

JUL 08 1981

APPENDIX A

NOTICE OF VIOLATION

Tennessee Valley Authority  
Browns Ferry 1, 2, and 3

Docket Nos. 50-259, 50-260 and 50-296  
License Nos. DPR-33, DPR-52 and DPR-68

As a result of the inspection conducted on April 28, through May 28<sup>5</sup>, 1981, and in accordance with the Interim Enforcement Policy, 45 FR 66754 (October 7, 1980), the following violations were identified.

- A. Technical Specification 3.5.E.1 requires that the high pressure coolant injection system shall be operable whenever there is irradiated fuel in the reactor vessel and the reactor vessel pressure is greater than 122 psig. Operating Instruction 57 requires the HPCI injection valve (73-44) to be open for the HPCI system to be considered operable when a unit battery is out of service.

Contrary to the above, the HPCI system was inoperable in that the HPCI injection valve (73-44) was not opened when a unit battery was removed from service at 1:16 p.m. on May 5, 1981, nor was the system declared inoperable until 27 hours after unit battery was removed from service for a load test. This violation continued for about 26 hours after our inspector informed plant supervision of the violation.

This is a Severity Level IV Violation (Supplement I.D.3) applicable to Unit 3.

- B. Technical Specification 3.11.A.1.b requires that the high pressure fire protection system shall have automatic initiation logic operable.

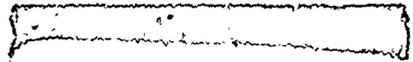
Contrary to the above, the requirement for automatic initiation logic to be operable was not met in that on May 1, 1981 it was determined by the licensee that the automatic start feature for the fire pumps was inoperable and had been in this condition since sometime after April 17, 1981. This condition was caused by leads being lifted in the automatic start circuit during plant modifications.

This is a Severity Level V Violation (Supplement I.E.).

- C. Technical Specification 6.3.A.6 requires detailed written procedures be prepared, approved and adhered to for surveillance and operations of systems and components involving nuclear safety.

1. Contrary to the above, written procedures were not adhered to in that the HPCI discharge valve (73-44) was not opened for Unit 3 as required by the prerequisite section of Surveillance Instruction 4.9.A.2.c during a battery discharge test on May 5-6, 1981.

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2. Contrary to the above, a written procedure, which specified switch position (Operating Instruction 31) was not followed for the operation of the control room emergency pressurization unit (CREU) in that on May 20, 1981 at 8:45 a.m., the "B" CREU unit would not start automatically on an accident signal because of an improper switch lineup.

This is a Severity Level V Violation (Supplement I.E.).

Pursuant to the provisions of 10 CFR 2.201, you are hereby required to submit to this office within thirty days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2) the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved. Consideration may be given to extending your response time for good cause shown. Under the authority of Section 182 of the Atomic Energy Act of 1954, as amended, this response shall be submitted under oath or affirmation.

Date: JUL 08 1981

