



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 7, 2018

Mr. Bryan C. Hanson  
Senior Vice President  
Exelon Generation Company, LLC  
President and Chief Nuclear Officer  
Exelon Nuclear  
4300 Winfield Rd  
Warrenville, IL 60555

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2 – STAFF REVIEW OF MITIGATING STRATEGIES ASSESSMENT REPORT OF THE IMPACT OF THE RE-EVALUATED SEISMIC HAZARD DEVELOPED IN RESPONSE TO THE MARCH 12, 2012, 50.54(f) LETTER (CAC NOS. MF7812 AND MF7813; EPID L-2016-JLD-0006)

Dear Mr. Hanson:

The purpose of this letter is to provide the U.S. Nuclear Regulatory Commission's (NRC) assessment of the seismic hazard mitigating strategies assessment (MSA), as described in the August 31, 2017, letter (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17243A018), submitted by Exelon Generation Company, Inc. (Exelon, the licensee) for Calvert Cliffs Nuclear Power Plant, Units 1 and 2 (CCNPP, Calvert Cliffs). The NRC staff evaluated the CCNPP strategies developed under Order EA-12-049 and described in Exelon's Final Integrated Plan (FIP) for CCNPP (ADAMS Accession No. ML16131A638). The staff's review of CCNPP's mitigating strategies was documented in a safety evaluation dated September 29, 2016 (ADAMS Accession No. ML16258A446). The purpose of the safety evaluation is to ensure that the licensee has developed guidance and proposed designs which, if implemented appropriately, should adequately address the requirements of Order EA-12-049. An inspection will confirm compliance with the order and was scheduled for the week of January 29, 2018. The following NRC staff review confirms that the licensee has adequately addressed the reevaluated seismic hazard within CCNPP's mitigation strategies for beyond-design-basis external events.

#### BACKGROUND

By letter dated March 12, 2012 (ADAMS Accession No. ML12053A340), the NRC issued a request for information pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.54(f) (hereafter referred to as the 50.54(f) letter). The 50.54(f) letter was issued as part of implementing lessons-learned from the accident at the Fukushima Dai-ichi nuclear power plant. Enclosure 1 to the 50.54(f) letter requested that licensees reevaluate the seismic hazard using present-day methodologies and guidance.

Concurrent with the reevaluation of seismic hazards, the NRC issued Order EA-12-049, "Issuance of Order to Modify Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML12054A736). The order

requires holders of operating power reactor licenses and construction permits issued under 10 CFR Part 50 to develop, implement, and maintain guidance and strategies to maintain or restore core cooling, containment, and spent fuel pool cooling following a beyond-design-basis external event. In order to proceed with the implementation of Order EA-12-049, licensees used the current design basis flood and seismic hazard or the most recent flood and seismic hazard information, which may not be based on present-day methodologies and guidance, in developing their mitigation strategies.

On December 10, 2015 (ADAMS Accession No. ML16005A621), the Nuclear Energy Institute (NEI) submitted Revision 2 to NEI 12-06, including guidance for conducting MSAs using the reevaluated hazard information. The NRC subsequently endorsed NEI 12-06, Revision 2, with exceptions, clarifications, and additions, in Japan Lessons-Learned Division (JLD) interim staff guidance (ISG) JLD-ISG-2012-01, Revision 1, "Compliance with Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML15357A163).

#### MITIGATION STRATEGIES ASSESSMENT

By letter dated July 8, 2015 (ADAMS Accession No. ML15153A073), the NRC staff documented its review of the licensee's reevaluated seismic hazard, also referred to as the mitigation strategies seismic hazard information (MSSHI). The NRC staff confirmed that the licensee's ground motion response spectra (GMRS) exceeds the safe shutdown earthquake (SSE) for CCNPP, Units 1 and 2 over the frequency range of approximately 6 to 40 hertz (Hz). The NRC staff also determined that the CCNPP Individual Plant Examination of External Events (IPEEE) program did not meet the IPEEE program screening criteria described in the NEI Guidance document, "Seismic Evaluation Guidance: Screening, Prioritization and Implementation Details (SPID) for the Resolution of Fukushima Near-Term Task Force Recommendation 2,1: Seismic" (ADAMS Accession No. ML12333A170). As such, a seismic risk evaluation, a high frequency confirmation (HF) and spent fuel pool (SFP) evaluation are merited. Further, the NRC staff concluded that the GMRS determined by the licensee adequately characterizes the reevaluated hazard for the CCNPP site and is suitable for use in subsequent evaluations and confirmations, as needed, for the response to the 50.54(f) letter.

By letter dated December 4, 2015 (ADAMS Accession No. ML15338A002), Exelon submitted a HF confirmation report for CCNPP. By letter dated February 18, 2016 (ADAMS Accession No. ML15364A544), the NRC staff concluded, based on its review, that the licensee correctly implemented the guidance in conducting the HF confirmation for CCNPP. No component modifications were required.

By letter dated August 31, 2017 (ADAMS Accession No. ML17243A018), Exelon submitted the seismic MSA report for CCNPP. The licensee stated that the CCNPP MSA was performed consistent with Appendix H of NEI 12-06, Revision 4 (ADAMS Accession No. ML16354B421). In a letter to the NEI dated February 8, 2017 (ADAMS Accession No. ML17034A286), the NRC staff stated that JLD-ISG-2012-01, Revision 2 (ADAMS Accession No. ML17005A182) had been issued and had been made publicly available. This ISG revision endorsed NEI 12-06, Revision 4, with exceptions, clarifications and additions. However, the NRC letter to the NEI also cautioned that JLD-ISG-2012-01, Revision 2, is not intended to be referenced by licensees in submittals to the NRC, and that the NRC staff would not make use of this ISG revision, until all applicable Congressional Review Act (CRA) requirements have been met. Currently, the CRA requirements for JLD-ISG-2012-01, Revision 2 have not been met.

Regarding NEI 12-06, Revision 4, the NRC staff conducted a thorough review, with numerous stakeholder interactions, that ultimately resulted in the exceptions, clarifications and additions discussed in the NRC's letter dated February 8, 2017. Based on that review, the NRC staff concludes that following the provisions of NEI 12-06, Revision 4, with the exceptions, clarifications, and additions contained in JLD-ISG-2012-01, Revision 2, is an acceptable alternative to NEI 12-06, Revision 2. Therefore, the methodology used by the licensee is acceptable to perform an assessment of the mitigation strategies that addresses the reevaluated seismic hazard.

The NRC staff performed a checklist review of the seismic hazard MSA for CCNPP. The checklist is provided as an enclosure to this letter. The NRC staff found that CCNPP met the intent of the guidance. The staff did not identify any deficiencies. All evaluated components demonstrated adequate seismic capacity and no component modifications were required.

The NRC staff completed its review of the seismic hazard MSA for CCNPP and concluded that sufficient information has been provided to demonstrate that the licensee's plans for the development and implementation of guidance and strategies under Order EA-12-049 appropriately address the reevaluated seismic hazard information stemming from the 50.54(f) letter.

If you have any questions, please contact me at (301)415-3041 or via e-mail at Stephen.Wyman@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to be 'S. Wyman', with a long horizontal line extending to the right.

Stephen M. Wyman, Project Manager  
Beyond-Design-Basis Engineering Branch  
Division of Licensing Projects  
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosure:  
Technical Review Checklist

cc w/encl: Distribution via Listserv

**TECHNICAL REVIEW CHECKLIST**  
**BY THE OFFICE OF NUCLEAR REACTOR REGULATION**  
**RELATED TO PATH FOUR MITIGATING STRATEGY ASSESSMENT**  
**CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2**  
**DOCKET NOS. 50-317 AND 50-318**

The NRC staff performed the following checklist review based on the Enclosure of the August 31, 2017, letter (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17243A018) for Calvert Cliffs Nuclear Power Plant, Units 1 and 2 (CCNPP). Deviations, deficiencies, and conclusions are noted at the end of each section and an overall conclusion is provided at the end of the checklist.

I. Background and Assessment to Mitigation Strategies Seismic Hazard Information (MSSHI)

<p>This section establishes basic background and assessment to MSSHI criteria in Nuclear Energy Institute (NEI) 12-06, Appendix H.</p>	
<p>Licensee approach to mitigating strategies assessment (MSA):</p>	
<p>Was the MSA conducted in accordance with NEI 12-06, Revision 2 as endorsed by the staff?</p>	<p>Yes / No</p>
<p>Was the MSA conducted using an alternate method?</p>	<p>Yes / No</p>
<p>Status of Order EA-12-049 Flexible Mitigation Strategy at the time of this review:</p>	
<p>Has the licensee submitted a Final Integrated Plan?</p>	<p>Yes / No</p>
<p>Has the NRC staff completed a safety evaluation for the mitigation strategy?</p>	<p>Yes / No</p>
<p>Has the NRC staff confirmed compliance with Order EA-12-049 by successfully completing the temporary instruction (TI)-191 inspection?</p>	<p>Yes / No</p>
<p>Status of MSSHI</p>	
<p>Did the licensee use the Ground Motion Response Spectra (GMRS) and Uniform Hazard Response Spectra (UHRS) as submitted in response to the 50.54(f) request for information and reviewed by the NRC staff?</p>	<p>Yes / No</p>

<p>Has the plant equipment relied on for FLEX strategies previously been evaluated as seismically robust to the plant safe shutdown earthquake (SSE) levels?</p> <p>Is the maximum ratio of GMRS/SSE in the range of 1-10 Hertz (Hz) less than 2?</p> <p>Did the licensee meet the seismic evaluation criteria described in NEI 12-06, Section H.5?</p>	<p>Yes / No / NA</p> <p>Yes / No</p> <p>Yes / No</p>
<p>Notes from staff reviewer: The GMRS/SSE ratio is approximately 1.17. This meets the criteria of NEI 12-06, Appendix H.5.</p> <p>Deviation(s) or deficiency(ies) and Resolution: Licensee performed assessment using NEI 12-06 Revision 4. The NRC staff found that following the provisions of NEI 12-06, Revision 4, with the exceptions, clarifications, and additions contained in JLD-ISG-2012-01, Revision 2, is an acceptable alternative to NEI 12-06, Revision 2 at CCNPP. NRC staff evaluated using provisions of alternative guidance in NEI 12-06 Revision 4.</p> <p>Consequence(s): None</p>	
<p>The NRC staff concludes:</p> <ul style="list-style-type: none"> <li>The licensee meets the background and assessment to MSSHI criteria in NEI 12-06, Appendix H.</li> </ul>	<p>Yes / No</p>

II. Expedited Seismic Evaluation Process (ESEP) Equipment

<p>Equipment used in support of the FLEX strategies has been evaluated to demonstrate seismic adequacy following the guidance in Section 5 of NEI 12-06. As stated in Appendix H of NEI 12-06, previous seismic evaluations should be credited to the extent that they apply for the assessment of the MSSHI, including the ESEP evaluations performed in accordance with Electric Power Research Institute (EPRI) Report 3002000704. "Seismic Evaluation Guidance: Augmented Approach for the Resolution of Fukushima Near-Term Task Force Recommendation 2.1: Seismic." (ADAMS Accession No. ML13102A142).</p> <p>Licensees may reference a previous ESEP submittal, submit a new or updated ESEP report, or provide other adequate justification or evaluation.</p> <p>Did the licensee previously perform an ESEP?</p>	<p>Yes / No</p>
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<p>Did the licensee provide a new or updated ESEP report with the MSA?</p> <p>If the licensee did not perform ESEP, did they provide adequate justification that the expedited seismic equipment list structures, systems, and components (SSCs) are acceptable in accordance with the original guidance and in accordance with NEI 12-06 Section H.5 C<sub>10%</sub> capacity criteria?</p> <p>If the licensee did not perform the ESEP, did they perform an evaluation consistent with the guidance in NEI 12-06, Section H.4.4, Steps 2 and 3, including the evaluation of FLEX components that were not previously evaluated to GMRS or 2 times the SSE?</p>	<p><del>Yes</del> / No</p> <p><del>Yes</del> / <del>No</del> / NA</p> <p><del>Yes</del> / <del>No</del> / NA</p>
<p>Notes from staff reviewer: The licensee stated that FLEX items not included in the ESEP were evaluated for the CCNPP MSSHI except the FLEX Storage Commercial Building (FSCB) which houses FLEX equipment not needed to implement the strategy as noted in Section 3.6 of the CCNPP FLEX safety evaluation (ADAMS Accession No. ML16258A446). The licensee performed an analysis in accordance with NEI 12-06 Section H.5 and concluded that these items have adequate capacity.</p> <p>Deviation(s) or deficiency(ies) and Resolution: None</p> <p>Consequence(s): None</p>	
<p>The NRC staff concludes:</p> <ul style="list-style-type: none"> <li>The licensee has evaluated seismic adequacy of equipment used in support of FLEX strategy consistent with the NEI 12-06, Appendix H guidance.</li> </ul>	<p>Yes / <del>No</del></p>

III. Inherently / Sufficiently Rugged Equipment

<p>Appendix H, Section 4.4 of NEI 12-06, Revision 2 documents the process and justification for inherently and sufficiently rugged SSCs.</p> <p>The licensee:</p> <p>Documented the inherently and sufficiently rugged SSCs consistent with the NEI 12-06 Appendix H guidance.</p>	<p>Yes / <del>No</del></p>
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<p>Did the licensee provide adequate description/documentation of the evaluation?</p>	<p>Yes / <del>Ne</del></p>
<p>Notes from staff reviewer: The licensee stated no buried tanks exist within the FLEX strategy. The staff also reviewed referenced document CA09996 to confirm haul path liquefaction analysis.</p> <p>Deviation(s) or deficiency(ies) and Resolution: None</p> <p>Consequence(s): None</p>	
<p>The NRC staff concludes:</p> <ul style="list-style-type: none"> <li>• The licensee followed the NEI 12-06, Appendix H guidance in evaluating SSCs not deemed inherently rugged.</li> </ul>	<p>Yes / <del>Ne</del></p>

V. Spent Fuel Pool (SFP) Cooling

<p>Per NEI 12-06, Appendix H, Section 4.4, licensees need to evaluate the adequacy of SFP cooling equipment to the GMRS. Most plants include the Order EA-12-051 SFP Level Instrument as part of the strategy.</p>	
<p>The licensee:</p> <ul style="list-style-type: none"> <li>• Clearly identified the SSCs and locations of the equipment that is part of the final FLEX SFP cooling strategy.</li> <li>• Clearly stated the seismic design-basis (e.g. SSE) of the equipment used in the strategy.</li> <li>• Provided adequate description or documentation of the SFP cooling equipment's evaluation to the GMRS. Portable equipment and flexible hoses do not need to be evaluated.</li> </ul>	<p>Yes / <del>Ne</del></p> <p>Yes / <del>Ne</del></p> <p>Yes / <del>Ne</del></p>
<p>Notes from staff reviewer: The NRC staff confirmed that the SFP cooling equipment described in the licensee's FIP was previously evaluated to the SSE for CCNPP. The NRC staff confirmed the maximum ratio of GMRS to SSE for CCNPP (1.17) is less than the lowest ratio for C10% capacities (1.36).</p> <p>Deviation(s) or deficiency(ies) and Resolution: None</p>	

Consequence(s): None	
The NRC staff concludes: <ul style="list-style-type: none"> <li>The licensee followed the NEI 12-06, Appendix H guidance in evaluating SFP cooling.</li> </ul>	Yes / <del>No</del>

VI. High Frequency (HF)

Per NEI 12-06, Appendix H, Section 4.4, licensees with GMRS exceedance of the SSE above 10 Hz need to evaluate bi-stable components such as relays using the methodology described in NEI 12-06, Section H.4.2. The HF evaluation may have been submitted under separate letter or may be sent as an attachment to the MSA Report. The staff review checklist is included as an attachment to this report.	
The licensee: <ul style="list-style-type: none"> <li>GMRS exceeds the SSE above 10 Hz.</li> <li>Provided a HF evaluation as described in NEI 12-06, Section H.4.2.</li> <li>Appeared to follow the guidance for the HF evaluation.</li> <li>Provided results of demand vs. capacity with identification of resolutions as needed.</li> </ul>	Yes / <del>No</del> Yes / <del>No</del> / <del>NA</del> Yes / <del>No</del> / <del>NA</del> <del>Yes</del> / <del>No</del> / <del>NA</del>
Notes from staff reviewer: The NRC staff confirmed that the GMRS at CCNPP does not exceed 0.2g and meets the criteria of Section 3.1.1 of the EPRI Report 3002004396, "High Frequency Program Application Guidance for Functional Confirmation and Fragility Evaluation" (ADAMS Accession No. ML15223A102). Low spectral accelerations up to 0.2g have been determined to be non-damaging.	
Deviation(s) or deficiency(ies) and Resolution: None	
Consequence(s): None	
The NRC staff concludes: <ul style="list-style-type: none"> <li>The licensee's component capacity evaluation met the intent of the HF guidance.</li> </ul>	Yes / <del>No</del>

VII. Conclusions:

The NRC staff assessed the licensee's implementation of the MSA guidance for CCNPP. Based on its review, the NRC staff concludes that the licensee's implementation of the MSA meets the intent of the guidance. The staff concludes that through the implementation of the MSA guidance, the licensee identified and evaluated the seismic capacity of the mitigating strategies equipment to ensure functionality will be maintained following a seismic event up to the GMRS. As noted in the review checklist, the staff identified one deviation and no exceptions taken from the guidance and the licensee did not identify any necessary equipment modifications or changes to the strategy.

In summary, the NRC staff has reviewed the seismic hazard MSA for CCNPP. The NRC staff concludes that sufficient information has been provided to demonstrate that the licensee's plans for the development and implementation of guidance and strategies under Order EA-12-049 appropriately address the reevaluated seismic hazard information stemming from the 50.54(f) letter.

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**\* via e-mail**

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