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ACCELERATED DIST. BUTION DEMONSTRATION SYSTEM

# REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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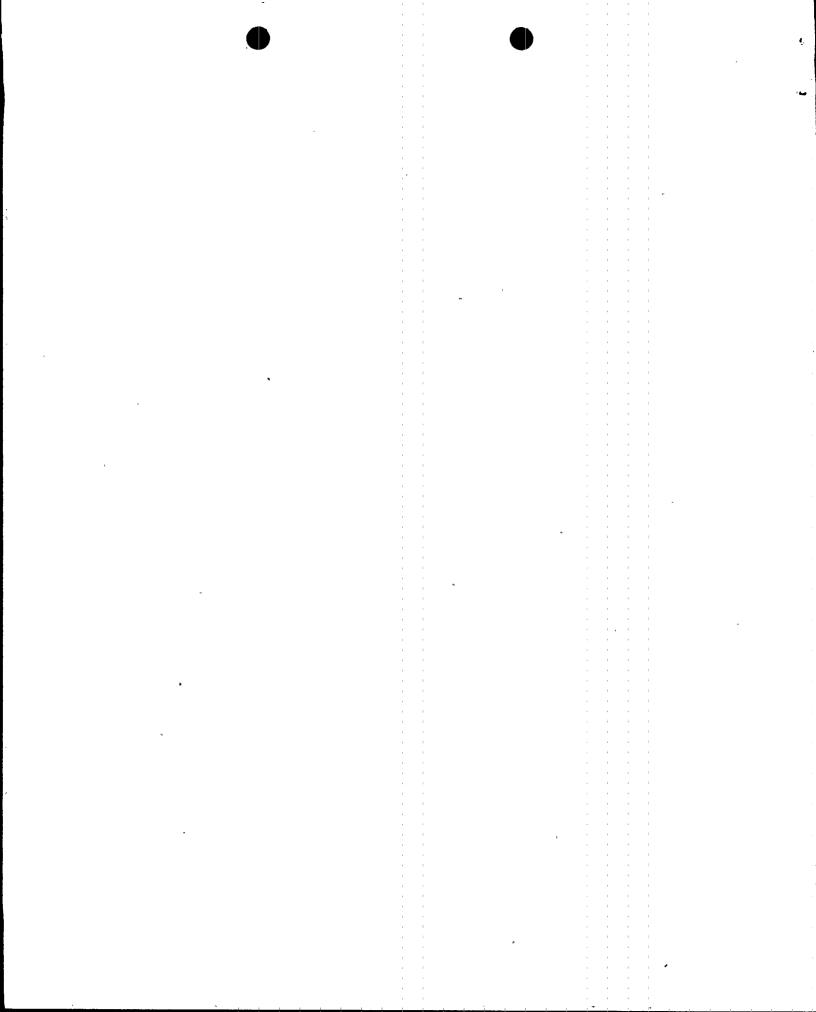
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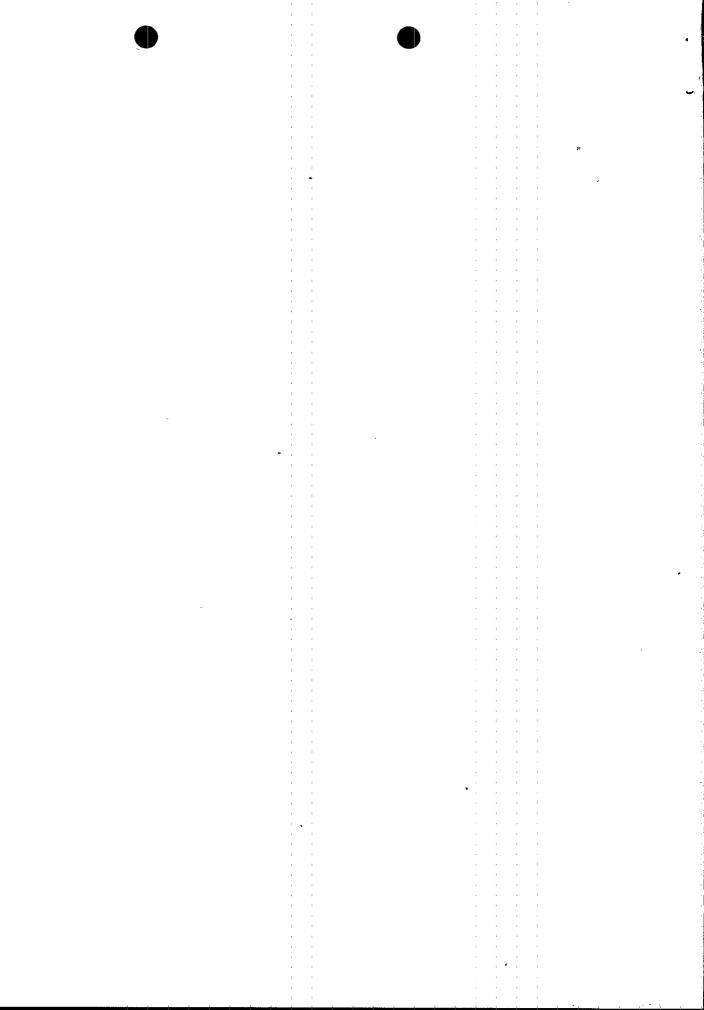
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U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88

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#### Description of Event

NRC Form 366A

9-83)

Two similar events involving missed compensatory sampling for out-of-service radiation monitors occurred due to deficient procedures which contributed to poor communication. Browns Ferry units 1, 2, and 3 were defueled during these events. Unit 3 was affected by these events.

On November 16, 1988, at 0900 hours, it was discovered that compensatory sampling required by Technical Specifications (TS) 3.2.D.2-3, due to the unit 3 raw cooling water (RCW)(EllS identifier KG) effluent radiation monitor (EIIS identifier IL) being inoperable, had not been initiated upon the RCW return to service. Chemistry was notified on November 15, 1988, at 2311 hours, the 3A reactor building (RBCCW) heat exchanger (EIIS identifier CL) was placed in service with RCW flow and the 3B heat exchanger was still out-of-service. Chemistry began compensatory sampling per Surveillance Instruction (SI) 4.2.D.-3B on the 3A heat exchanger only. At 0500 hours, on November 16, 1988, Chemistry personnel observed that the 3B heat exchanger had RCW flow and initiated sampling at that time. The RBCCW 3B heat exchanger was still not in service on the RBCCW side.

On November 18, 1988, at 0240 hours, it was discovered that compensatory sampling required by TS 3.2.D.2-3 had been erroneously discontinued. Sampling was terminated at 0430 hours on November 16, 1988, when Operations notified Chemistry that the spare RBCCW heat exchanger serving unit 3 was taken out-of-service. Chemistry personnel believed that the RCW had also been secured which was not the case. Sampling was reinitiated on November 18, 1988, at 0302.

### Cause of Event

NRC FORM 366A (9-93)

Both events were caused by a communication problem between the chemistry lab shift supervisor and the unit operator. This problem was due to inadequate Operating Instructions (OI). The OI for the RBCCW system (OI-70) requires that Chemistry be notified when an RBCCW heat exchanger is placed in or removed from service along with the status of the RCW flow to the heat exchanger. However, the OI for placing the RCW in service to the heat exchanger does not require similar notification.

In the first event, the RCW to the 3B heat exchanger was placed back in service per OI-24 after maintenance outage, however, the 3B RBCCW heat exchanger, was not placed in service. The RCW; side status was not communicated to Chemistry. The second event occurred when the spare RBCCW heat exchanger, in service to unit 3, was taken out-of-service but Chemistry was not aware that the RCW flow to the heat exchanger remained in service. Although OI-70 requires Operations to notify Chemistry of changes to both the RBCCW and RCW sides of the heat exchanger, it is not clear that Chemistry should be advised of RCW status even when only the RBCCW side status changes.

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LICENSEE EVENT REPOR	(LER) TEXT CONTINUATION
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U.S. NUCLEAR REGULATORY COMMISSION
- APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

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# Analysis of Event

VRC Form 366A

(9-83)

NRC FORM 366A

(9.83)

There is no indication that any radiation release limits were exceeded for either event.

For the first event, the 3A RBCCW heat exchanger being sampled indicated activity below maximum permissible concentration. After initiation of sampling on the 3B RBCCW heat exchanger neither 3A nor 3B indicated activity above maximum permissible concentration. Since the RBCCW flow was isolated from the 3B heat exchanger, there would have been no increased consequences to this event had unit 3 been operating at full power.

For the second event, samples taken prior to the event at 0309 hours on November 16, 1988, from the outlet of the spare RBCCW heat exchanger and after the event at 0302 hours on November 18, 1988, both showed effluent count rates below the lower limit of detection. The 3A and 3B RBCCW heat exchangers were in service and compensatory samples were taken at their outlets during this time. All radiation measurements were below detectable levels. The radiation level of RCW samples has been consistently below detectible levels in the past. Since the RBCCW flow was isolated from the spare heat exchanger, there would have been no increased consequences to this event had unit 3 been operating at full power.

## Corrective Action

For both events, the immediate corrective action was to resume the required compensatory sampling.

Procedures which control operations which have the potential for creating the need for compensatory sampling have been reviewed. It was determined that the procedures were adequate with the exception of the OIs for RCW and RBCCW systems. They will be revised to ensure that notifications to Chemistry when an RBCCW heat exchanger is placed in or removed from service clearly includes the status of RCW flow. Additionally, SI 4.2.D-3B will be revised to ensure that all RBCCW heat exchanger RCW samples are included in a composite of each unit's heat exchanger discharge by requiring sampling at the monitor sample point when available. When the sample point at the monitor is unavailable, all unit heat exchangers will be assumed to have RCW flow and sampled for inclusion in a composite sample. A copy of this LER will be included in the required reading for all licensed operators.

Previous Similar Events - BFR0-50-259/88010



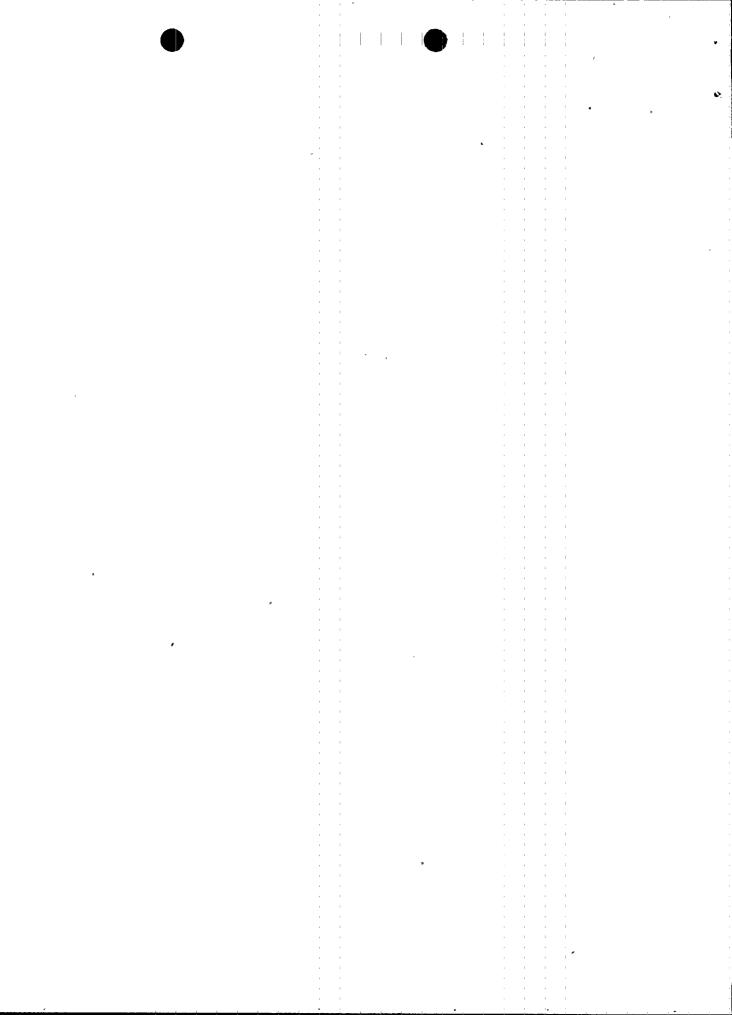
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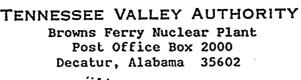
#### Commitments

Revise the OI for both RBCCW and RCW systems to ensure that the Chemistry Laboratory is notified specifically of RCW status when an RBCCW heat exchanger is removed from service and of an RCW status change for an out-of-service RBCCW heat exchanger. Procedures changes will be complete by February 17, 1989.

Revise SI 4.2.D-3B to require that all RBCCW heat exchangers be sampled and . combined if the preferred sample point (monitor) is not available. Procedure change will be complete by February 17, 1989.

Include a copy of this LER in the required reading package for all licensed operators. The review will be complete by February 17, 1989.





DEC 15 1985

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFR0-50-296/88006

The enclosed report provides details concerning the procedural deficiency resulting in failure to comply with technical specifications. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(i).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Guy G. Campbell Plant Manager Browns Ferry Nuclear Plant

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Enclosures cc (Enclosures): Regional Administration U.S. Nuclear Regulatory Commission Office of Inspection and Enforcement Region II 101 Marietta Street, Suite 2900 Atlanta, Georgia 30303

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Resident Inspector, Browns Ferry Nuclear Plant

