuclear safety. Items that may be included in this review are NRC inspection reports, QA audit, NSRB audit results, American Nuclear Insurer (ANI) inspection results, and significant corrective action reports (CARs).
- o. Performance of special reviews, investigations, or analysis, and report thereon as requested by the Plant Manager or the Nuclear Safety Review Board.
- p. Review of the Fire Protection Program, "Fire Protection Report, Volume-1", and implementing procedures, "Safe Shutdown Instructions", and the submittal of PORC approved changes to the Nuclear Safety Review Board.

ENCLOSURE 2

PROPOSED TECHNICAL SPECIFICATION (TS)

BROWNS FERRY NUCLEAR PLANT (BFN)

UNITS 1, 2, AND 3

Summary of the Proposed Changes

I. Unit 1

Delete License Condition 2.C.4:

"The Facility may be modified as described in Section X of Plan For Evacuation, Repair, and Return to Service of Browns Ferry Units 1 and 2 (March 2, 1975 fire) dated April 13, 1975 and versions thereto."

II. Unit 2

A. Delete License Condition 2.C.5 first paragraph:

"The Facility may be modified as described in Section X of Plan For Evacuation, Repair, and Return to Service of Browns Ferry Units 1 and 2 (March 2, 1975 fire) dated April 13, 1975 and versions thereto."

B. Delete License Condition 2.C.5.a:

"Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Appendix R Safe Shutdown Program as described in submittal dated April 14, 1989, and as revised in submittal dated December 11, 1990, and as approved in NRC Safety Evaluation dated March 6, 1991, subject to the following provision:

The licensee may make changes to the approved Appendix R Shutdown Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire."

C. Delete definition NN from TS Section 1.0:

"NN. BFN has developed an Appendix R Safe Shutdown Program. This Program is to ensure that the equipment required by the Appendix R Safe Shutdown Analysis is maintained and demonstrated functional as follows:

1. The functional requirements of the Safe Shutdown systems and equipment, as well as appropriate compensatory measures should these systems/components be unable to perform their intended function are outlined in Section III of the Program.
 2. Testing and monitoring of the Appendix R Safe Shutdown systems and equipment are defined in Section V of the Program.
 3. Changes made to the BFN Appendix R Safe Shutdown Program will be processed in accordance with License Condition 2.C.5.(a).
- D. Make the following changes to TS Section 6.0, Administrative Controls:

1. Delete the following from Section 6.5.1.6:

"A review of the Appendix R Safe Shutdown Program and Implementing Procedures. Submit all PORC approved changes to NSRB."

2. Delete the following from Section 6.5.2.8.h, Unit Audit Activities:

"and Appendix R Safe Shutdown Program."

3. Delete the following from Section 6.8.1.1.f, Written Procedures:

"and Appendix R Safe Shutdown Program."

III. Units 1, 2, and 3

- A. Add license conditions (13), (15), and (7) to Units 1, 2, and 3 respectively, as follows:

"Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for BFN as approved in the SER dated _____ (and Supplements dated _____) subject to the following provision.

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- B. Delete TS Section 3.11/4.11 Fire Protection Systems, "Limiting Conditions for Operation/Surveillance Requirements" in its entirety including Tables 3.11.A, 3.11.B, 3.11.C, 3.11.D, and its associated bases.



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C. Make the following changes to TS, Section 6.0, Administrative Controls:

1. Delete paragraph 6.2.2.g, minimum fire brigade staffing:

"A site fire brigade of at least five members shall be maintained onsite at all times. The fire brigade shall not include the Shift Engineer and the other members of the minimum shift crew necessary for safe shutdown of the unit, nor any personnel required for other essential functions during a fire emergency."

2. Change the "asterisk" paragraph related to 6.2.2.e and 6.2.2.g to read:

"The Health Physics Technician may be absent from the facility for a period of time not to exceed 2 hours provided immediate action is taken to fill the required position."

E. Add to Section 6.5.1.6, Plant Operating Review Committee (PORC) responsibilities:

"Review of the Fire Protection Program, 'Fire Protection Report, Volume 1,' and implementing procedures, 'Safe Shutdown Instructions,' and the submittal of PORC approved changes to Nuclear Safety Review Board."

Reason for Change

BFN Units 1, 2, and 3 TSs are being revised in order to implement the guidance set forth in Generic Letters (GL) 86-10 "Implementation of Fire Protection Requirements" and 88-12 "Removal of Fire Protection Requirements from Technical Specifications." The proposed change will relocate TS Sections 3.11 Bases, 3.11/4.11 and 6.2.2.g requirements to the Fire Protection Program and add administrative controls for the Fire Protection Program to TS to implement the license condition. The Fire Protection Report will be a part of the BFN Updated Final Safety Analysis Report (UFSAR) and will become BFN's Fire Protection Program.

Justification of Change

The proposed change to the BFN license will allow BFN to implement an NRC approved fire protection program in accordance with the guidance set forth in GL 86-10 and GL 88-12. This proposal revises the facility operating license by deleting the license condition that requires BFN to modify Units 1 and 2 in accordance with Section X of the Fire Recovery Plan, deletes the Unit 2 Appendix R license condition, and adds the standard license condition for fire protection to the operating licenses for Units 1, 2, and 3.

On January 15, 1992, TVA submitted the Fire Protection Program for Staff approval. The BFN Fire Protection Program supersedes Section X of the Plan For Evaluation, Repair and Return to Service of Browns Ferry Units 1 and 2 (March 2, 1975 fire) and will become part of the UFSAR. Through deletion of the license condition issued as a result of recovery from the March 2, 1975



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fire and adding the standard license condition for fire protection, BFN will be able to revise specific features of the approved program without NRC approval provided that such changes do not involve a change in the License Condition or the ability to achieve and maintain safe shutdown of the plant under the provisions of 10 CFR 50.59.

The Fire Protection Program, provided in the January 15, 1992 letter, included the Unit 2 Appendix R Safe Shutdown Program that was approved by the Staff in a March 6, 1991 letter. This license amendment added License Condition 2.C.5.a. The standard license condition encompasses the Appendix R Safe Shutdown Program, and a separate license condition is no longer required. Therefore, TVA is deleting this Unit 2 license condition. Since the Appendix R Safe Shutdown information provided by the Fire Protection Program was Unit 2 specific, TVA will provide to the Staff an Appendix R Analysis and Appendix R Safe Shutdown Program update for Units 1 and 3 prior to their respective restart.

By deletion of Unit 2 License Condition 2.C.5.a, definition NN is no longer required. The Appendix R Program, provided by the January 15, 1992 letter, has functional requirements and testing activities that will ensure systems required for safe shutdown are maintained operable.

The BFN fire protection requirements are currently implemented through TSs. The proposed amendment will relocate TS Section 3.11/4.11 "Fire Protection Systems/Surveillance Requirements" and its Bases into the Fire Protection Report. In this manner, the former TS requirements will be administratively controlled through the Fire Protection Program. The review of the program and its revisions will be the responsibility of the Plant Operations Review Committee (PORC). This proposed TS amendment adds this requirement to the Administrative Control section through the addition of paragraph 6.5.1.6.p.

The BFN fire brigade staffing requirements are currently implemented through paragraph 6.2.2.g of TS section 6.0, Administrative Control. The proposed amendment relocates this requirement to the Fire Protection Report.

The proposed TS change deletes the license condition referring to the Appendix R Safe Shutdown Program. PORC review of the Appendix R Program will be performed under review of the Fire Protection Program, provided by the Fire Protection Report. Therefore, PORC has responsibility for review of the overall program, a separate TS requirement requiring review of the Appendix R Program is no longer needed. Therefore, TVA is deleting Unit 2 TS paragraph 6.5.1.6.1. TVA also deletes reference to the Appendix R Program from Unit 2 TS paragraphs 6.5.2.8.h and 6.8.1.1.f because audit activities and written procedures will fall under the overall program.

ENCLOSURE 3

DETERMINATION OF NO SIGNIFICANT HAZARDS CONSIDERATION
BROWNS FERRY NUCLEAR PLANT (BFN)

UNITS 1, 2, AND 3

Description of Proposed TS Amendment

Browns Ferry Units 1, 2, and 3 TS Sections 3.11 Bases, 3.11/4.11 and 6.2.2.g requirements are being relocated, in their entirety, to the Fire Protection Program. The report will be part of the UFSAR and will become BFN's Fire Protection Program.

This amendment request consists of the deletion of license condition 2.C.4 for Unit 1, and for Unit 2 the first paragraph in 2.C.5, and 2.C.5.a. The amendment adds the Standard License Condition for fire protection to the Units 1, 2, and 3 operating licenses and adds administrative controls over the Fire Protection Report to TS to implement the license condition. It also deletes TS 3.11 Bases, 3.11/4.11 Fire Protection Systems, and Section 6.2.2.g Fire Brigade Staffing Requirements for BFN. TS 3.11 Bases, 3.11/4.11 Fire Protection System, and 6.2.2.g Fire Brigade Staffing Requirements have been incorporated into the Fire Protection Program.

The amendment also adds paragraph 6.5.1.6.p, requiring PORC review of the Fire Protection Program. For Unit 2 only, the amendment deletes definition "NN," paragraph 6.5.1.1, and deletes reference to the Appendix R Program in paragraphs 6.5.2.8.h and 6.8.1.1.f.

Basis for Proposed No Significant Hazards Consideration Determination

NRC has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92(c). A proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from an accident previously evaluated, or (3) involve a significant reduction in the margin of safety.

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The existing license condition requires that BFN Units 1 and 2 comply with the license condition based on Section X of the Fire Recovery Plan and, that Unit 2 operate under the auspices of the Appendix R Safe Shutdown Program. The proposed license condition will require all provisions of the present Fire Protection Program to be maintained in effect and that changes to the program be made in accordance with the provisions of 10 CFR 50.59. The overall objective of the Fire Protection Program and the license condition is to ensure safe shutdown of the plant in the event of a fire. The provision of 10 CFR 50.59 preserves the ability to achieve and maintain safe shutdown of the plant.

The proposed TS amendment will have no affect on the fire protection program. The operability requirements of the fire protection features and the surveillance requirements are not being changed. This change removes the fire protection TS 3.11 Bases, 3.11/4.11 Fire Protection Systems and 6.2.2.g, Fire Brigade Staffing Requirements from TSs, and places them into the UFSAR; and adds administrative requirements. This relocation is considered administrative in nature; therefore, this amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed license condition will ensure that the ability to achieve and maintain safe shutdown in the event of a fire is preserved. The proposed license condition is consistent with the objectives of GL 86-10. That is, proposed changes to the Fire Protection Program are reviewed against the criteria contained in 10 CFR 50.59.

The proposed TS changes do not alter the requirements to maintain the operability of the fire protection features, and does not alter minimum staffing requirements. This change relocates these requirements to the Fire Protection Report. Plant procedures will continue to provide specific instructions for implementing Limiting Conditions for Operation action and surveillance requirements. There has been no reduction in commitments and, this administrative change meets requirements of existing TSs. Therefore, these proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. This change does not involve a significant reduction in the margin of safety.

Removal of the existing license conditions referencing Section X of the Fire Recovery Plan and the Appendix R Safe Shutdown Program is consistent with guidance set forth in GL 86-10. The proposed license condition is consistent with GL 86-10 in that it requires that changes to the Fire Protection Program be made in accordance with the provisions of 10 CFR 50.59. The Fire Protection Program will be incorporated into the UFSAR during the next annual update.

TS 3.11 Bases, 3.11/4.11, and 6.2.2.g are being deleted, and the requirements contained therein will be incorporated into the UFSAR during the next scheduled update. Plant procedures will continue to provide the specific instructions necessary for the implementation of the requirements, just as when the requirements resided in the TSs. Changes to the program will be reviewed under 10 CFR 50.59. Therefore, this administrative change will not result in a reduction in the margin of safety.

Determination of Basis for Proposed No Significant Hazards

Since the application for amendment involves a proposed change that is not encompassed by the criteria for which significant hazards consideration exists, TVA has made a proposed determination that the application involves no significant hazards considerations.