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SUBJECT: -Forwards revised ASME Section XI inservice sys pressure test program.

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TENNESSEE VALLEY AUTHORITY

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JAN 15 1988

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Gentlemen:

In the Matter of)
Tennessee Valley Authority)

) Docket Nos. 50-259
) 50-260
) 50-296

BROWNS FERRY NUCLEAR PLANT - ASME SECTION XI INSERVICE SYSTEM PRESSURE TEST PROGRAM

Provided as Enclosure 1 for NRC staff review and approval is a completely revised ASME Section XI Inservice System Pressure Test (SPT) Program for Browns Ferry Nuclear Plant developed in accordance with 10 CFR 50.55a(g). This program outlines the requirements for performing the first 10-year interval system pressure tests for units 1, 2, and 3 at Browns Ferry Nuclear Plant on ASME Code Class 1, 2, and 3 or equivalent components containing water, steam, or radioactive material (other than radioactive management systems). Included in the enclosure are two program relief requests, a tentative schedule for hydrostatic testing of units 1, 2, and 3 and a listing of systems to be pressure tested. The revised program supercedes all previous SPT program submittals.

The tentative testing schedule in the enclosed program indicates dates for actual tests performed. As indicated in the schedule, the remaining tests will be performed prior to the end of the first 10-year interval. We have determined that the interval for units 1, 2, and 3 will require an extension, allowed by the ASME code, for completion of first 10-year interval testing. The extension will be necessary because of the priority placed on unit 2 restart and system pressure tests which cannot be completed until just prior to each unit restart. We will notify NRC prior to March 1, 1988 of our intentions concerning extension of the first 10-year interval for the respective units.

BFN is approaching the end of the first 10-year inspection interval without receiving NRC approval of the SPT program. Accordingly, TVA requests that an expeditious review and safety evaluation be performed by the staff on this program. In the interim, BFN will proceed with performing system pressure tests per the enclosed program.

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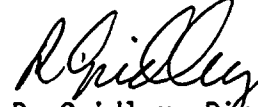
U.S. Nuclear Regulatory Commission

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Appropriate licensee review application fees were provided to NRC in an earlier program submittal. If you have any questions regarding this submittal, please telephone G. W. Morris, BFN Site Licensing, (205) 729-3583.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



R. Gridley, Director
Nuclear Licensing and
Regulatory Affairs

Enclosures

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ENCLOSURE 1
IN-SERVICE SYSTEM PRESSURE TEST
PROGRAM FOR FIRST 10 YEAR INTERVAL

BROWNS FERRY NUCLEAR PLANT UNITS 1, 2, AND 3



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IN-SERVICE PRESSURE TEST PROGRAM

Owner: Tennessee Valley Authority

Address of Corporate Office: Knoxville Office Complex
400 Commerce Avenue
Knoxville, Tennessee 37902

Name and Address of Nuclear Power Plant: Browns Ferry Nuclear Plant
P.O. Box 2000
Decatur, Alabama 35602

Applicable Nuclear Power Units: Browns Ferry Nuclear Plant,
Units 1, 2, and 3

Commercial Operation Date: Unit 1/8-1-74
Unit 2/3-1-75
Unit 3/3-1-77

COMMON IMPLEMENTATION DATE FOR
BROWNS FERRY NUCLEAR PLANT

<u>UNIT</u>	<u>COMMERCIAL OPERATION</u>	<u>CONCURRENT CYCLE</u>	<u>80-MONTH CYCLE START</u> ¹	<u>120-MONTH CYCLE START</u>	<u>END OF FIRST 10-YEAR INTERVAL</u> ²
1	8-1-74	7-1-80	7-1-81	11-1-84	3-1-88
2	3-1-75	7-1-80	7-1-81	11-1-84	3-1-88
3	3-1-77	7-1-80	7-1-81	11-1-84	3-1-88

¹Forty-month cycle was extended by one year (IWA-2400). The second 10-year interval will only be nine years.

²The program submittal for the second 10-year interval will be based on the code in effect one year before March 1, 1988 (Reference memorandum from NRC to TVA dated June 20, 1986, A02 860630 006).

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1.0 STATEMENT OF APPLICABILITY

This program outlines requirements for performing the first 10-year interval system pressure tests of Browns Ferry Nuclear Plant (BFN) units 1, 2, and 3, ASME Code Class 1, 2, and 3 or equivalent components containing water, steam, or radioactive material (other than radioactive waste management systems). This program has been organized to fulfill in-service examination requirements of Nuclear Quality Assurance Manual, Part II, Section 5.1, Procedure 2.0 and PMP BF 1402.02 and to comply as practical with the requirements of Section XI of the ASME Boiler and Pressure Vessel Code.

The requirements of this program are applicable starting at the date of commercial operation of the unit.

Specifics concerning performance of system pressure tests are not a part of this program but are included in plant surveillance instructions.

2.0 PURPOSE

The in-service System Pressure Test (SPT) Program shall be used for planning inspections and pressure testing of Browns Ferry ASME Class 1, 2, and 3 equivalent components for the first 10-year inspection interval.

3.0 INSPECTION INTERVALS AND INSPECTION PERIODS

The in-service SPT required by ASME Section XI shall be performed during each 10-year interval of service (inspection interval). The inspection intervals represent calendar years after the unit has been placed in commercial service. The commercial operation date for unit 1 is August 1, 1974, unit 2 is March 1, 1975, and unit 3 is March 1, 1977. The first 10-year interval may be decreased or extended by as much as one year. If the unit is out-of-service continuously for six months or more, the inspection interval may be extended for an equivalent period. Units 1, 2, and 3 were incorporated into a concurrent cycle beginning July 1, 1980 (letter to NRC L00 801231 705). This approach was approved by NRC on June 20, 1986 (A02 860630 006).

The inspection interval shall be separated into three inspection cycles or periods, i.e., calendar months of plant service, at 40, 80, and 120 months.

4.0 CODES OF RECORD

4.1 This program was prepared to meet the requirements of the 1974 Edition, Summer 1975 Addenda of Section XI of the ASME Boiler and Pressure Vessel Code except as stated below:



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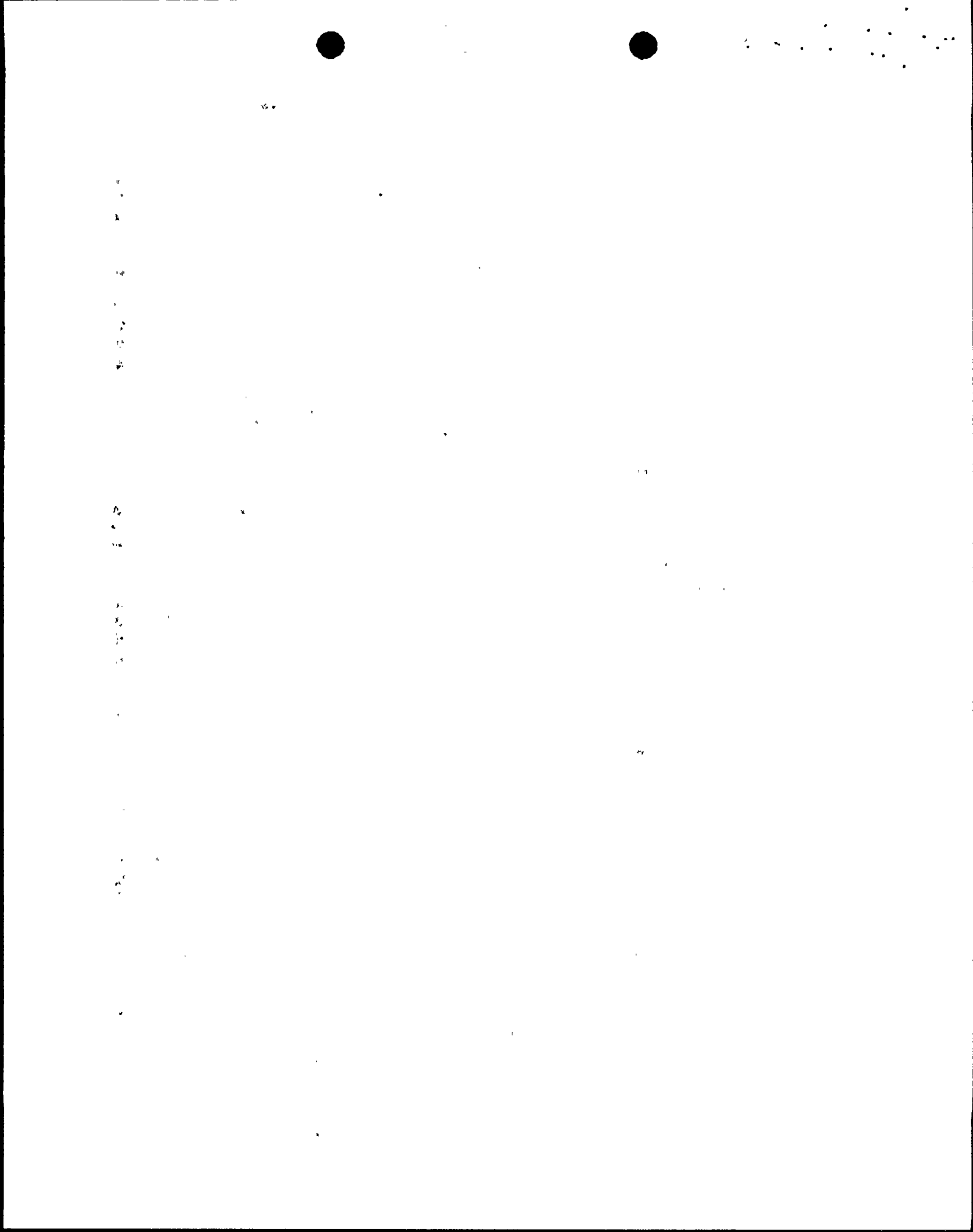
In accordance with 10 CFR 50.55 a(g)(4)(IV) the following areas shall meet the requirements of the 1980 Edition, Winter 1981 Addenda of ASME Section XI of the Code in lieu of the base code of record. These areas shall consist of:

- (1) Test Pressurization Boundaries (IWA-5220 and Footnote 1 for Examination Category B-P of Table IWB-2500-1)
- (2) Hydrostatic Test Condition Holding Time (IWA-5213)
- (3) Code Class 2 or Equivalent Test Temperature (IWC-5230)
- (4) System Hydrostatic Pressure Test of Code Class 2 or equivalent exempt and nonexempt components
- (5) Hydrostatic Pressure Including Static Head (IWA-5265)

4.1.1 Identification of test pressurization boundaries shall be in accordance with Subsubarticle IWA-5220. In accordance with Paragraph IWA-5224:

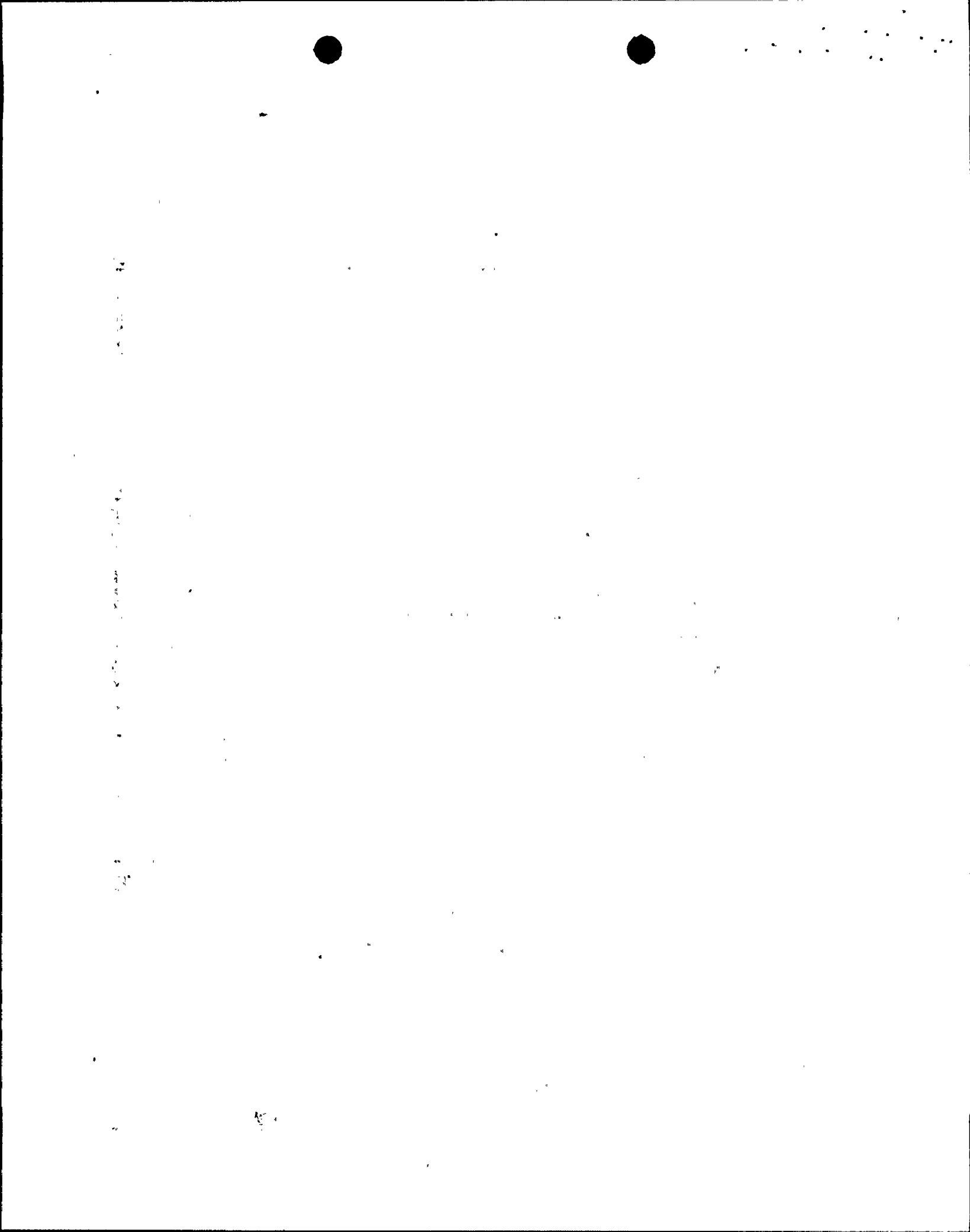
- (a) The boundary subject to test pressurization during a system hydrostatic test [IWA-5211(d)] shall be defined by the system boundary (or each portion of the boundary) within which the components have the same minimum required classification and are designed to the same primary pressure rating as governed by the system function and the internal fluid operating conditions, respectively.
- (b) Systems which share safety functions for different modes of plant operation, and within which the component classifications differ, shall be subject to separate system pressure tests of each portion of the system boundary having the same minimum required component classifications.
- (c) Systems designed to operate at different pressures under several modes of plant operation or postaccident conditions shall be subject to an SPT within the test boundary defined by the operating mode with the higher pressure.
- (d) Where the respective system primary pressure ratings on the suction and discharge sides of system pumps differ, the system test boundary shall be divided into two separate boundaries (such as suction side and discharge side test boundaries). In the case of positive displacement pumps, the boundary interface shall be considered as the pump. In the case of centrifugal pumps, the boundary interface shall be the first shutoff valve on the discharge side of the pump.

For Class 2 and 3 systems, the hydrostatic test pressure shall be determined by the component within the test boundary with the lowest design pressure.



During system leakage tests, IWA-5211(a), on Class 1 components the test boundary shall be defined as "the reactor coolant system boundary with all valves in the normal position which is required for normal reactor operation startup." The VT-2 examination shall, however, extend to and include the second closed valve at the boundary extremity, as stated in footnote 1, examination category B-P, Table IWB-2500-1 of the 1980 Edition, Winter 1981 Addenda of ASME Section XI.

- 4.1.2 Hydrostatic test condition holding time shall be in accordance with paragraph IWA-5213. The holding time after pressurization to test conditions, before the visual examinations commence shall be as follows: system hydrostatic tests - 4-hour holding time required after attaining the test pressure and temperature conditions for insulated systems, and 10 minutes for noninsulated systems or components.
- 4.1.3 System test temperature for ASME Class 2 components or equivalent shall be controlled in accordance with paragraph IWC-5230:
 - (a) The system test temperature during a system hydrostatic test in systems containing ferritic steel components shall meet the requirements specified by fracture prevention criteria.
 - (b) In systems containing ferritic steel components for which fracture toughness requirements were neither specified nor required in the construction of the components, the system test temperature shall be determined by the Owner.
 - (c) No limit on system test temperature is required for systems comprised of components constructed entirely of austenitic steel materials.
- 4.1.4 System pressure test of ASME Section XI Code Class 2 or equivalent exempt and nonexempt components required by paragraph IWC-2510 of the 1974 Edition, Summer 1975 Addenda of ASME Section XI, shall be conducted at a frequency in accordance with Table IWC-2500-1 Examination Category C-H Item Nos. C7.20, C7.40, C7.60, and C7.80 of ASME Section XI 1980 Edition, Winter 1981 Addenda.
- 4.1.5 System hydrostatic pressure tests of Code Class 1, 2, and 3 or equivalent components involving static head shall be controlled in accordance with paragraph IWA-5265:
 - (a) When testing an isolated component, the pressure measuring instrument or sensor shall be connected close to the component.

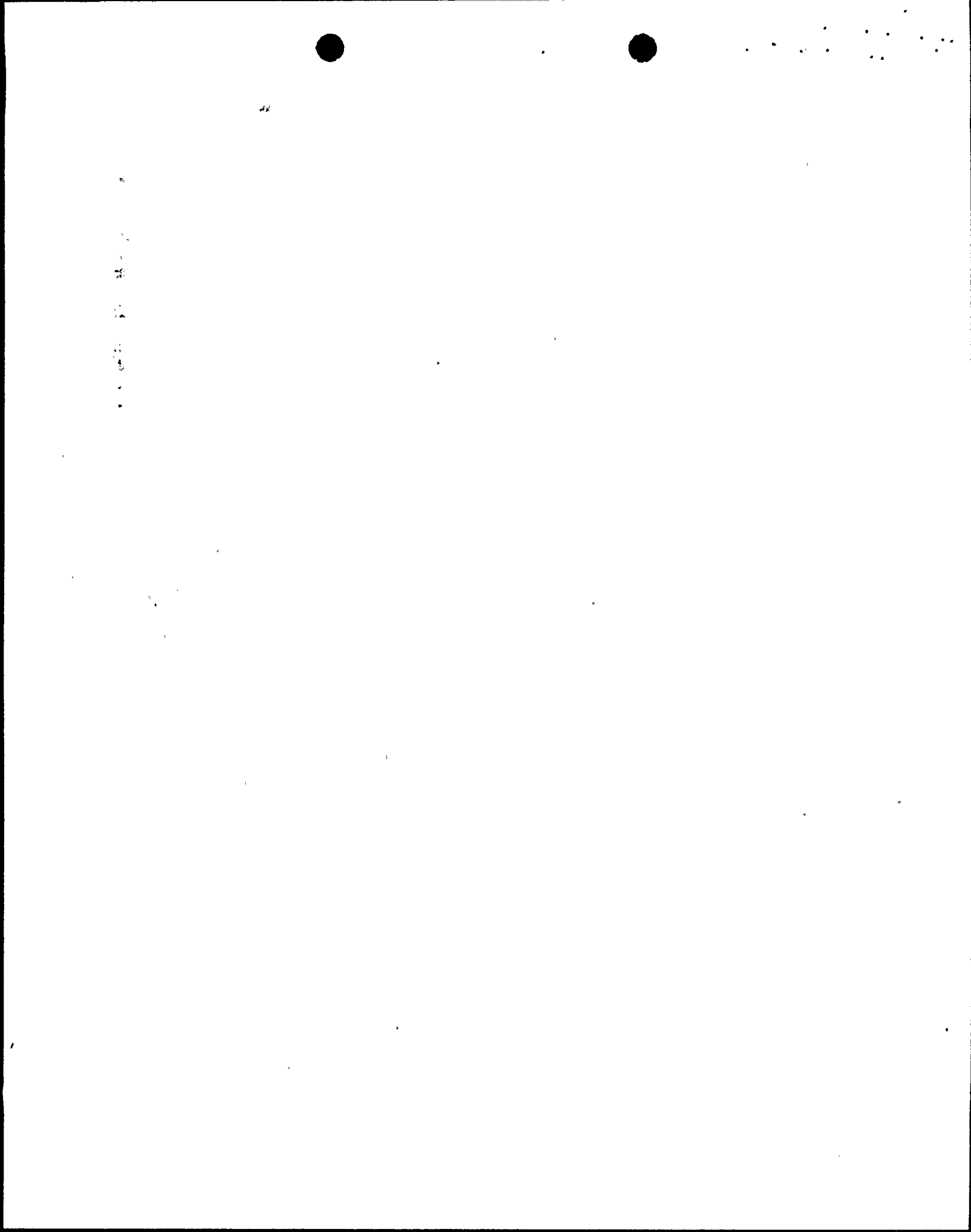


ASME SECTION XI
TABLE IWC-2500-1
EXAMINATION CATEGORIES
TABLE 4.1.4-1

EXAMINATION CATEGORY C-H, ALL PRESSURE RETAINING COMPONENTS						
Item No.	Parts Examined ¹	Test ² Required	Examination ³ Method	Acceptance Standard	Extent of Examination ⁴	Frequency of Examination
C7.10	Pressure Vessels Pressure Retaining Components	IWC-5221 test ⁸	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.20	Pressure Retaining Components	IWC-5222 test	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.30	Piping Pressure Retaining Components	IWC-5221 test ⁸	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.40	Pressure Retaining Components	IWC-5222 test	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.50	Pumps Pressure Retaining Components	IWC-5221 test ⁸	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.60	Pressure Retaining Components	IWC-5222 test	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.70	Valves Pressure Retaining Components	IWC-5221 test ⁸	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶
C7.80	Pressure Retaining Components	IWC-5222 test	Visual, VT-2	IWA-5250	Pressure retaining boundary ⁷	Each inspection period ⁶

NOTES:

- (1) Other than open-ended portions of systems.
- (2) System pressure tests of IWA-5000 and IWC-5000.
- (3) Visual examination of IWA-5240.
- (4) No components within the pressure retaining boundary [as defined by Note (7)] are exempt or excluded from the examination requirements, except as specified in IWA-5214(c) for repairs and replacements.
- (5) The system hydrostatic test (IWC-5222) shall be conducted at or near the end of each inspection interval or during the same inspection period of each inspection interval of Inspection Program B.
- (6) Where portions of a system are subject to system pressure tests associated with two different system functions, the VT-2 examination need only be performed during the test conducted at the higher of the test pressures of the respective system function.
- (7) The pressure retaining boundary includes only those portions of the system required to operate or support the safety system function up to and including the first normally closed valve (including a safety or relief valve) or valve capable of automatic closure when the safety function is required.
- (8) A system hydrostatic test (IWC-5222) and accompanying VT-2 examination are acceptable in lieu of the system pressure test (IWC-5221) and VT-2 examination.



- (b) When testing a group of components or a multicomponent system, the pressure measuring instrument or sensor shall be connected to any point within the pressure boundary of the components or system such that the imposed pressure on any component, including static head, will not exceed 106% of the specified test pressure for the system.

Test pressure, P_T shall be measured within the system test boundary. At no time shall the actual pressure applied at the system boundary lowpoint exceed 106% of the test pressure, P_T .

5.0 METHOD OF IMPLEMENTATION AND RESPONSIBILITIES

The SPT Program preparation is the responsibility of the ISI Programs Section. Any requests for relief or revisions initiated by other groups shall be submitted to the ISI Programs Section for approval before incorporation into this program. The ISI Programs Section shall submit the SPT Program to the NRC for approval via the Manager, Nuclear Licensing and Regulatory Affairs.

6.0 REQUESTS FOR RELIEF

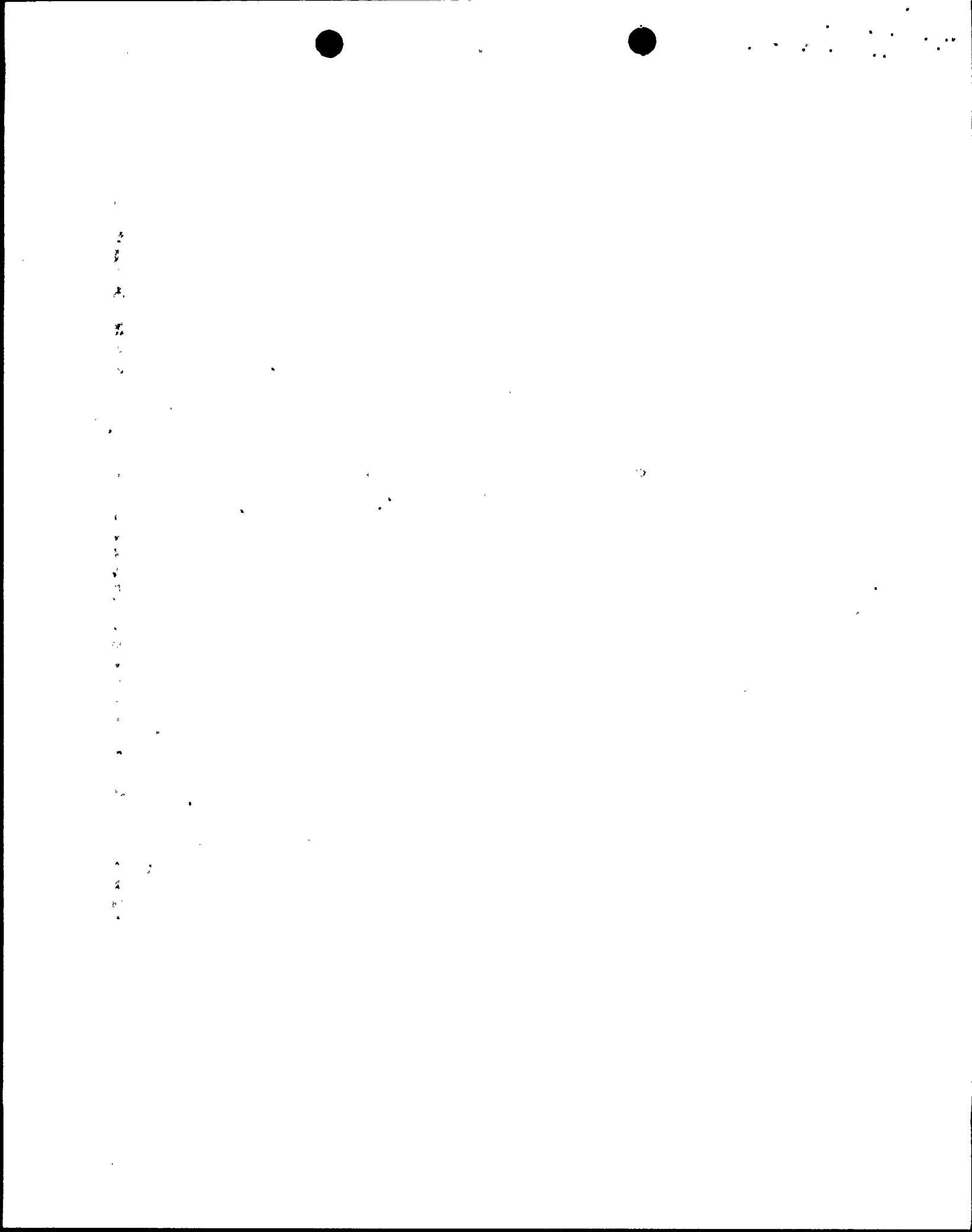
REQUEST FOR RELIEF H-1

Request for Relief H-1 has been withdrawn.

Reference: Memorandum from L. M. Mills to T. A. Ippolito dated
January 23, 1981 (A27 810123 004)

REQUEST FOR RELIEF H-2

Request for Relief H-2 has been withdrawn.



REQUEST FOR RELIEF H-3

UNITS - 1, 2, 3

SYSTEM - Main Steam (MS), Reactor Core Isolation Cooling (RCIC), High Pressure Coolant Injection (HPCI)

CLASS - ASME Code Class 2 equivalent

TEST REQUIREMENT - ASME Section XI, 1974 Edition, Summer 1975 Addenda, Article IWA-5000, Paragraph IWC-5220(a). The system hydrostatic test pressure shall be at least 1.25 times the system design pressure and conducted at a test temperature not less than 100°F except as may be required to meet the test temperature requirements of IWA-5230.

BASIS FOR RELIEF -

- 1) In the main steam system the outboard main steam isolation valves (MSIV) serve as the boundary between class 1 and class 2. Testing the class 2 piping, as required by paragraph IWC-5220(a), would require pressuring the MSIVs in the reverse direction from their design.

Pressurizing the valve in this direction will cause the valve to unseat and leak. The valve manufacturer has stated that mechanically restraining the valve could damage the valve stem.
- 2) In the HPCI system, check valve FCV-73-45 is designed to prevent flow from class 1 to class 2, and is the boundary between class 1 and class 2. This valve cannot hold pressure from the class 2 direction.
- 3) In the RCIC system, check valve FCV-71-40 is designed to prevent flow from class 1 to class 2 and is the boundary between class 1 and class 2. This valve cannot hold pressure from the class 2 direction.



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REQUEST FOR RELIEF H-3 (cont'd)

ALTERNATE TESTING -

- 1) All class 2 MS system piping will be tested in conjunction with the class 1 reactor vessel system hydrostatic pressure test at the appropriate class 1 pressure.
- 2) That portion of the class 2 HPCI system piping between FCV-73-45 and FCV-73-44 will be tested in conjunction with the class 1 reactor feedwater system at the appropriate class 1 pressure.
- 3) That portion of the class 2 RCIC system piping between FCV-71-40 and FCV-71-39 will be tested in conjunction with the class 1 reactor feedwater system at the appropriate class 1 pressure.

REFERENCE -

TVA Drawings:

MS	47W801-1
	47W801-2
HPCI	47W812-1
RCIC	47W813-1



REQUEST FOR RELIEF H-4

Request for Relief H-4 has been withdrawn.

REQUEST FOR RELIEF H-5

Request for Relief H-5 has been withdrawn.

REQUEST FOR RELIEF H-6

Request for Relief H-6 has been withdrawn.

REQUEST FOR RELIEF H-7

Request for Relief H-7 has been withdrawn.

REQUEST FOR RELIEF H-8

Request for Relief H-8 has been withdrawn.

REQUEST FOR RELIEF H-9

Request for Relief H-9 has been withdrawn.



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REQUEST FOR RELIEF H-10

Request for Relief H-10 has been withdrawn.



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REQUEST FOR RELIEF H-11

Request for Relief H-11 has been withdrawn.

Reference: Memorandum from L. M. Mills to T. A. Ippolito dated July 11, 1979
(A27 790711 006)

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REQUEST FOR RELIEF H-12

UNITS - 1, 2, 3

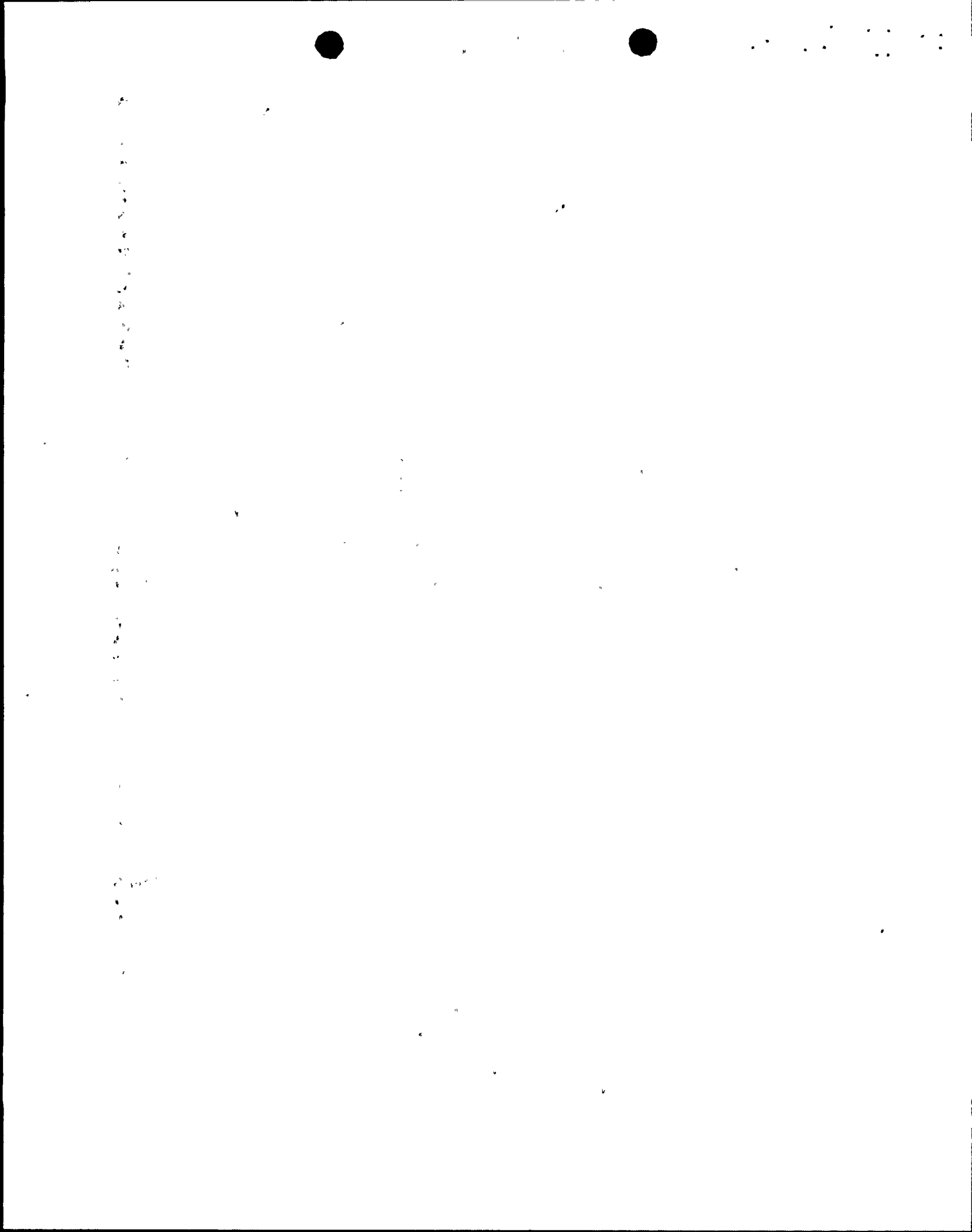
SYSTEM - Control Rod Drive (CRD): 3/4-inch piping between valves FCV 85-39B* and 85-617* and 1-inch and 1/2-inch piping between valves FCV 85-39A*, 85-590*, and the hydraulic accumulators.

CLASS - ASME Code Class 2 equivalent

TEST REQUIREMENT - ASME Section XI, 1974 Edition, Summer 1975 Addenda, Paragraph IWC-5220(a). The system hydrostatic test pressure shall be at least 1.25 times the system design pressure (P_d) and conducted at a test temperature not less than 100°F except as may be required to meet the test temperature requirements of IWA-5230.

BASIS FOR RELIEF - During the analysis of the system for the hydrostatic pressure test, several potential problems were identified that could cause safety concerns with personnel and possible damage to equipment. The portion of CRD piping under evaluation has a design pressure of 1750 psig and a design temperature of 150°F. In accordance with the requirements of the 1974 Edition, Summer 1975 Addenda the hydrostatic test pressure would be 2188 psig (1.25 times design pressure). The design configuration of the hydraulic control unit at BFN has a piston with pressure seals that should not have a differential pressure greater than 300 psig to ensure they are not damaged during a test. This limitation requires nitrogen to be supplied to the opposite side of the piston within 300 psig of the hydrostatic test pressure. This design also contains a rupture disc which will burst to relieve pressure at 1900 to 2100 psig. At the hydrostatic test pressure of 2188 psig, the minimum required nitrogen pressure would be 1888 psig which would require the nitrogen tank rupture disc to be removed and plugged. This high pressure nitrogen (1888 psig) poses danger to personnel in two ways: (1) the nitrogen is pressurizing a system above its normal operating pressure; and (2) a large quantity of nitrogen is released into the space occupied by the test personnel when the nitrogen tank rupture disc is removed.

* There are 185 valves with this number which are denoted by -1 through -185 after the flow diagram valve number.



REQUEST FOR RELIEF H-12

BASIS FOR RELIEF (cont'd)-

If the test pressure is reduced to 1.1 times the design pressure, as allowed by IWC-5222 of the 1977 Edition Winter 1977 Addenda and later codes, the hydrostatic pressure will be reduced to 1925 psig, which allows the nitrogen pressure to be reduced to 1625 psig to maintain the 300 psig pressure differential. This lower nitrogen pressure (1625 psig) does not require the rupture disc to be removed. Therefore, both the lower nitrogen pressure and not removing the rupture disc will (1) reduce the amount of nitrogen released into plant spaces, (2) simplify the procedure, and (3) reduce the chance of injury to personnel or damage to equipment.

For the second 10-year interval, BFN will be updating its code of record for in-service inspection examinations and SPTs from the 1974 Edition, Summer 1975 Addenda to the latest NRC-approved code in accordance with 10 CFR 50.55a(b)(2), which will require the system hydrostatic pressure to be 1.1 times the design pressure for systems with design temperature of 200°F or less (IWC-5222). The next scheduled performance of the CRD Hydrostatic Test would be performed at the lower test pressure (1.1 times design pressure) rather than the test pressure required by the 1974 Edition, Summer 1975 Addenda (1.25 times design pressure).

For these reasons TVA feels the CRD System hydrostatic test pressure should be reduced: to minimize the risk to personnel and equipment, simplify test procedure, and perform an effective pressure test as allowed by later editions of ASME Codes.

Based on the above justification, TVA concludes that a significant decrease in safety would not result from the proposed alternate inspection below.

ALTERNATE TESTING

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Perform CRD system hydrostatic pressure test at 1.1 times system design pressure (P_d) for 3/4-inch piping between valves FCV 85-39B* and 85-617* and 1-1/2-inch (1.5 inch) piping between valves FCV 85-39A*, 85-590*, and the hydraulic accumulators.

REFERENCE

-

TVA Drawing CRD 47W820-2

7.0 TENTATIVE SYSTEM PRESSURE TEST SCHEDULE

BFNP
TENTATIVE SYSTEM PRESSURE TEST SCHEDULE

SURVEILLANCE INSTRUCTION NO.	DESCRIPTION	PERFORMANCE DATE		
		U-1	U-2	U-3
3.3.1.A	Reactor Vessel Leak Check	Each Refueling Outage		
3.3.1.B	Reactor Vessel	*	*	*
3.3.2.A	Drywell Floor Drain Penetration	*	8-20-86	*
3.3.2.B	Drywell Equipment Drain Penetration	*	1-22-87	*
3.3.3.A	Fuel Pool Cooling (78-61 to 78-62)	*	9-17-86	*
3.3.3.B	Fuel Pool Cooling (74-91 to 78-534)	*	10-21-86	*
3.3.3.C	Fuel Pool Cooling (Pumps & Heat Exchangers)	*	8-17-87	*
3.3.4	Standby Liquid Control	*	10-7-80	*
3.3.4.A	Standby Liquid Control	*	*	*
3.3.5	Demineralized Water Penetration	*	7-16-86	*
3.3.6	Core Spray	2-28-79	*	*
3.3.7	Control Rod Drive	*	5-27-87	*
3.3.8.A.1	Residual Heat Removal (RHR) Loop I Discharge	*	*	*
3.3.8.A.2	RHR Loop I Discharge	*	*	*
3.3.8.A.3	RHR Loop I Discharge	*	*	*
3.3.8.A.4	RHR Loop I Discharge	*	*	*
3.3.8.A.5	RHR Loop I Discharge	*	*	*
3.3.8.A.6	RHR Loop I Discharge	*	*	*
3.3.8.A.7	RHR Loop I Instrument Lines	*	*	*
3.3.8.C.1	RHR Loop II Discharge	*	*	*
3.3.8.C.2	RHR Loop II Discharge	*	*	*
3.3.8.C.3	RHR Loop II Discharge	*	*	*
3.3.8.C.4	RHR Loop II Discharge	*	*	*
3.3.8.C.5	RHR Loop II Discharge	*	10-2-87	*
3.3.8.C.6	RHR Loop II Discharge	*	3-27-87	*
3.3.8.C.7	RHR Loop II Instrument Lines	*	*	*
3.3.8.E	RHR Suction	*	*	*
3.3.8.H	RHR Crossties	*	*	*
3.3.9	High Pressure Coolant Injection	1-3-84	11-22-80	11-28-78
3.3.10	Reactor Core Isolation Cooling	10-22-81	11-22-80	12-5-78
3.3.11	RBCCW Containment Penetration	*	*	*
3.3.13.A	Residual Heat Removal Service Water (RHRSW) Low Pressure (Leak Check)	*	*	*
		10-30-81	12-11-80/	10-31-81
		*	10-30-81	*
3.3.13.A.1,2,3,4	RHRSW Buried Piping	*	*	*
3.3.13.B	RHRSW High Pressure (Leak Check)	11-19-81	12-11-80/	4-8-82
		*	8-19-83	*
3.3.13.B.1	RHRSW High Pressure	*	6-24-86	*
3.3.13.B.2	RHRSW High Pressure	*	6-24-86	*
3.3.13.B.3	RHRSW High Pressure	*	6-24-86	*
3.3.13.B.4	RHRSW High Pressure	*	6-24-86	*
3.3.13.C.1	RHRSW Low Pressure	*	*	*
3.3.13.C.2	RHRSW Low Pressure	*	*	*
3.3.13.C.3	RHRSW Low Pressure	*	*	*
3.3.13.C.4	RHRSW Low Pressure	*	*	*

*Tests to be performed prior to the end of the first ten-year interval.

BFNP
TENTATIVE SYSTEM PRESSURE TEST SCHEDULE

<u>SURVEILLANCE</u> <u>INSTRUCTION NO.</u>	<u>DESCRIPTION</u>	<u>PERFORMANCE DATE</u>
3.3.14.A	U-3 Emergency Equipment Cooling Water (EECW) Leakage Test	7-26-79/9-18-82
3.3.14.B	U-3 EECW Leakage Test	7-26-79/9-18-82
3.3.14.C	U-2 EECW Leakage Test	12-11-80/8-19-83
3.3.14.D	U-2 EECW Leakage Test	12-11-80/8-19-83
3.3.14.E	U-1 EECW Leakage Test	9-25-79/9-17-83
3.3.14.F	U-1 EECW Leakage Test	9-25-79/9-17-83
3.3.14.A.1	EECW South Header	*
3.3.14.A.2	EECW North Header	*
3.3.14.B.1	EECW Unit 1&2 Diesel Supply	9-2-87
3.3.14.B.2	EECW Unit 1&2 Diesel Supply	9-11-87
3.3.14.B.3	EECW Unit 1&2 Diesel Supply	6-5-87
3.3.14.B.4	EECW Unit 1&2 Diesel Supply	*
3.3.14.C.1	EECW Unit 3 Diesel Supply	7-7-87
3.3.14.C.2	EECW Unit 3 Diesel Supply	7-30-87
3.3.14.C.3	EECW Unit 3 Diesel Supply	6-9-87
3.3.14.C.4	EECW Unit 3 Diesel Supply	5-30-87
3.3.14.D.1	EECW Unit 1&2 Diesel Heat Exchangers	*
3.3.14.D.2	EECW Unit 1&2 Diesel Heat Exchangers	*
3.3.14.D.3	EECW Unit 1&2 Diesel Heat Exchangers	*
3.3.14.D.4	EECW Unit 1&2 Diesel Heat Exchangers	*
3.3.14.E.1	EECW Unit 3 Diesel Heat Exchangers	*
3.3.14.E.2	EECW Unit 3 Diesel Heat Exchangers	*
3.3.14.E.3	EECW Unit 3 Diesel Heat Exchangers	*
3.3.14.E.4	EECW Unit 3 Diesel Heat Exchangers	*
3.3.14.F.1	EECW Diesel Room Chiller	*
3.3.14.F.2	EECW Diesel Room Chiller	*
3.3.14.G.1	EECW - RHR Room Cooler U-1	*
3.3.14.G.2	EECW - RHR Room Cooler U-1	*
3.3.14.G.3	EECW - RHR Room Cooler U-2	*
3.3.14.G.4	EECW - RHR Room Cooler U-2	*
3.3.14.G.5	EECW - RHR Room Cooler U-3	*
3.3.14.G.6	EECW - RHR Room Cooler U-3	*

*Tests to be performed prior to the end of the first ten-year interval.



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BFNP
TENTATIVE SYSTEM PRESSURE TEST SCHEDULE

<u>SURVEILLANCE</u> <u>INSTRUCTION NO.</u>	<u>DESCRIPTION</u>	<u>PERFORMANCE DATE</u>
3.3.14.H.1	EECW - 3A-1/3B-1 Chiller Supply (N)	*
3.3.14.H.2	EECW - 3A-2/3B-2 Chiller Supply (S)	*
3.3.14.H.3	Control Bay Chiller Supply (N)	*
3.3.14.H.4	Control Bay Chiller Supply (S)	*
3.3.14.I.1	EECW - H ₂ O ₂ Panel U-1	*
3.3.14.I.2	EECW - H ₂ O ₂ Panel U-1	*
3.3.14.I.3	EECW - H ₂ O ₂ Panel U-2	*
3.3.14.I.4	EECW - H ₂ O ₂ Panel U-2	*
3.3.14.I.5	EECW - H ₂ O ₂ Panel U-3	*
3.3.14.I.6	EECW - H ₂ O ₂ Panel U-3	*
3.3.14.J.1	EECW - Control Room Chillers U-1&2	*
3.3.14.J.2	EECW - Control Room Chillers U-1&2	*
3.3.14.K.1	EECW - Control Room Chillers U-3	*
3.3.14.K.2	EECW - Control Room Chillers U-3	*
3.3.14.L.1	EECW - Buried Piping	*
3.3.14.L.2	EECW - Buried Piping	*
3.3.14.L.3	EECW - Buried Piping	*
3.3.14.L.4	EECW - Buried Piping	*
3.3.14.M.1	EECW - CS Room Cooler U-1	*
3.3.14.M.2	EECW - CS Room Cooler U-1	*
3.3.14.M.3	EECW - CS Room Cooler U-2	*
3.3.14.M.4	EECW - CS Room Cooler U-2	*
3.3.14.M.5	EECW - CS Room Cooler U-3	*
3.3.14.M.6	EECW - CS Room Cooler U-3	*

*Tests to be performed at the end of the first ten-year interval.

Enclosure 2

LIST OF COMMITMENTS MADE IN THIS SUBMITTAL

1. Submit TVA intentions regarding extension of the first 10 year inservice inspection interval by 3/1/88.

1. 1950年10月1日

2. 1950年10月1日

3.