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ACCESSION NBR: 8409110177 DOC. DATE: 84/09/06 NOTARIZED: YES DOCKET #
 FACIL: 50-259 Browns Ferry Nuclear Power Station, Unit 1, Tennessee 05000259
 50-260 Browns Ferry Nuclear Power Station, Unit 2, Tennessee 05000260
 50-296 Browns Ferry Nuclear Power Station, Unit 3, Tennessee 05000296

AUTH. NAME: AUTHORITY AFFILIATION
 MILLS, L.M. Tennessee Valley Authority
 RECIP. NAME: RECIPIENT AFFILIATION
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards background info. & proposed insp. schedule for recirculation inlet safenead thermal sleeve attachment weld, per 840830 meeting request.

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NOTES: NMSS/FCAF 1cy. 1cy NMSS/FCAF/PM. 05000259
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		RGN2		1	1			
EXTERNAL:	ACRS		16	10	10	LPDR		03
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	NTIS			1	1			
NOTES:				2	2			

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

400 Chestnut Street Tower II

September 6, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

In the Matter of the) Docket Nos. 50-259
Tennessee Valley Authority) 50-260
50-296

By letter from D. B. Vassallo to H. G. Parris dated August 22, 1984, we were formally notified of cracks detected in the recirculation system inlet nozzle safe-ends on Peach Bottom Atomic Power Station unit 2. In response to that letter TVA representatives attended a meeting on August 30, 1984 to discuss the situation of the Browns Ferry Nuclear Plant with respect to this identified cracking.

In the meeting your staff requested formal documentation of statements made by TVA representatives present regarding previous and future inspections, and TVA's plans if cracking is detected in upcoming outages. The information requested is enclosed.

It was stated that this information is needed for finalization of the safety evaluation being prepared in support of Browns Ferry unit 3 startup. The enclosed information has been coordinated with your staff.

If you have any questions, please get in touch with us through the Browns Ferry Project Manager.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills
L. M. Mills, Manager
Nuclear Licensing

Subscribed and sworn to before
me this 6th day of Sept 1984.

Paulette N. White
Notary Public
My Commission Expires 8-24-88

Enclosure
cc: See page 2

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PDR ADOCK 05000259
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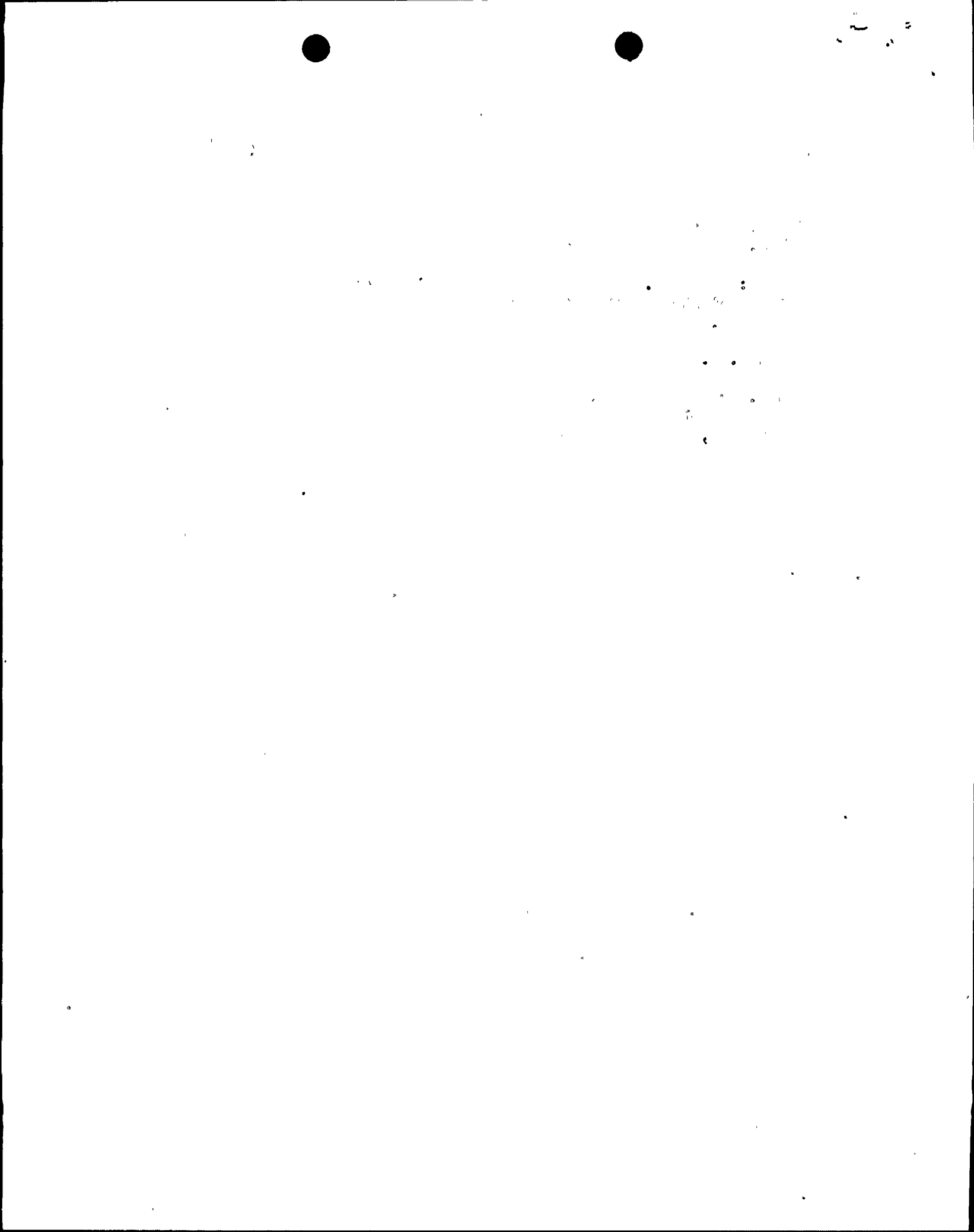
Mr. Harold R. Denton

September 6, 1984

cc (Enclosure):

U.S. Nuclear Regulatory Commission
Region II
ATTN: James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Mr. R. J. Clark
Browns Ferry Project Manager
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, Maryland 20814



ENCLOSURE
BACKGROUND INFORMATION AND
PROPOSED INSPECTION SCHEDULE FOR
RECIRCULATION INLET SAFE-END THERMAL SLEEVE ATTACHMENT WELD
BROWNS FERRY NUCLEAR PLANT

Pursuant to NRC's request in our August 30, 1984 meeting, the following information is submitted for you to forward to NRC for their records.

Units 1, 2, and 3 - Background Information on Subject Safe Ends

Unit 1 - Material: Coulter Steel, Type 316, Carbon: .05-.06 percent, installed by B & W

Unit 2 - Material: Japan Special Steel, Type 316, Carbon: .05-.06 percent, installed by IHI

Unit 3 - Material: Japan Special Steel, Type 316, Carbon: .06-.07 percent, installed by JHI

Past Inspection on Subject Welds

Unit 1 - Inspected 5 of 10 in 1978

Unit 1 - Inspected 2 of 10 in 1980

Unit 2 - Inspected 10 of 10 in 1979

Unit 2 - Inspected 4 of 10 in 1982

Unit 3 - Inspected 5 of 10 in 1978

Future Inspections

Unit 1 - 10 of 10 when removed in spring 1985

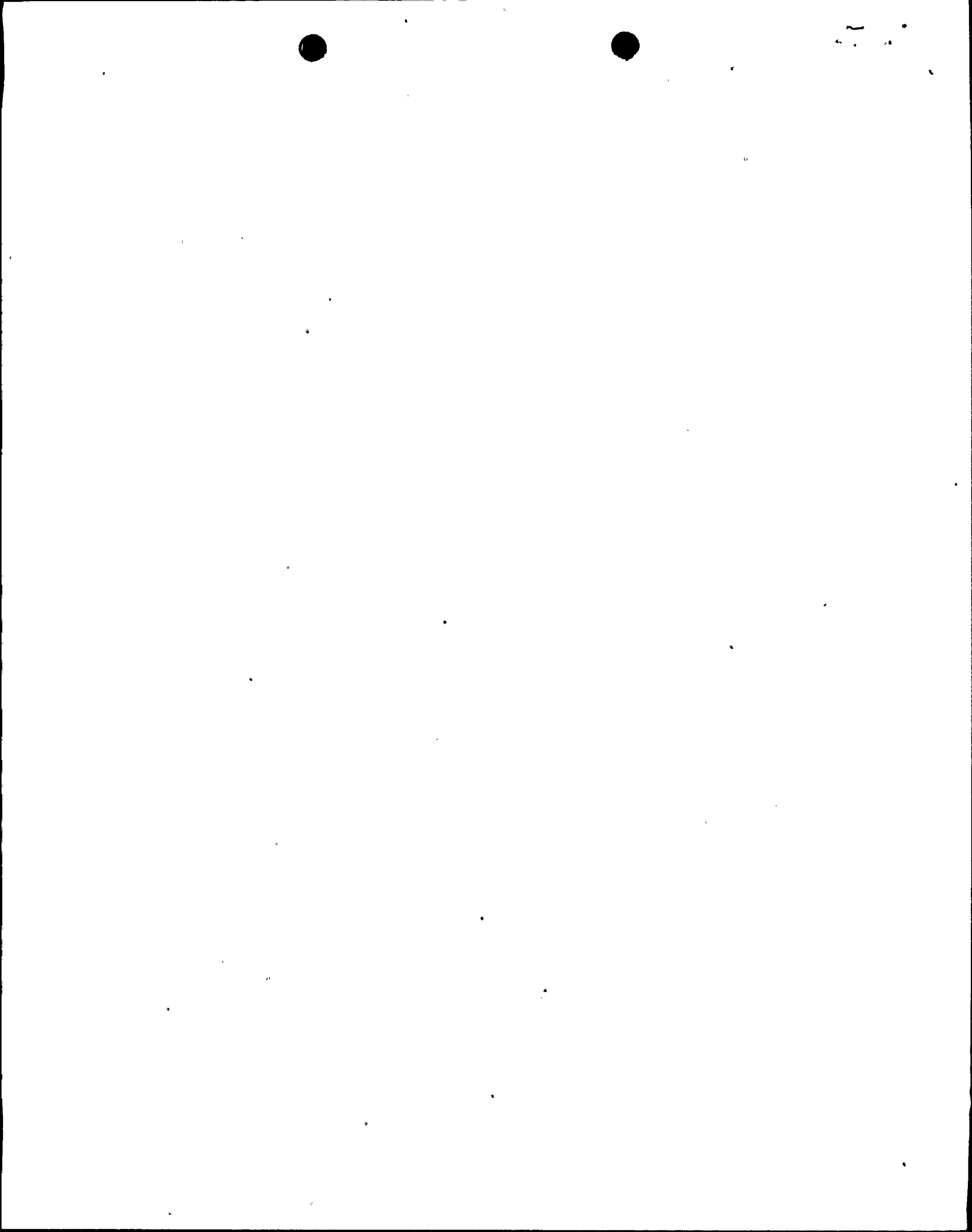
Unit 2 - 5 of 10 during upcoming refuel outage in fall 1984 (2 per heat of material)

Unit 3 - 4 of 10 during upcoming refuel outage in fall 1985 (2 per heat of material)

Inspection Upgrading Contingencies

Since units 2 and 3 are similar (material, vendor, vintage), unit 2 has more operating hours, and unit 3 is relatively free from intergranular stress corrosion cracking on recirculation and allied piping, TVA considers unit 2 to be a conservative representation of the two units.

- If cracking in the subject area is detected on the upcoming inspection of unit 2, the unit 2 sample will be expanded to 100 percent.



- If cracking on unit 2 is minor (i.e., two or more reportable indications not requiring repair), unit 3 will be inspected for similar indications within 180 days.
- If more serious cracking is noted on unit 2 (i.e., one or more indications of cracking requiring repair), unit 3 will be inspected for a similar condition within 60 days.
- If unit 2 or 3 should develop leakage due to cracking in the subject area, the companion unit will be shut down within 14 days to inspect for a similar condition.

