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JAN 25 2018

Docket Nos.: 52-025
52-026

ND-18-0042
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3 and Unit 4
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 3.1.00.03 [Index Number 735]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of January 17, 2018, Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Uncompleted Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 3.1.00.03 [Index Number 735] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing this ITAAC. Southern Nuclear Operating Company will, at a later date, provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI 08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact Tom Petrak at 706-848-1575.

Respectfully submitted,

Michael J. Yox
Regulatory Affairs Director Vogtle 3 & 4

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Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC 3.1.00.03 [Index Number 735]

MJY/LBP/amw

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Document Services RTYPE: VND.LI.L06

File AR.01.02.06

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**Southern Nuclear Operating Company
ND-18-0042
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC 3.1.00.03 [Index Number 735]**

ITAAC Statement

Design Commitment

3. The plant parameters listed in Table 2.5.4-1, minimum inventory table, in subsection 2.5.4, DDS, with a "Yes" in the "Display" column, can be retrieved in the TSC.

Inspections/Tests/Analyses

An inspection will be performed for retrievability of the plant parameters in the TSC.

Acceptance Criteria

The plant parameters listed in Table 2.5.4-1, minimum inventory table, in subsection 2.5.4, DDS, with a "Yes" in the "Display" column, can be retrieved in the TSC.

ITAAC Completion Description

An inspection is performed to verify the retrievability of the VEGP Unit 3 and Unit 4 plant parameters in the Technical Support Center (TSC). The inspection for retrievability confirms that the plant parameters listed in COL Appendix C Table 2.5.4-1, Minimum Inventory of Controls, Displays, and Alerts at the RSW, with a "Yes" in the "Display" column (Attachment A) can be retrieved in the TSC.

The inspection is performed using Procedure AAA, TSC Data Display and Processing System (DDS) Display Inspection – Unit 3 (Reference 1) and Procedure BBB, TSC Data Display and Processing System (DDS) Display Inspection – Unit 4 (Reference 2), and visually confirms that when each of the plant parameters identified in Attachment A is summoned at a workstation connected to the TSC Local Area Network the summoned plant parameter appears on a display monitor at that TSC workstation.

The inspection results are documented in Procedure XXX, TSC Data Display and Processing System (DDS) Display Inspection Report – Unit 3 (Reference 3) and Procedure YYY, TSC Data Display and Processing System (DDS) Display Inspection Report – Unit 4 (Reference 4) and confirms that the VEGP Unit 3 and Unit 4 plant parameters listed in Attachment A can be retrieved in the TSC.

References 3 and 4 are available for NRC inspection as part of the Unit 3 and Unit 4 ITAAC 3.1.00.03 Completion Packages (References 5 and 6).

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

References (available for NRC inspection)

1. Procedure AAA, TSC Data Display and Processing System (DDS) Display Inspection–Unit 3
2. Procedure BBB, TSC Data Display and Processing System (DDS) Display Inspection–Unit 4
3. Procedure XXX, TSC Data Display and Processing System (DDS) Display Inspection Report – Unit 3
4. Procedure YYY, TSC Data Display and Processing System (DDS) Display Inspection Report – Unit 4
5. ITAAC 3.1.00.03 Completion Package (Unit 3)
6. ITAAC 3.1.00.03 Completion Package (Unit 4)
7. NEI 08-01, “Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52”

Attachment A

Minimum Inventory of Controls, Displays, and Alerts at the RSW*

Description *	Display *
Neutron Flux	Yes
Startup Rate	Yes
Reactor Coolant System (RCS) Pressure	Yes
Wide-range Hot Leg Temperature	Yes
Wide-range Cold Leg Temperature	Yes
RCS Cooldown Rate Compared to the Limit Based on RCS Pressure	Yes
Wide-range Cold Leg Temperature Compared to the Limit Based on RCS Pressure	Yes
Containment Water Level	Yes
Containment Pressure	Yes
Pressurizer Water Level	Yes
Pressurizer Water Level Trend	Yes
Pressurizer Reference Leg Temperature	Yes
Reactor Vessel-Hot Leg Water Level	Yes
Pressurizer Pressure	Yes
Core Exit Temperature	Yes
RCS Subcooling	Yes
RCS Cold Overpressure Limit	Yes
In-containment Refueling Water Storage Tank (IRWST) Water Level	Yes
Passive Residual Heat Removal (PRHR) Flow	Yes
PRHR Outlet Temperature	Yes
Passive Containment Cooling System (PCS) Storage Tank Water Level	Yes
PCS Cooling Flow	Yes
IRWST to Normal Residual Heat Removal System (RNS) Suction Valve Status	Yes
Remotely Operated Containment Isolation Valve Status	Yes
Containment Area High-range Radiation Level	Yes
Containment Pressure (Extended Range)	Yes
Core Makeup Tank (CMT) Level	Yes

* Excerpt from COL Table 2.5.4-1