

Pennsylvania Power & Light Company  
Allegheny Electric Cooperative, Inc.  
Docket No. 50-387  
Susquehanna Steam Electric Station, Unit 1  
Facility Operating License

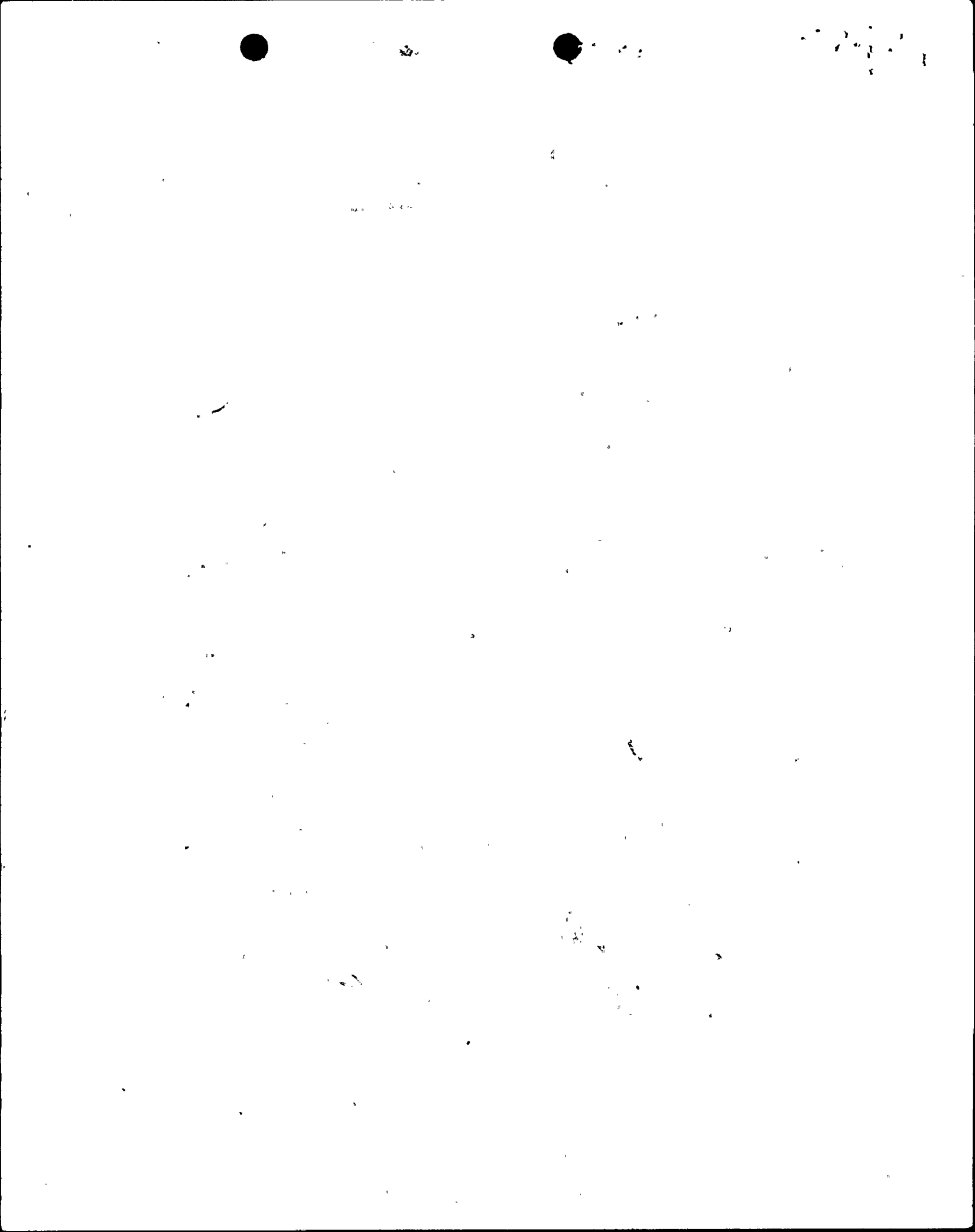
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for a license filed by the Pennsylvania Power & Light Company and the Allegheny Electric Cooperative, Inc. (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Susquehanna Steam Electric Station, Unit 1 (the facility), has been substantially completed in conformity with Construction Permit No. CPPR-101 and the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - E. The Pennsylvania Power & Light Company\* is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;

\*The Pennsylvania Power & Light Company is authorized to act as agent for the Allegheny Electric Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

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- F. The licensees have satisfied the applicable provisions of 10 CFR 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;
- G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;
- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Facility Operating License No. NPF-14 subject to the condition for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
- I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.

2. Based on the foregoing findings and the Initial Decision issued by the Atomic Safety and Licensing Board on April 12, 1982, regarding this facility, Facility Operating License No. NPF-14 is hereby issued to the Pennsylvania Power & Light Company and the Allegheny Electric Cooperative, Inc. to read as follows:

A. This license applies to the Susquehanna Steam Electric Station, Unit 1, a boiling water nuclear reactor and associated equipment (the facility), owned by the licensees. The facility is located in Luzerne County, Pennsylvania, and is described in the licensees' Final Safety Analysis Report as supplemented and amended through Amendment 48, and the licensees' Environmental Report as supplemented and amended through Amendment 48.

B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:

(1) Pursuant to Section 103 of the Act and 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities", Pennsylvania Power & Light Company (PP&L) and the Allegheny Electric Cooperative, Inc. to possess, and PP&L to use, and operate the facility at the designated location in Luzerne County, Pennsylvania, in accordance with the procedures and limitations set forth in this license;

(2) PP&L, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended through Amendment 48;

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- (3) PP&L, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed neutron sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) PP&L, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) PP&L, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

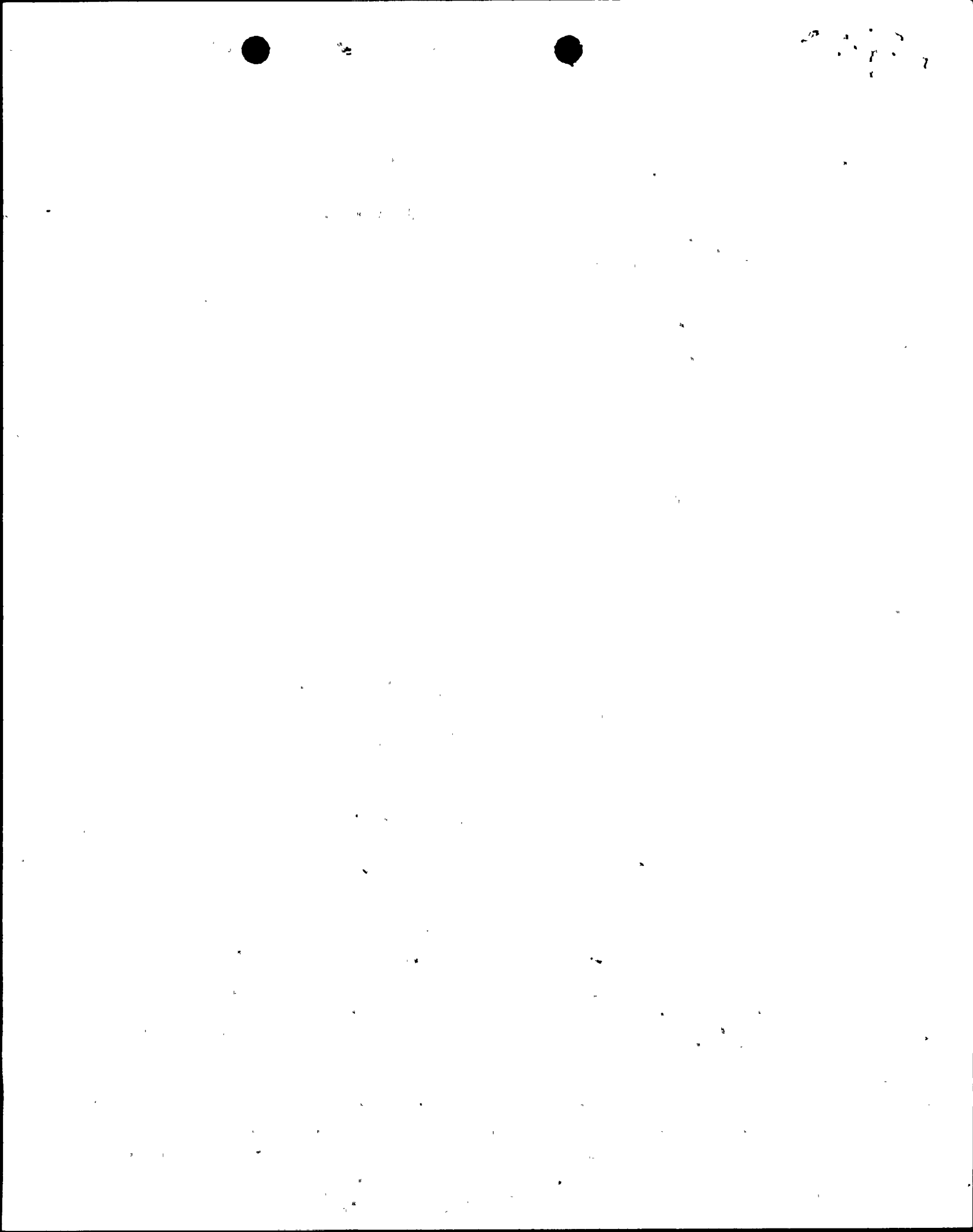
(1) Maximum Power Level

Pennsylvania Power & Light Company (PP&L) is authorized to operate the facility at reactor core power levels not in excess of 3293 megawatts thermal in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license. Pending Commission approval, this license is restricted to power levels not to exceed five percent of full power.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in this license. PP&L shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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(3) Conduct of Work Activities During Fuel Load and Initial Startup

PP&L shall review by committee all facility construction, Preoperational Testing, and System Demonstration activities performed concurrently with facility initial fuel loading or with the facility Startup Test Program to assure that the activity will not affect the safe performance of the facility fuel loading or the portion of the facility Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the facility fuel loading or the portion of the facility Startup Program being performed. The committee for the review shall be composed of a least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Assistant Superintendent of Plant's staff.

(4) Thermal and Hydraulic Design (Section 4.4, SER)

- (a) PP&L is prohibited from power operation under natural circulation conditions.
- (b) Prior to startup following the first refueling outage, PP&L shall provide, for NRC review and approval, a new stability analysis, indicating the results for appropriate exposure core conditions.

(5) Qualification of Purge Valves (Section 6.2.4, SSER #1)

- (a) Until such time as qualification data for purge valves are provided to and approved by the NRC, operation of the purge and vent containment isolation valves by the licensee shall comply with the requirements of the interim position as stated in the attachment to II.E.4.2 in NUREG-0737. As part of the interim position, the purge and vent valves shall be blocked to a maximum opening of no greater than 50 degrees.
- (b) Prior to exceeding five percent of full power, PP&L shall provide purge valve qualification documentation to the NRC for review and approval.

(6) Fire Protection Program (Section 9.5, SER, SSER#1, SSER#2, SSER#3)

PP&L shall maintain in effect and fully implement all provisions of the approved Fire Protection Review Report, as amended through Revision 1 dated March 1981. In addition, PP&L shall maintain the fire protection program set forth in Appendix R to 10 CFR Part 50.

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(7) Battery Room Area (Section 9.5.4, SER, SSER#1, SSER#3)

Prior to exceeding five percent of full power and subject to NRC review and approval, PP&L shall either conduct at an approved testing laboratory an ASTM E-119 test of the as-installed one-hour cable wrap configuration or install an automatic fire extinguishing system.

(8) Operation with Partial Feedwater Heating at End-of-Cycle (Section 15.1, SER, SSER #1)

Prior to operation with partial feedwater heating, PP&L shall provide for NRC review and approval, analyses which show a more limiting change does not occur in the minimum critical power ratio than that obtained using normal feedwater heating.

(9) Initial Test Program (Section 14, SER, SSER #1)

PP&L shall conduct the post-fuel-loading initial test program (set forth in Section 14 of the licensee's Final Safety Analysis Report, as amended through Amendment 48) without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- (a) Elimination of any test identified as essential in Section 14 of the licensees' Final Safety Analysis Report, as amended through Amendment 48;
- (b) Modifications of test objectives, methods or acceptance criteria for any test identified as essential in Section 14 of the licensee's Final Safety Analysis Report, as amended through Amendment 48;
- (c) Performance of any test at a power level different from that described in the program; and
- (d) Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).

(10) Inservice Inspection Program (Section 5.2.4 and 6.6, SER, SSER#1, SSER#3)

By June 30, 1983, PP&L shall submit a revised inservice inspection program for NRC review and approval.

(11) Seismic System Analysis (Section 3.7.2, SSER#3)

By the dates indicated, PP&L shall provide documentation to the NRC for review which states the results of recheck of all calculations associated with calculating masses, section

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properties, and spring stiffnesses used in stick models for the following structures:

- (a) Containment July 30, 1982
- (b) Reactor/Control Structure August 25, 1982  
(Vertical model)
- (c) Diesel Generator Building August 25, 1982
- (d) Engineering Safeguard Service August 25, 1982  
Water Pumphouse

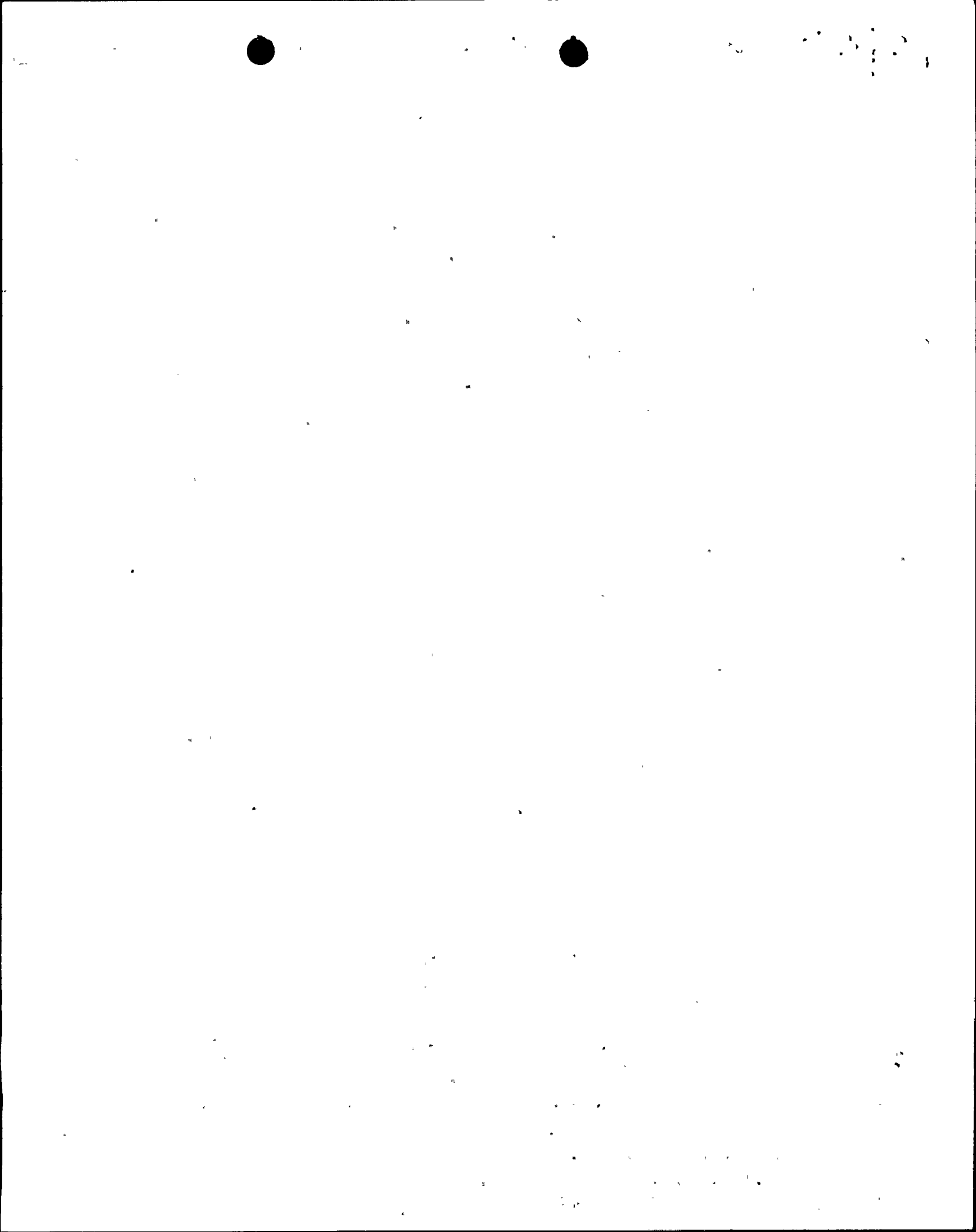
(12) Radon (ASLB Initial Decision, Paragraph 223)

This license will be subject to the ultimate outcome of the consolidated radon proceeding currently underway before the Appeal Boards in Docket Nos. 50-277, 50-278, 50-320, 50-354 and 50-355.

(13) Nearby Facilities (Section 2.2.2, SSER#3)

- (a) PP&L shall provide notification to the NRC prior to any modifications to the orifice in either the principal or secondary flow lines, shown on Transcontinental Gas Pipe Line Corp. drawing number MB-1P-1 and 34-3452MB-1P-1, Rev. 1, exceeding 2 inches in diameter. Prior to any restrictor modifications which increase the effective orifice diameter greater than 2 inches the facility shall be placed in a cold shutdown condition.
- (b) Prior to exceeding five percent of full power, PP&L shall implement administrative controls which will preclude both lines referenced in Transcontinental Gas Pipe Line Corp. drawing number MB-1P-1, Rev. 1, being simultaneously open, and shall submit a copy of the administrative controls to NRC for review.
- (c) By February 28, 1983, PP&L shall submit a report for NRC review and approval that describes either:
  1. a passive 2 inch flow restrictor to be installed in the gas pipeline in proximity to the nuclear station, or
  2. relocation of the pipeline to a distance where unrestricted flow in the pipeline would not be hazardous to the safe operation of the nuclear plant.
- (d) By September 30, 1984, the option chosen by PP&L and approved by NRC shall be fully implemented.

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(14) Seismic and Loss-of-Coolant Accident Loads (Section 4.2.3, SSER #3)

By August 30, 1982, PP&L shall submit to NRC a complete description of the analytical methods along with analytical results with regard to fuel bundle liftoff. This submittal should contain information equivalent to that to be included in the General Electric Topical Report (NEDE-21175-P) regarding fuel bundle liftoff.

(15) Control Room Design Review (Appendix F, SER, SSER#3)

By September 1, 1982, PP&L shall complete correction of the following human engineering discrepancies as noted in Appendix F of the Safety Evaluation Report:

- 2.a.(3) Left/right convention on all controllers.
- 6.f. Unconventional labeling.

(16) Wetwell to Drywell Vacuum Breakers (Section 6.2.1.8, SSER #3)

Thirty days prior to operation in excess of five percent power, PP&L shall provide the results of its vacuum breaker performance evaluation program for NRC review and approval.

(17) Scram Discharge System Piping (Section 4.6, SER, SSER#1, SSER#2, SSER#3)

(a) Within 60 days of the issuance of the BWR Owner's Group Report regarding modifications to the Emergency Procedure Guidelines, the licensee shall submit a report addressing the Emergency Procedure Guidelines with regard to Scram Discharge Volume (SDV) pipe breaks. PP&L shall implement any required system or procedural modifications on a schedule acceptable to the NRC staff.

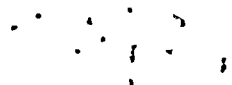
(b) Prior to startup following the first refueling outage, PP&L shall incorporate the following additional modifications into the scram discharge volume system:

- (1) Redundant vent and drain valves, and
- (2) Diverse and redundant SDV instrumentation for each instrumented volume, including both delta pressure sensors and float sensors.

(18) Environmental Qualification (Section 3.11, SER, SSER#1, SSER#2, SSER#3)

(a) PP&L shall complete all actions related to environmental qualification of equipment on a schedule specified in Section 3.11 and Appendix 3.B of Supplement No. 3 of the Safety Evaluation Report.

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(b) Complete and auditable records must be available and maintained at a central location which describe the environmental qualification methods used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," Revision 1, dated July 1981. Such records shall be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document compliance with NUREG-0588.

(c) Prior to startup following the first refueling outage, PP&L shall be in compliance with the provisions of NUREG-0588 for safety-related electrical equipment exposed to a harsh environment.

(19) Assurance of Proper Design and Construction (Section 17.6, SSER #3)

Prior to exceeding five percent of full power, PP&L shall have conducted an independent review of the mechanical and structural design of the feedwater system located inside containment extending from the Reactor Pressure Vessel nozzles to the containment penetration. This verification review shall consider design, installation, inspection, testing, and any other aspects necessary to ensure conformance with the design. This review shall be performed independently of PP&L and its contractors who perform design and construction activities for the Susquehanna Steam Electric Station.

(20) Emergency Preparedness (Appendix D, SSER #1, SSER #2)

Prior to exceeding five percent power, PP&L shall demonstrate that the state of offsite preparedness, which has been determined to be acceptable for operation at up to five percent power, provides assurance that adequate protective measures can and will be taken in the event of a radiological emergency during operations in excess of five percent power. The use of 10 CFR 50.54(s)(2) to specify a period within which corrective actions must be taken to assure an adequate state of emergency preparedness will include instances where NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's proposed rule set forth in 44 CFR Part 350 is an indication that major substantive problems exist in achieving or maintaining an adequate state of preparedness. Any corrective period specified will relate to substantive problems identified by the Federal Emergency Management Agency.

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(21) School District Emergency Plans (ASLB Initial Decision, Paragraph 223)

This license will be subject to a finding (prior to operation at power levels exceeding five percent of full power) by the Director of Nuclear Reactor Regulation, in consultation with the Federal Emergency Management Agency, that all school districts within the plume exposure pathway emergency planning zone for the Susquehanna Steam Electric Station have completed written emergency plans to respond to fixed nuclear facility accidents.

(22) Municipality Transportation Resources (ASLB Initial Decision, Paragraph 223)

This license will be subject to a finding (prior to operation at power levels exceeding five percent of full power) by the Director of Nuclear Reactor Regulation, in consultation with the Federal Emergency Management Agency, that all municipalities within the plume exposure pathway emergency planning zone have completed their emergency response plans on the transportation resources and program.

(23) Seismic and Dynamic Qualification (Section 3.10, SER, SSER#1, SSER#3)

(a) Prior to startup following the first refueling outage, PP&L shall complete any modifications or replacement of equipment found necessary as a result of the licensee's fatigue evaluation program. In the interim, PP&L shall document the occurrence of every safety relief valve discharge into the suppression pool; the associated cumulative damage factors shall be calculated for typical representative equipment and kept up-to-date; and PP&L shall report to NRC any malfunction of equipment that occurs or should be suspected to have occurred due to any safety relief valve discharge.

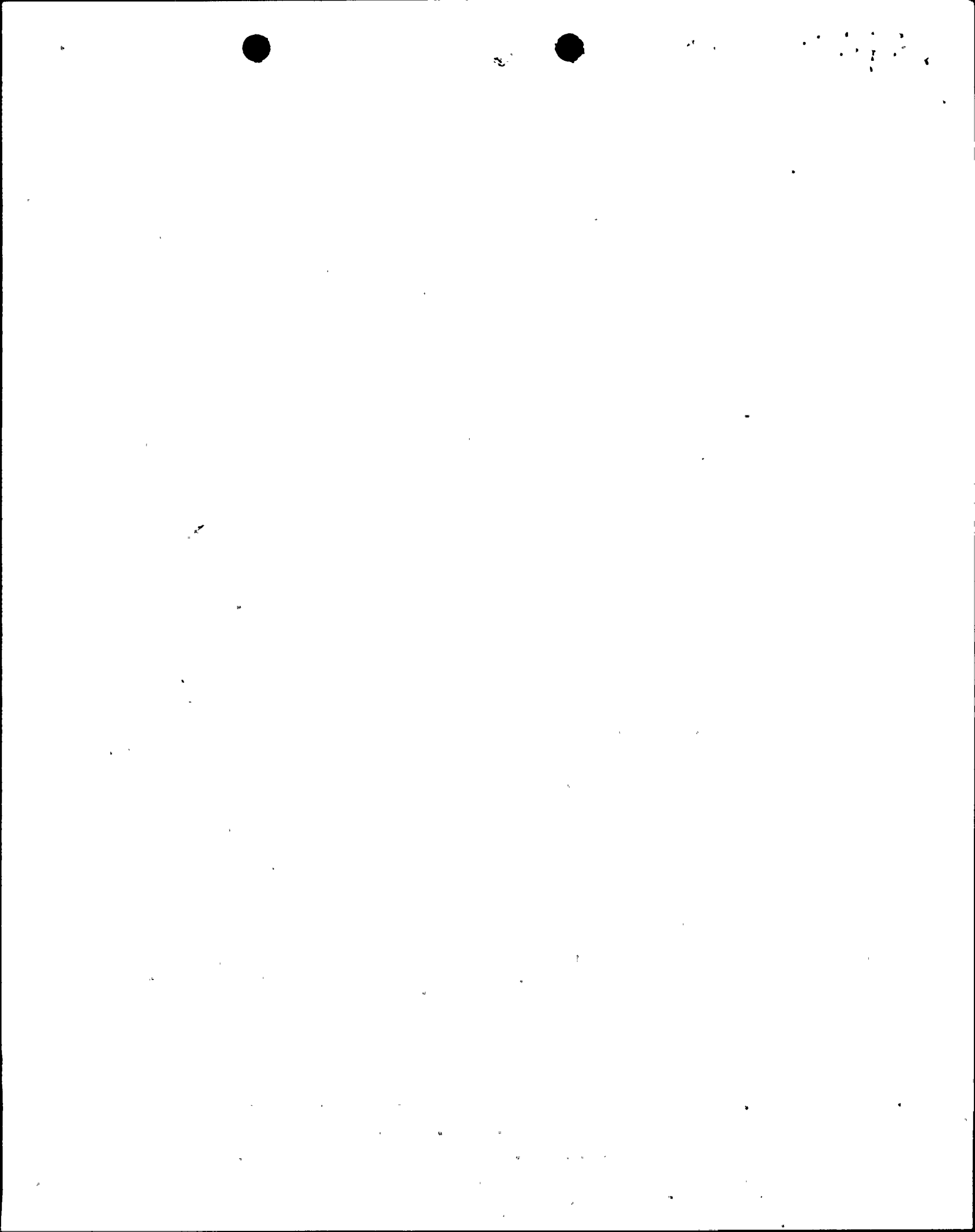
(b) PP&L shall complete all actions related to seismic and dynamic qualification of equipment identified in section 3.10 of Supplement No. 3 of the Safety Evaluation Report on the schedule specified therein.

(24) Containment Purge System (Section 6.2.4, SER)

Prior to startup following the first refueling outage, PP&L shall install design features (e.g. screens) on the containment purge system to prevent blocking of the purge and vent valves by debris produced in an accident.

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(25) Additional Instrumentation and Control Concerns (Section 7.7.2, SER, SSER #2)

Prior to startup following the first refueling outage, PP&L shall resolve the following concerns to the NRC's satisfaction:

- (a) whether common electrical power sources or sensor malfunctions may cause multiple control systems failures, and
- (b) whether high energy line breaks will result in unacceptable consequential control system failures.

(26) Surveillance of Control Blade (Section 4.2.3, SER)

Within 30 days after plant startup following the first refueling outage, PP&L shall comply with items 1, 2, and 3 of IE Bulletin No. 79-26, Revision 1, "Boron Loss from BWR Control Blades", and submit a written response on item 3.

(27) Emergency Diesel Engine Starting Systems (Section 9.6.3, SER)

Prior to startup following the first refueling outage, PP&L shall install air dryers upstream of the air receivers.

(28) NUREG-0737 Conditions (Section 22, SER)

PP&L shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2 and 3, NUREG-0776.

(a) Nuclear Steam Supply System Vendor Review of Procedures (I.C.7, SER, SSER #1)

Prior to beginning low-power testing, PP&L shall assure that the General Electric review of the power ascension test procedures has been completed and the General Electric recommendations have been incorporated.

(b) Special Low Power Testing and Training (I.G.1, SER, SSER#3)

During the first fuel cycle, PP&L shall perform Simulated Loss of All AC Power Test. At least four weeks prior to the test, PP&L shall provide a safety analysis and test procedure to NRC.

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(c) Post Accident Sampling (II.B.3, SER, SSER#1, SSER#3)

Prior to startup following the first refueling outage, PP&L shall provide to NRC a revised procedure for core damage estimation to incorporate the requirements in Section 22.2, II.B.3 of Supplement No. 3 of the Safety Evaluation Report.

(d) Instrumentation for Detection of Inadequate Core Cooling (II.F.2, SER, SSER #1, SSER#3)

(i) By August 31, 1982, PP&L shall submit a report addressing the analysis performed by the BWR Owners Group regarding additional instrumentation relative to inadequate core cooling and shall implement the staff's requirements after the completion of the staff's review of this report.

(ii) By October 31, 1982, PP&L shall submit its proposal for conforming with item II.F.2 of NUREG-0737 in view of the BWR Owners Group report.

(e) Modification of Automatic Depressurization System Logic (II.K.3.18, SER, SSER #1, SSER #2, SSER #3)

(a) By October 1, 1982, PP&L shall evaluate the alternative design modifications of the BWR Owners Group relative to the logic for the automatic depressurization system, submit such evaluation, and propose modifications to the NRC for review and approval.

(b) Prior to startup following the first refueling outage, PP&L shall implement the approved alternative logic modification of the automatic depressurization system.

(f) Effect of Loss of Power on Alternating Current Pump Seals (II.K.3.25, SER, SSER #1)

Prior to startup after the first refueling, PP&L shall provide an emergency power supply to the cooling system for the recirculation pump seals.

(g) Upgrade Emergency Support Facilities (III.A.1.2, SER, SSER#1, SSER#2)

PP&L shall complete its Emergency Response Facilities as follows:

- |                                     |                    |
|-------------------------------------|--------------------|
| (1) Safety Parameter Display System | September 30, 1983 |
| (2) Emergency Operations Facility   | October 1, 1982    |
| (3) Technical Support Center        | October 1, 1982    |

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D. PP&L shall maintain in effect and fully implement all provisions of the Commission approved physical security, and guard training and qualification plans; including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plans, which contain 10 CFR 73.21 information, are collectively entitled: "Susquehanna Steam Electric Station Physical Security Plan" (which includes response to security contingencies as Chapter 11) dated March 14, 1978 with the following changes; Change A dated July 31, 1978, Change B dated February 15, 1979, Change C dated August 15, 1979, Change D dated September 28, 1979, Change E dated May 22, 1980, Change F dated March 27, 1981, Change G dated May 29, 1981, Change H dated June 26, 1981, Change I dated March 19, 1982, Change J dated April 1, 1982, and Change K dated May 4, 1982, Change L dated July 9, 1982, and including Chapter 11 revision dated June 5, 1981; and "Susquehanna Steam Electric Station Security Training and Qualification Plan" dated May 27, 1980, as revised April 30, 1981.

E. Exemptions from certain requirements of Appendices G and H to 10 CFR Part 50 are described in the Safety Evaluation Report and Supplements 1 and 2 to the Safety Evaluation Report. In addition, an exemption was requested until receipt of new fuel for first refueling from the requirements for criticality monitors in the spent fuel pool area, 10 CFR Part 70.24. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

F. This license is subject to the following additional condition for the protection of the environment:

Before engaging in additional construction or operational activities which may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement and its Addendum, PP&L shall provide a written notification to the Director of the Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

G. Reporting to the Commission:

(a) PP&L shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through C(28), and F of this license within twenty-four (24) hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the NRC Regional Administrator, Region I, or designee, not later than the first working day following the violation, with a written followup report within fourteen (14) working days.

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- (b) PP&L shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- H. PP&L shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- I. This license is effective as of the date of issuance and shall expire at midnight on July 17, 2022.

FOR THE NUCLEAR REGULATORY COMMISSION

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Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

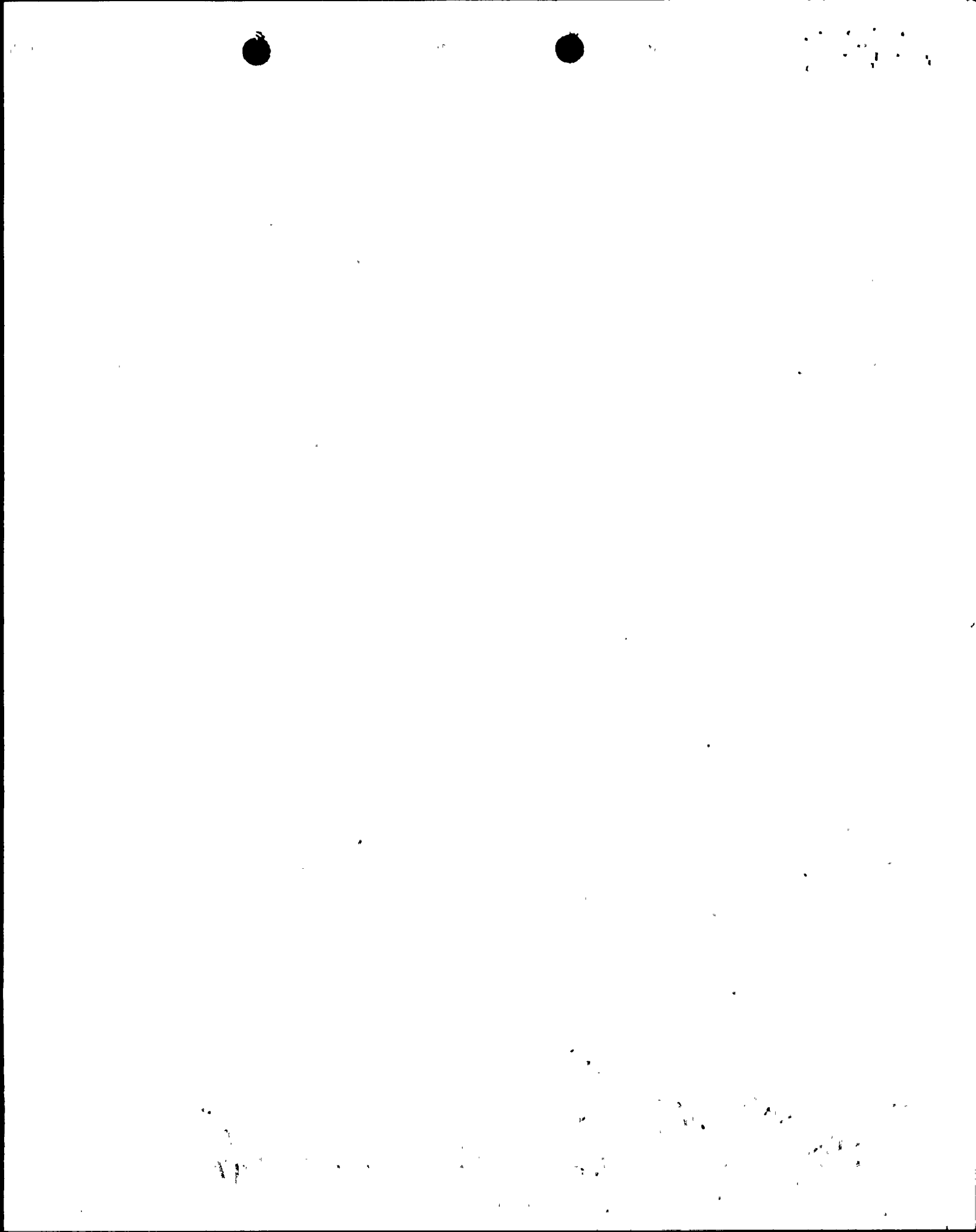
Attachments:

- 1. Attachment 1
- 2. Appendix A - Technical Specifications (NUREG-0931)
- 3. Appendix B - Environmental Protection Plan

Date of Issuance: July 17, 1982

\*SEE TRANSMITTAL LETTER FOR CONCURRENCES.

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ATTACHMENT 1

1. OUTSTANDING ITEM TO BE ACCOMPLISHED PRIOR TO LOADING FUEL

- a. Ground Reactor Protective System Cabling and Cabinetry as stated in Construction Deficiency Report 80-00-28 and conduct necessary testing.

2. OUTSTANDING ITEMS TO BE ACCOMPLISHED BEFORE INITIAL CRITICALITY

- a. Demonstrate recirculation loop riser double weld configuration acceptability.
- b. Demonstrate acceptability of loadings on equipment nozzles and of stress intensification factors on weld components.
- c. Verify and document proper seismic mounting of safety-significant temperature sensors.
- d. Verify and document that the instrumentation supplied by the NSSS vendor has the requisite accuracy in accordance with the design specifications.
- e. Provide for verifying operating activities in accordance with NUREG-0737 item I.C.6 and FSAR Section 18.1.13.
- f. Verify installation of additional post-accident monitoring instrumentation in accordance with NUREG-0737 item II.F.1 and FSAR Section 18.1.30.
- g. Implement a program for reducing leakage from potentially radioactive systems in accordance with NUREG-0737 item III.D.1.1 and FSAR Section 18.1.69.
- h. Verify installation of radioactive Iodine monitoring equipment inplant in accordance with NUREG-0737 item III.D.3.3 and FSAR Section 18.1.70.
- i. Verify that Unit 2 equipment used in Unit 1 is qualified and properly identified.
- j. Complete walkdown of welds requiring in-service-inspection and assure required accessibility has not been compromised by other equipment.
- k. Establish specific controls that assure calibration of equipment required by the Technical Specifications.
- l. Upon issue of the Operating License Technical Specifications, verify that specified conditions, setpoints, and action points in facility procedures are consistent with those Technical Specifications.
- m. Replace deficient Agastat GP relays in safety systems with qualified relays in accordance with the commitment documented in Inspection Report 50-387/82-17 Detail 2.



- n. Demonstrate that stress analyses consider the effect of grouted pipe penetrations and show acceptability of the as-built configuration.
- o. Evaluate vendor-supplied personnel monitoring equipment to assure appropriate equipment is being supplied to personnel in accordance with 10 CFR 20.202.
- p. Establish a personnel neutron exposure monitoring program in accordance with 10 CFR 20.202.
- q. Establish a whole body counting program, including thyroid calibration, in accordance with 10 CFR 20.201.
- r. Establish controls to assure calibration of portable radiation monitoring equipment in accordance with 10 CFR 20.201.

3. OUTSTANDING ITEM TO BE COMPLETED BEFORE EXCEEDING 5% POWER

- a. Correct the Emergency Service Water water hammer reported by Pennsylvania Power and Light Company letter PLA 1129 dated June 18, 1982.



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