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 AUTH. NAME: CURTIS, N.W. AUTHOR AFFILIATION: Pennsylvania Power & Light Co.
 RECIP. NAME: SCHWENCER, A. RECIPIENT AFFILIATION: Licensing Branch 2

SUBJECT: Forwards generic BWR safety relief valve test results to confirm adequacy of safety relief valves for plant for all test conditions per 800917 ltr. Final repts on valve qualification & piping/supports will be submitted.

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 TITLE: Response to NUREG -0737/NUREG-0660 TMI Action Plan Rgmts (OL's)

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NORMAN W. CURTIS
Vice President-Engineering & Construction-Nuclear
770-5381

July 1, 1981

Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
SAFETY/RELIEF VALVE TEST PROGRAM PRELIMINARY RESULTS
ER 100450 FILES 204-9,841-12 PLA-865

Dear Mr. Schwencer:

- References:
1. Letter from T. J. Dente, (BWR Owners' Group) to D. G. Eisenhut (NRC), "Transmittal of Preliminary Data from Generic BWR Safety/Relief Valve (SRV) Test Program" dated July 1, 1981
 2. Letter from D. B. Waters (BWR Owners' Group) to R. H. Vollmer (NRC), "NUREG-0578 Requirement 2.1.2-Performance Testing of BWR and PWR Relief and Safety Valves" dated September 17, 1980
 3. Letter from D. B. Waters (BWR Owner's Group) to D. G. Eisenhut (NRC), "Responses to NRC Questions on the BWR SRV Test Program" dated March 31, 1981 (Response to NRC Question No. 1)

In response to NUREG 0737, Item II.D.1, this letter provides the results of a preliminary review of the generic BWR SRV test program results (Reference 1) in order to confirm the adequacy of the SRV's for Susquehanna Plant for all test conditions as defined in the test description (Reference 2).

The Susquehanna Plant employs the Crosby type of SRV, Model No. 6R10. The test results for the Crosby Valve, Model No. 6R10 are applicable to this plant's valves (Reference 3). A preliminary review of the generic BWR SRV test program results demonstrates that the tested valve satisfies the acceptance criteria for operability. Consequently, based on this preliminary review, the operational adequacy of the SRV's for the Susquehanna Plant has been demonstrated.

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Mr. A. Schwencer
Page 2

Final reports on SRV qualification and piping and supports will be submitted as committed in our response to NUREG 0737, requirement II.D.1 (PLA-659).

Very truly yours,



N. W. Curtis
Vice President-Engineering & Construction-Nuclear

DPM/mks

cc: R. M. Stark - NRC