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 AUTH. NAME AUTHOR AFFILIATION
 CURTIS, N.W., Pennsylvania Power & Light Co.
 RECIPIENT NAME RECIPIENT AFFILIATION
 YOUNGBLOOD, B.J. Licensing Branch 1

SUBJECT: Forwards containment steam bypass calculation, high pressure diaphragm leakage test & results of analysis for wetwell spray termination of pressure increase, per NRC requests.

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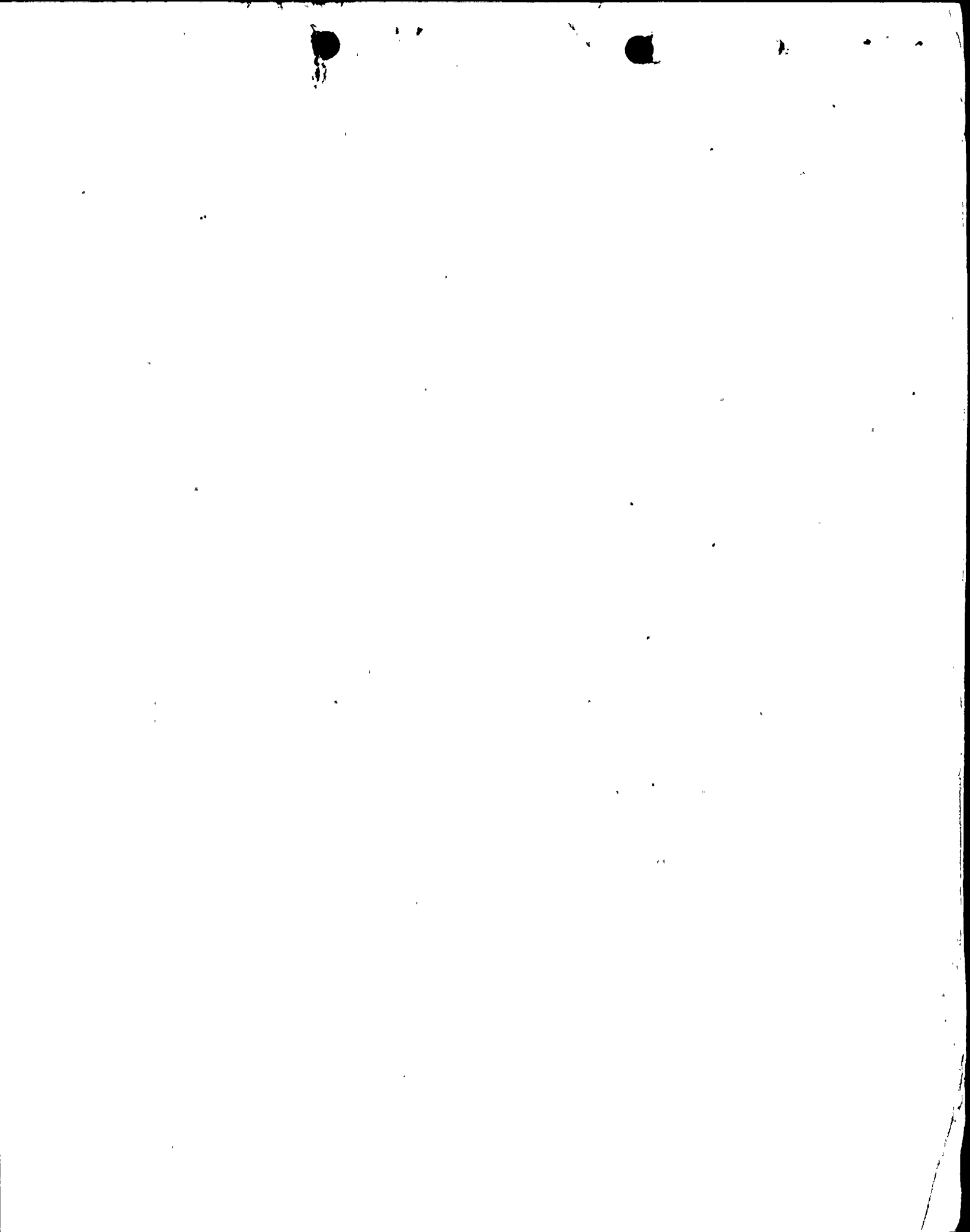
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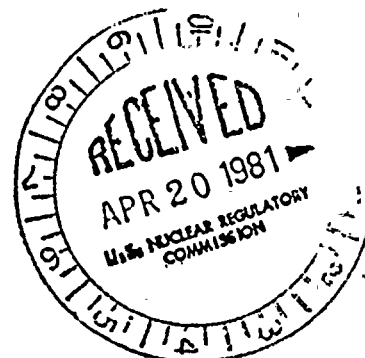


PP&L

TWO NORTH NINTH STREET, ALLENTOWN, PA. 18101. PHONE: (215) 821-5151

April 16, 1981

Mr. B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



SUSQUEHANNA STEAM ELECTRIC STATION
CONTAINMENT SYSTEMS BRANCH CONCERNS
ER 100450 FILE 841-2
PLA-742

Attachment 1 contains a Containment Steam Bypass calculation requested by the Containment Systems Branch at the March 9 & 10, 1981 meeting between NRC and PP&L. This calculation assumes an initial wetwell pressure of 30 psig and shows approximately 27 minutes available without operator initiated spray before wetwell pressure reaches 53 psig design pressure. This calculation is conservative as it does not take credit for heat sinks which would be expected to add 10-20 minutes to the 27 minutes calculated. This calculation supports PP&L's position that automatic wetwell spray initiation is not required.

Attachment 2 addresses the high pressure diaphragm leakage test required by SRP 6.2.1.1.C Appendix I. This conservative calculation shows that the maximum leakage possible during the SSES Structural Integrity Test was 6.5% as compared to the 10% maximum acceptance criteria in the SRP. The pressure during the SSES SIT was 33 psid as compared to a maximum SSES LOCA ΔP of 23 psid. Based on this analysis, PP&L feels that an additional high pressure test is not required.

Attachment 3 contains the results of an analysis for Wetwell Spray Termination of Pressure Increase as requested by the Containment Systems Branch.

Very truly yours,

N. W. Curtis

N. W. Curtis
Vice President-Engineering & Construction-Nuclear

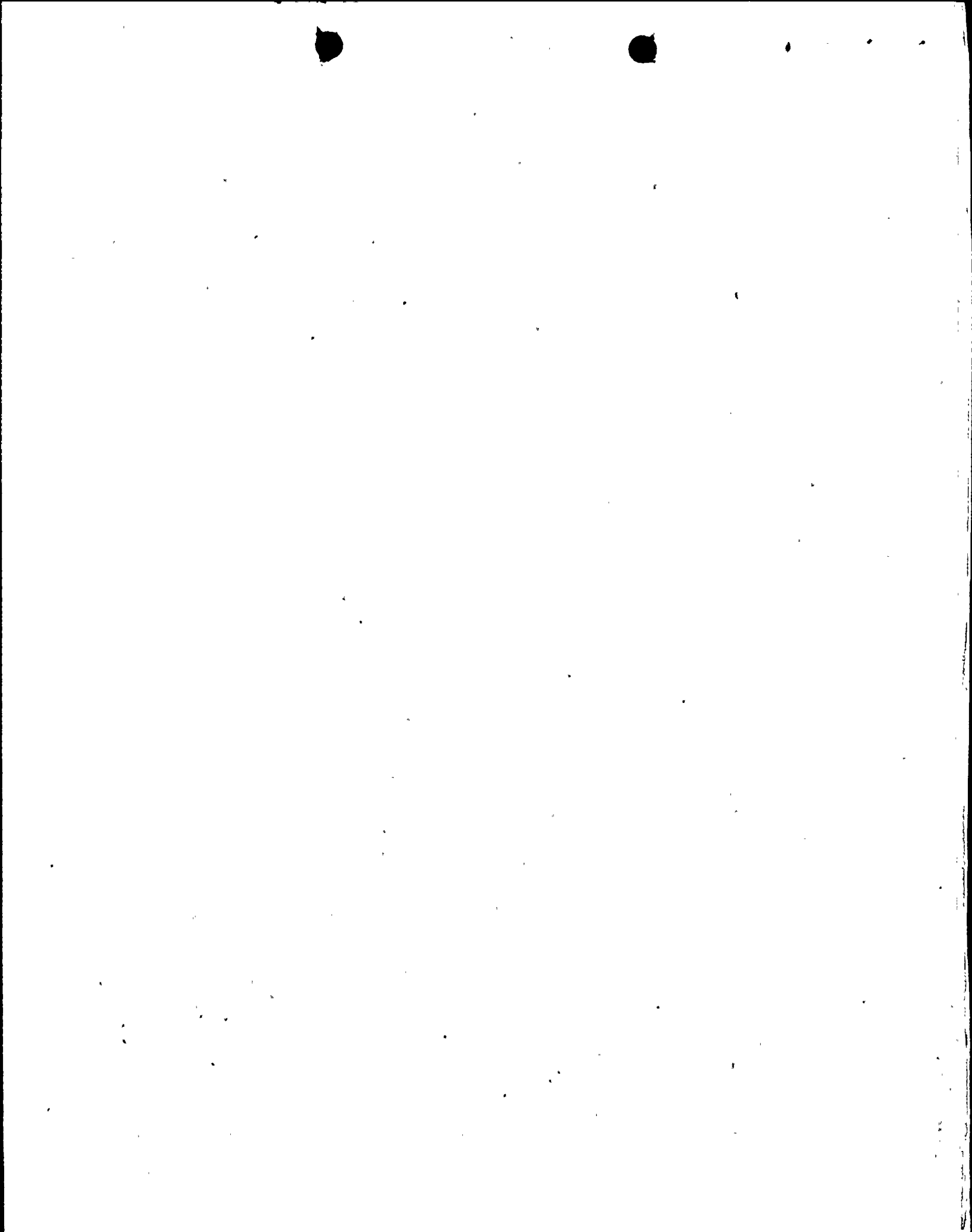
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Attachments

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ATTACHEMENT 1 TO PLA 742

