

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8404100286 DOC.DATE: 84/04/02 NOTARIZED: YES DOCKET #
 FACIL:50-260 Browns Ferry Nuclear Power Station, Unit 2, Tennessee 05000260
 AUTH.NAME AUTHOR AFFILIATION
 MILLS,L.M. Tennessee Valley Authority
 RECIPI.NAME RECIPIENT AFFILIATION
 DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests exemption from test interval requirements of
 10CFR50, App J until 840915. Cycle 5 refueling outage
 scheduled to begin on 840824 & continued operation will
 result in exceeding max time interval.

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NOTES: NMSS/FCAF 1cy, 1cy NMSS/FCAF/PM,

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TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

April 2, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

In the Matter of the) Docket Nos. 50-260
Tennessee Valley Authority)

As discussed with R. J. Clark of your staff, we are hereby requesting an exemption from the test interval requirements of 10 CFR 50 Appendix J for the Browns Ferry Nuclear Plant unit 2. Unit 2 is currently scheduled to shutdown for the cycle 5 refueling outage on August 24, 1984. Continued operation to that date will result in the maximum time interval between leak rate tests required by 10 CFR 50 Appendix J to be exceeded on a number of primary containment system components. To support operation to that date we are requesting an extension of the permissible test interval.

The list of components for which an extension of the test interval will be required is provided as an enclosure to this letter. Included in the list is the surveillance interval end date for each component.

We are requesting extension of the interval to September 15, 1984. This will allow for unanticipated delays in starting the unit 2 cycle 5 outage.

We request approval of this exemption request as soon as possible, but no later than July 20, 1984. Your cooperation in this matter is greatly appreciated.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills
L. M. Mills, Manager
Nuclear Licensing

Subscribed and sworn to before
me this 2nd day of April 1984.

Paulette H. White
Notary Public
My Commission Expires 9-5-84

Enclosure
cc: See page 2

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PDR ADOCK 05000260
P PDR



Mr. Harold R. Denton

April 2, 1984

cc (Enclosure):

U.S. Nuclear Regulatory Commission
Region II
ATTN: James P. O'Reilly, Regional Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

Mr. R. J. Clark
Browns Ferry Project Manager
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, Maryland 20814

ENCLOSURE

UNIT 2 PRIMARY CONTAINMENT SYSTEM

COMPONENTS REQUIRING AN EXTENSION OF TEST INTERVAL
BROWNS FERRY NUCLEAR PLANT

IB = Inboard
OB = Outboard

Component	Number	Description	Expiration Date
Bellows	X-7A IB	Primary Steamline	9-7-84
Bellows	X-7A OB	Primary Steamline	9-7-84
Bellows	X-7B IB	Primary Steamline	9-7-84
Bellows	X-7B OB	Primary Steamline	9-7-84
Bellows	X-7C IB	Primary Steamline	9-6-84
Bellows	X-7C OB	Primary Steamline	9-7-84
Bellows	X-7D IB	Primary Steamline	9-6-84
Bellows	X-7D OB	Primary Steamline	9-6-84
Bellows	X-8 IB	Primary Steamline Drain	9-7-84
Bellows	X-8 OB	Primary Steamline Drain	9-7-84
Bellows	X-9A IB	Feedwater Line	9-6-84
Bellows	X-9A OB	Feedwater Line	9-6-84
Bellows	X-9B IB	Feedwater Line	9-6-84
Bellows	X-9B OB	Feedwater Line	9-6-84
Bellows	X-10 IB	Steamline to RCIC Turbine	9-6-84
Bellows	X-10 OB	Steamline to RCIC Turbine	9-6-84
Bellows	X-11 IB	Steamline to HPCI Turbine	9-7-84
Bellows	X-11 OB	Steamline to HPCI Turbine	9-7-84
Bellows	X-12 IB	RHR Shutdown Supply Line	9-7-84
Bellows	X-12 OB	RHR Shutdown Supply Line	9-7-84
Bellows	X-13A IB	RHR Return Line	9-7-84
Bellows	X-13A OB	RHR Return Line	9-7-84
Bellows	X-13B IB	RHR Return Line	9-7-84
Bellows	X-13B OB	RHR Return Line	9-7-84
Bellows	X-14 IB	Reactor Water Cleanup Line	9-7-84
Bellows	X-14 OB	Reactor Water Cleanup Line	9-7-84
Bellows	X-16A IB	Core Spray Line	9-7-94
Bellows	X-16A OB	Core Spray Line	9-7-94
Bellows	X-16B IB	Core Spray Line	9-7-94
Bellows	X-16B OB	Core Spray Line	9-7-94
Bellows	X-17 IB	RHR Head Spray Line	9-7-84
Bellows	X-17 OB	RHR Head Spray Line	9-7-84



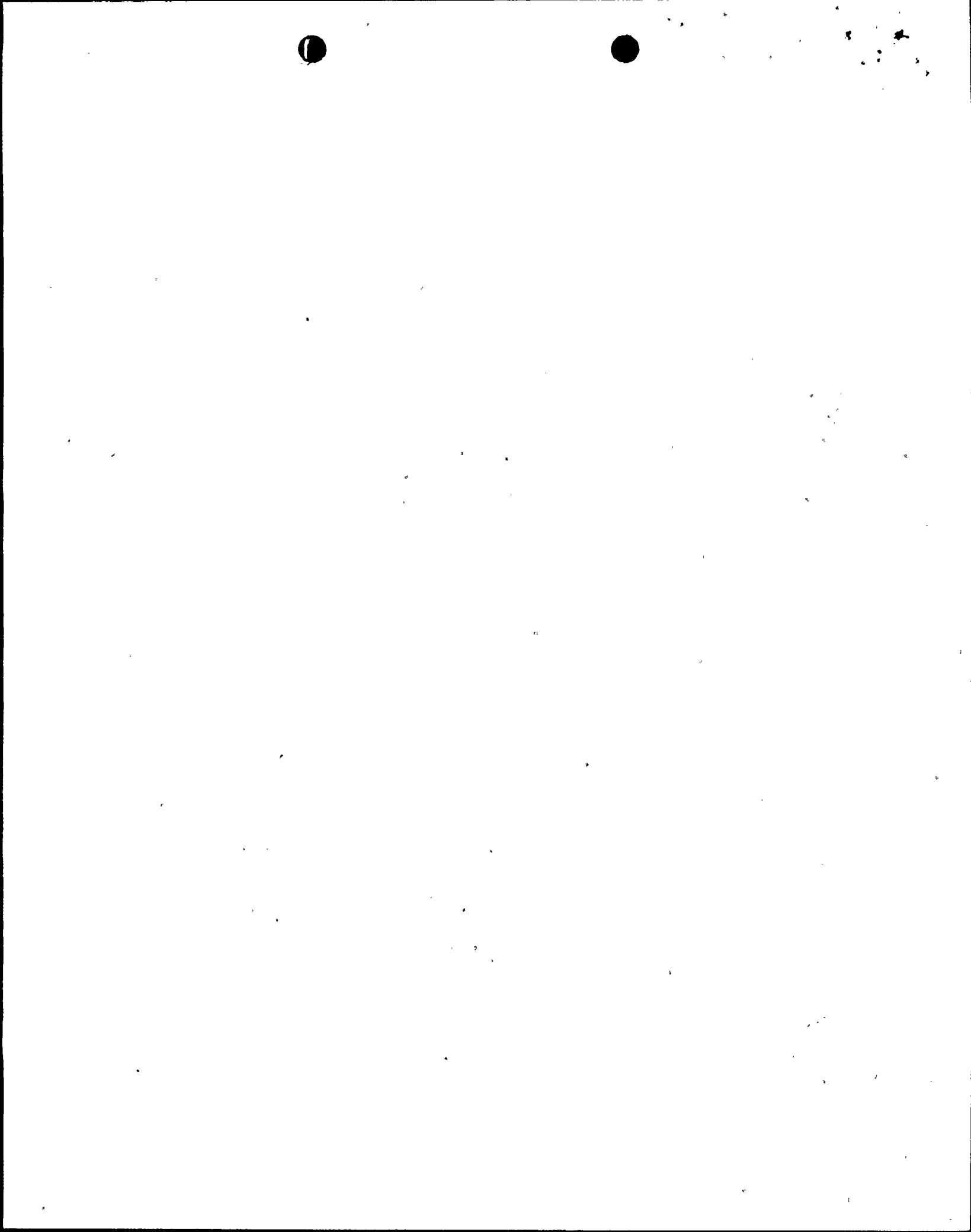
UNIT 2 PRIMARY CONTAINMENT SYSTEM

COMPONENTS REQUIRING AN EXTENSION OF TEST INTERVAL

Component	Number	Description	Expiration Date
Electrical Penetration	X-100 A	Indication and Control	8-12-84
Electrical Penetration	X-100 B	Neutron Monitoring	8-12-84
Electrical Penetration	X-100 C	Neutron Monitoring	8-13-84
Electrical Penetration	X-100 D	Neutron Monitoring	8-13-84
Electrical Penetration	X-100 E	Neutron Monitoring	8-13-84
Electrical Penetration	X-100 F	Neutron Monitoring	8-13-84
Electrical Penetration	X-100 G	CRD Rod Position Indication	8-12-84
Electrical Penetration	X-101 A	Recirculation Pump Power	9-4-84
Electrical Penetration	X-101 B	Recirculation Pump Power	9-4-84
Electrical Penetration	X-101 C	Recirculation Pump Power	9-4-84
Electrical Penetration	X-101 D	Recirculation Pump Power	9-4-84
Electrical Penetration	X-102	Thermocouples	8-12-84
Electrical Penetration	X-103	CRD Rod Position Indication	8-13-84
Electrical Penetration	X-104 A	Indication and Control	8-12-84
Electrical Penetration	X-104 B	CRD Rod Position Indication	8-12-84
Electrical Penetration	X-104 C	Neutron Monitoring	8-12-84
Electrical Penetration	X-104 D	Thermocouples	9-9-84
Electrical Penetration	X-104 E	Indication and Control	8-13-84
Electrical Penetration	X-104 F	Indication and Control	8-13-84
Electrical Penetration	X-105 B	Recirculation Pump Power	9-4-84
Electrical Penetration	X-105 C	Recirculation Pump Power	9-4-84
Electrical Penetration	X-105 D	Spare	9-9-84
Electrical Penetration	X-106 A	CRD Rod Position Indication	8-12-84
Electrical Penetration	X-106 B	Neutron Monitoring	8-12-84
Electrical Penetration	X-107 A	Neutron Monitoring	8-12-84
Electrical Penetration	X-107 B	Spare	8-12-84
Electrical Penetration	X-108 A	Power	9-6-84
Electrical Penetration	X-108 B	CRD Rod Position Indication	8-13-84
Electrical Penetration	X-109	CRD Rod Position Indication	8-12-84
Electrical Penetration	X-110 A	Power	9-4-84
Electrical Penetration	X-110 B	CRD Rod Position Indication	8-13-84
Electrical Penetration	X-219	Vacuum Breaker Inst.	9-4-84

UNIT 2 PRIMARY CONTAINMENT SYSTEM
COMPONENTS REQUIRING AN EXTENSION OF TEST INTERVAL

Component	Number	Description	Expiration Date
Double O-Ring Seal	X-35 D	T. I. P. Drive	9-5-84
Double O-Ring Seal	X-35 E	T. I. P. Drive	9-5-84
Double O-Ring Seal	X-35 G	T. I. P. Drive	9-5-84
Double O-Ring Seal	X-47	Power Operations Test	9-7-84
Double O-Ring Seal	1	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	2	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	3	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	4	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	5	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	6	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	7	Shear Lug Inspec. Cover Hatch	9-5-84
Double O-Ring Seal	8	Shear Lug Inspec. Cover Hatch	9-5-84
Valve	2-1143	Demineralized Water	9-7-84
Valve	2-1192	Service Water	9-8-84
Valve	2-1383	Service Water	9-8-84
Valve	12-738	Auxiliary Boiler to RCIC	8-11-84
Valve	12-742	Auxiliary Boiler to RCIC	8-11-84
Valve	32-62	Drywell Compressor Suction	9-4-84
Valve	32-63	Drywell Compressor Suction	9-4-84
Valve	32-336	Drywell Compressor Return	9-3-84
Valve	33-785	Service Air	9-3-84
Valve	33-1070	Service Air	9-3-84
Valve	43-13	Reactor Water Sample Line	9-5-84
Valve	43-14	Reactor Water Sample Line	9-5-84
Valve	43-28 A	RHR Suppression Chamber Sample Line	9-2-84
Valve	43-28 B	RHR Suppression Chamber Sample Line	9-2-84
Valve	43-29 A	RHR Suppression Chamber Sample Line	9-2-84
Valve	43-29 B	RHR Suppression Chamber Sample Line	9-2-84
Valve	63-525	Standby Liquid Control Discharge	9-5-84
Valve	63-526	Standby Liquid Control Discharge	9-5-84
Valve	71-2	RCIC Steam Supply	9-4-84
Valve	71-3	RCIC Steam Supply	9-4-84
Valve	71-32	RCIC Vacuum Pump Discharge	8-10-84
Valve	71-592	RCIC Vacuum Pump Discharge	8-10-84
Valve	73-2	HPCI Steam Supply	9-2-84
Valve	73-3	HPCI Steam Supply	9-2-84
Valve	73-81	HPCI Steam Supply Bypass	9-2-84
Valve	73-24	HPCI Turbine Exhaust Drain	8-12-84



UNIT 2 PRIMARY CONTAINMENT SYSTEM

COMPONENTS REQUIRING AN EXTENSION OF TEST INTERVAL

Component	Number	Description	Expiration Date
Valve	73-609	HPCI Turbine Exhaust Drain	8-12-84
Valve	74-54	RHR LPCI Discharge	9-2-84
Valve	74-67	RHR LPCI Discharge	9-2-84
Valve	74-68	RHR LPCI Discharge	9-2-84
Valve	74-71	RHR Suppression Chamber Spray	9-2-84
Valve	74-72	RHR Suppression Chamber Spray	9-2-84
Valve	74-74	RHR Drywell Spray	9-2-84
Valve	74-75	RHR Drywell Spray	9-2-84
Valve	75-25	Core Spray Discharge	9-4-84
Valve	75-26	Core Spray Discharge	9-4-84
Valve	75-53	Core Spray Discharge	9-4-84
Valve	75-54	Core Spray Discharge	9-4-84
Valve	75-57	Core Spray to Auxiliary Boiler	9-4-84
Valve	75-58	Core Spray to Auxiliary Boiler	9-4-84
Valve	76-17	Drywell/Suppression Chamber N ₂ Purge	8-11-84
Valve	76-18	Drywell N ₂ Purge Inlet	8-11-84
Valve	76-19	Suppression Chamber Purge Inlet	8-11-84
Valve	76-49	Containment Atmospheric Monitor	8-13-84
Valve	76-50	Containment Atmospheric Monitor	8-13-84
Valve	76-51	Containment Atmospheric Monitor	8-13-84
Valve	76-52	Containment Atmospheric Monitor	8-13-84
Valve	76-53	Containment Atmospheric Monitor	9-3-84
Valve	76-55	Containment Atmospheric Monitor	9-3-84
Valve	76-57	Containment Atmospheric Monitor	9-3-84
Valve	76-59	Containment Atmospheric Monitor	8-13-84
Valve	76-60	Containment Atmospheric Monitor	8-13-84
Valve	76-61	Containment Atmospheric Monitor	8-13-84
Valve	76-62	Containment Atmospheric Monitor	8-13-84
Valve	76-67	Containment Atmospheric Monitor	9-3-84
Valve	84-8B	Containment Atmospheric Dilution	8-11-84
Valve	84-8C	Containment Atmospheric Dilution	8-11-84
Valve	84-8D	Containment Atmospheric Dilution	8-11-84
Valve	84-601	Containment Atmospheric Dilution	8-11-84
Valve	84-602	Containment Atmospheric Dilution	8-11-84
Valve	84-603	Containment Atmospheric Dilution	8-11-84
Valve	90-254 A	Radiation Monitor Suction	9-3-84
Valve	90-254 B	Radiation Monitor Suction	9-3-84
Valve	90-255	Radiation Monitor Discharge	9-3-84
Valve	90-257 A	Radiation Monitor Discharge	9-3-84
Valve	90-257 B	Radiation Monitor Discharge	9-3-84

CM:WSH
3/5/84
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