

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7909170188 DOC. DATE: 79/09/05 NOTARIZED: NO DOCKET #  
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylv. 05000387  
 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv. 05000388  
 AUTH. NAME: CURTIS, N.W. AUTHOR AFFILIATION: Pennsylvania Power & Light Co.  
 RECIP. NAME: GRIER, B.H. RECIPIENT AFFILIATION: Region I, Philadelphia, Office of the Director

SUBJECT: Interim deficiency rept re pipe support design drawings released for fabrication & installation which lacked revised piping sys stress values developed subsequent to initial stress calculations. Final rept to be submitted by 800301.

DISTRIBUTION CODE: B019S COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2  
 TITLE: Construction Deficiency Report (10CFR50.55E)

NOTES: ~~SEND I&E 3 CYS FSAP & ALL AMPTS L.A. ICM EVERYTHING (ORNL)~~

	RECIPIENT ID CODE/NAME	COPIES		RECIPIENT ID CODE/NAME	COPIES	
		LTTR	ENCL		LTTR	ENCL
ACTION:	14 PM MINER	1	1	15 BC LWR #3	1	1
	20 LA LWR #3	1	1	21 AD LWR	1	1
	INTERNAL: 01 REG FILE	1	1	02 NRC PDR	1	1
	05 I&E	2	2	07 EDO & STAFF	1	1
	08 MPA	1	1	09 DEPY DIR DPM	1	1
	10 QAB	1	1	11 STANDRDS DEV	1	1
	12 JORDAN, E/IE	1	1	13 FIELD COOR/IE	1	1
	16 AD FOR ENGR	1	1	17 AD PLANT SYS	1	1
	17 AD REAC SFTY	1	1	22 DIR DPM	1	1
	23 OELD	1	1	AD/SITE TECH	1	1
	DIRECTOR DSE	1	1	GEOSCIENCES BR	1	1
	I&C SYS BR	1	1	MATL ENG BR	1	1
	MECH ENG BR	1	1	S HANAUER	1	1
	STRUCT ENG BR	1	1			
EXTERNAL:	03 LPDR	1	1	04 NSIC	1	1
	24 ACRS	16	16			

SEP 18 1979

TOTAL NUMBER OF COPIES REQUIRED: LTR 46 ENCL 0

MA  
4



TWO NORTH NINTH STREET, ALLENTOWN, PA. 18101      PHONE: (215) 821-5151

NORMAN W. CURTIS  
Vice President-Engineering & Construction  
821-5381

September 5, 1979

Mr. Boyce H. Grier  
Director, Region I  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

SUSQUEHANNA STEAM ELECTRIC STATION  
INTERIM REPORT OF A DEFICIENCY  
IN PIPE SUPPORT DESIGN DRAWINGS  
RELEASED FOR FABRICATION AND INSTALLATION  
ERS 100450/100508      FILE 840-4  
PLA-391

Dear Mr. Grier:

This serves to confirm information provided to R. Gallo, NRC, Reactor Inspector by A. R. Sabol, PP&L, on August 2, 1979 regarding the subject deficiency which is deemed reportable under 10 CFR 50.55(e).

The deficiency involves discrepancies between the Bechtel Plant Design Group's finalized stress analyses calculations and the loads which were used in the design of pipe support detail drawings which were fabricated and released for installation during the construction of the plant. The released pipe support detail drawings did not have factored into them revised piping system stress values which had been developed subsequent to the initial piping system stress calculations.

All of the safety implications are not known at this time; however, our best estimate is that there will be approximately 1000 pipe supports which may require design changes. Had the deficiencies gone undetected, it is anticipated that some supports could have been overstressed beyond allowable design limits by an order of 100 percent or greater.

The condition was first detected by Bechtel engineering personnel during preparations for piping systems stress walkdowns in the first quarter of 1979. At that time, a review performed on the basis of a random sample indicated that the problem was not generic or widespread. The problem appeared to have been confined to calculations which had been performed prior to mid-1976 and subsequently revised.

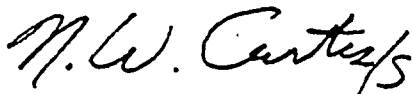
BOIA  
S/10  
1885

September 5, 1979

Subsequently, in preparation for turnover of the feedwater system to startup, a relatively large number of pipe support details, approximately 40 out of 276, which were found to be discrepant. Therefore, in order to facilitate as orderly and complete as possible turnover of systems for startup, a thorough verification program has been instituted. The verification program, which instituted following discovery of the condition, will cover all force, moment calculations and their related pipe support details.

On July 25, 1979, Bechtel QA identified the condition in its MCAR-1-36 requesting that Bechtel engineering evaluate the condition for reportability. Based upon Bechtel engineering's conclusion that the condition is reportable, a final report to MCAR-1-36 will be submitted to PP&L by December 28, 1979. Following, PP&L will submit its final report of said deficiency to the Commission by March 1, 1980.

Very truly yours,



N. W. Curtis  
Vice President-Engineering & Construction

ARS:mcb.

cc: Mr. Robert M. Gallo  
U. S. Nuclear Regulatory Commission  
P. O. Box 52  
Shickshinny, Pennsylvania 18655

Mr. Victor Stello (15)  
Director-Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Mr. G. McDonald  
Office of Management Information & Program Control  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

REGULATORY DOCKET FILE COPY



1950  
1951  
1952  
1953  
1954  
1955  
1956  
1957  
1958  
1959  
1960  
1961  
1962  
1963  
1964  
1965  
1966  
1967  
1968  
1969  
1970  
1971  
1972  
1973  
1974  
1975  
1976  
1977  
1978  
1979  
1980  
1981  
1982  
1983  
1984  
1985  
1986  
1987  
1988  
1989  
1990  
1991  
1992  
1993  
1994  
1995  
1996  
1997  
1998  
1999  
2000  
2001  
2002  
2003  
2004  
2005  
2006  
2007  
2008  
2009  
2010  
2011  
2012  
2013  
2014  
2015  
2016  
2017  
2018  
2019  
2020  
2021  
2022  
2023  
2024  
2025