

TENNESSEE VALLEY AUTHORITY
DIVISION OF POWER PRODUCTION
BROWNS FERRY NUCLEAR PLANT

MONTHLY OPERATING REPORT
August 1, 1980 - August 31, 1980

DOCKET NUMBERS 50-259, 50-260, AND 50-296
LICENSE NUMBERS DPR-33, DPR-52, AND DPR-68

Submitted By: *Robert Clucum*

Plant Manager

8009170 277



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Operations Summary

August 1980

The following summary describes the significant operational activities during the reporting period. In support of this summary, a chronological log of significant events is included in this report.

There were eleven reportable occurrences reported to the NRC during the month of August..

Unit 1

There were no scrams on the unit during the month.

Unit 2

There was one scram on the unit during the month. On August 16, the reactor scrambled from closure of the MSIV's caused by the PCIS not being reset prior to testing the second switch during a surveillance instruction (resulting in a full isolation signal).

Unit 3

There were no scrams on the unit during the month.

Operations Summary (Continued)

August 1980

Fatigue Usage Evaluation

The cumulative usage factors for the reactor vessel are as follows:

<u>Location</u>	<u>Usage Factor</u>		
	<u>Unit 1</u>	<u>Unit 2</u>	<u>Unit 3</u>
Shell at water line	0.00485	0.00368	0.00308
Feedwater nozzle	0.22994	0.15706	0.11326
Closure studs	0.19498	0.12571	0.09060

Note: This accumulated monthly information satisfies technical specification section 6.6.A.17.B(3) reporting requirements.

Common System

Approximately $1.38E+05$ gallons of waste liquid were discharged containing approximately $7.96E-02$ curies of activity.

Refueling Information

July 1980

Unit 1

Unit 1 is scheduled for its fourth refueling beginning on or about April 15, 1981, with a scheduled restart date of July 25, 1981. This refueling will involve loading additional 8 x 8 R (retrofit) fuel assemblies into the core, the final fix on the sparger modification, power supply on LPCI modification, and torus modifications if all approvals are received.

There are 764 fuel assemblies in the reactor vessel. The spent fuel storage pool presently contains 550 spent 7 x 7 fuel assemblies, five 8 x 8 fuel assemblies, and one 8 x 8 R fuel assembly. Because of modification work to increase spent fuel pool capacity to 3471 assemblies, present capacity is limited to 1152 assemblies.

Unit 2

Unit 2 is scheduled for its third refueling beginning on or about September 5, 1980, with a scheduled restart date of October 16, 1980. Resumption of operation on that date will require a change in technical specifications pertaining to the core thermal limits. Licensing information in support of these changes will be submitted to the NRC before the outage. This refueling involves replacing some more 7 x 7 fuel assemblies with 8 x 8 R (retrofit) assemblies, and power supply on LPCI modification.

There are 764 fuel assemblies in the reactor vessel. At the end of the month, there were 132 discharged cycle 1 fuel assemblies, 268 discharged cycle 2 fuel assemblies, and 240 new p 8 x 8 R assemblies in the spent fuel storage pool. The present storage capacity of the spent fuel pool is 820 assemblies. With present capacity, the 1979 refueling was the last

Refueling Information (Continued)

July 1980

Unit 2 (Continued)

refueling that could be discharged to the spent fuel pool without exceeding that capacity and maintaining full core discharge capability in the pool. However, new high density storage racks are being installed prior to the upcoming third refueling of Unit 2.

Unit 3

Unit 3 is scheduled for its third refueling beginning on or about October 17, 1980, with a scheduled restart date of November 25, 1980. This EOC-3 refueling, involves loading additional 8 x 8 R (retrofit) assemblies into the core, and power supply on LPCI modification.

There are 764 fuel assemblies presently in the reactor vessel. There are 144 discharged cycle 2 fuel assemblies, 208 discharged cycle 1 fuel assemblies, and 164 new P 8 x 8 R assemblies in the spent fuel storage pool. The present storage capacity of the spent fuel pool is 1528 assemblies. With present capacity, the 1980 refueling would be the last refueling that could be discharged to the spent fuel pool without exceeding that capacity and maintaining full core discharge capability in the pool.

Significant Operational Events

Unit 1

Date	Time	Event
8/01/80	0001	Reactor thermal power at 98%, maximum flow - control rod sequence "A".
8/04/80	0300	Reduced thermal power from 98% to 85% for turbine C. V. tests and SI's.
	0345	Increased thermal power to 90%, holding for control rod exercise SI.
	0405	Turbine C. V. tests, SI's, and control rod exercise. SI completed, commenced power ascension.
	0700	Reactor thermal power at 98%, maximum flow.
	1400	Reduced thermal power from 98% to 85% due to condenser high back pressure.
	1440	Commenced power ascension.
	1500	Reactor thermal power at 98%, maximum flow.
8/09/80	2205	Reduced thermal power from 98% to 52% for removal of "B" recirculation pump MG set from service for brush replacement.
8/10/80	0110	"B" recirculation pump MG set brush replacement completed and pump placed in service, holding at 52% thermal power for removal of "A" recirculation pump MG set from service for brush replacement.
	0125	"A" recirculation MG set removed from service for brush replacement.
	0235	"A" recirculation pump MG set brush replacement completed and pump placed in service, commenced power ascension.
	0500	Commenced PCIOMR from 65% thermal power - control rod sequence "A".
8/11/80	1500	Reactor thermal power at >99%, steady state.
8/12/80	0400	Thermal power reduced from >99% to 95% as a result of loss of preferred power.
	0930	Commenced PCIOMR from 95% thermal power - control rod sequence "A".
	1243	Reactor thermal power at >99%, steady state.
8/16/80	0040	Reduced thermal power from >99% to 80% for turbine C. V. tests and SI's.
	0320	Turbine C. V. tests and SI's completed, commenced power ascension.
	0400	Commenced PCIOMR from 90% thermal power - control rod sequence "A".
	1500	Reactor thermal power at >99%, steady state.
8/18/80	2202	Reduced thermal power from >99% to 75% for control rod 18-39 solenoid maintenance and control rod pattern adjustment.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
8/19/80	0210	Control rod 18-39 maintenance and control rod pattern adjustment completed, commenced power ascension.
	0530	Reactor thermal power at >99%, steady state.
8/22/80	2300	Reduced thermal power from >99% to 53% for control rod pattern adjustment, turbine C. V. tests and SI's.
8/23/80	0200	Control rod pattern adjustment, turbine C. V. tests and SI's completed, commenced power ascension.
	0230	Commenced PCIOMR from 70% thermal power - control rod sequence "A".
8/24/80	2300	Reactor thermal power 99%, maximum flow.
8/30/80	0015	Reduced thermal power from 99% to 80% for turbine C. V. tests and SI's.
	0240	Turbine C. V. tests and SI's completed, commenced power ascension.
	0700	Reactor thermal power at >99%, steady state.
8/31/80	2400	Reactor thermal power at >99%, steady state.

Significant Operational Events

Unit 2

Date	Time	Event
8/01/80	0001	Reactor thermal power at 45% with power ascension in progress.
	0130	Reduced thermal power from 60% to 50% for control rod pattern adjustment.
	0300	Control rod pattern adjustment completed, commenced power ascension.
	1330	Commenced PCIOMR from 68% thermal power - control rod sequence "B".
8/03/80	0700	Reactor thermal power at 96%, maximum flow.
8/04/80	1358	Reduced thermal power from 96% to 75% due to loss of plant preferred power and recirculation pumps run back.
	1540	Commenced power ascension.
	1545	Reactor thermal power at 95%, maximum flow.
8/05/80	2222	Reduced thermal power from 95% to 55% for control rod pattern adjustment.
8/06/80	0105	Control rod pattern adjustment completed, commenced power ascension, control rod sequence "B".
	0330	Commenced PCIOMR from 70% thermal power.
	0720	Reactor thermal power at 75%, holding for removal of "A" reactor feed pump from service for maintenance (replaced vibration pickup).
	1055	"A" reactor feed pump maintenance completed and pump placed in service, holding for removal of "B" reactor feed pump from service for maintenance (replaced vibration pickup).
	1320	"B" reactor feed pump maintenance completed and pump placed in service, commenced power ascension.
	1630	Commenced PCIOMR from 78% thermal power.
	8/07/80	1800
8/10/80	0205	Reduced thermal power from 98% to 85% for turbine C.V. tests and SI's.
	0320	Turbine C. V. tests and SI's completed, commenced power ascension.
	0700	Reactor thermal power at 98%, maximum flow.
8/11/80	2217	"B" recirculation pump ran back, due to electrical switching, reducing thermal power from 98% to 93%.
	2235	"B" recirculation pump run back reset, commenced PCIOMR from 93% thermal power.

Significant Operational Events

Unit 2

Date	Time	Event
8/12/80	2320	Reactor thermal power at 97%, maximum flow.
8/15/80	2340	Reduced thermal power from 97% to 80% for turbine C.V. tests and SI's.
8/16/80	0155	Turbine C. V. tests and SI's completed, commenced power ascension.
	0539	Reactor Scram No. 108 ⁽¹⁾ from 95% thermal power while performing SI 4.2.A-8 (MSL Tunnel High Temperature).
	1322	Commenced rod withdrawal.
	1625	Reactor Critical No. 118, sequence "B".
	1933	Rolled T/G.
	1955	Synchronized generator, commenced power ascension.
8/17/80	0704	Commenced PCIOMR from 80% thermal power - control rod sequence "B".
8/18/80	0600	Reactor thermal power at 97%, maximum flow.
8/19/80	0040	Reduced thermal power from 97% to 80% for control rod pattern adjustment.
	0115	Control rod pattern adjustment completed, commenced PCIOMR from 80% thermal power, moving toward all rod out condition.
	1500	Reactor thermal power at 94%, maximum flow, EOC 3 coast down.
8/22/80	1500	Reactor thermal power at 95%, maximum flow.
8/26/80	2310	Reduced thermal power from 92% to 75% for removal of "C" RFP from service for Special Test No. 185 (vibration checks).
8/27/80	0907	ST 185 (vibration checks) completed and "C" RFP placed in service, commenced power ascension.
	1000	Commenced PCIOMR from 81% thermal power, moving toward an all rods out condition.
8/28/80	1050	Reactor thermal power at 94%, maximum flow.
8/29/80	2300	Reactor thermal power at 92%, EOC-3 coast down.
	2342	Reduced thermal power from 92% to 80% for turbine C. V. tests and SI's.
8/30/80	0230	Turbine C. V. tests and SI's completed, commenced power ascension.
	0700	Commenced PCIOMR from 85% thermal power.
	2300	Reactor thermal power at 92%.
8/31/80	2400	Reactor thermal power at 92%, EOC-3 coast down continues.



Significant Operational Events

Unit 3

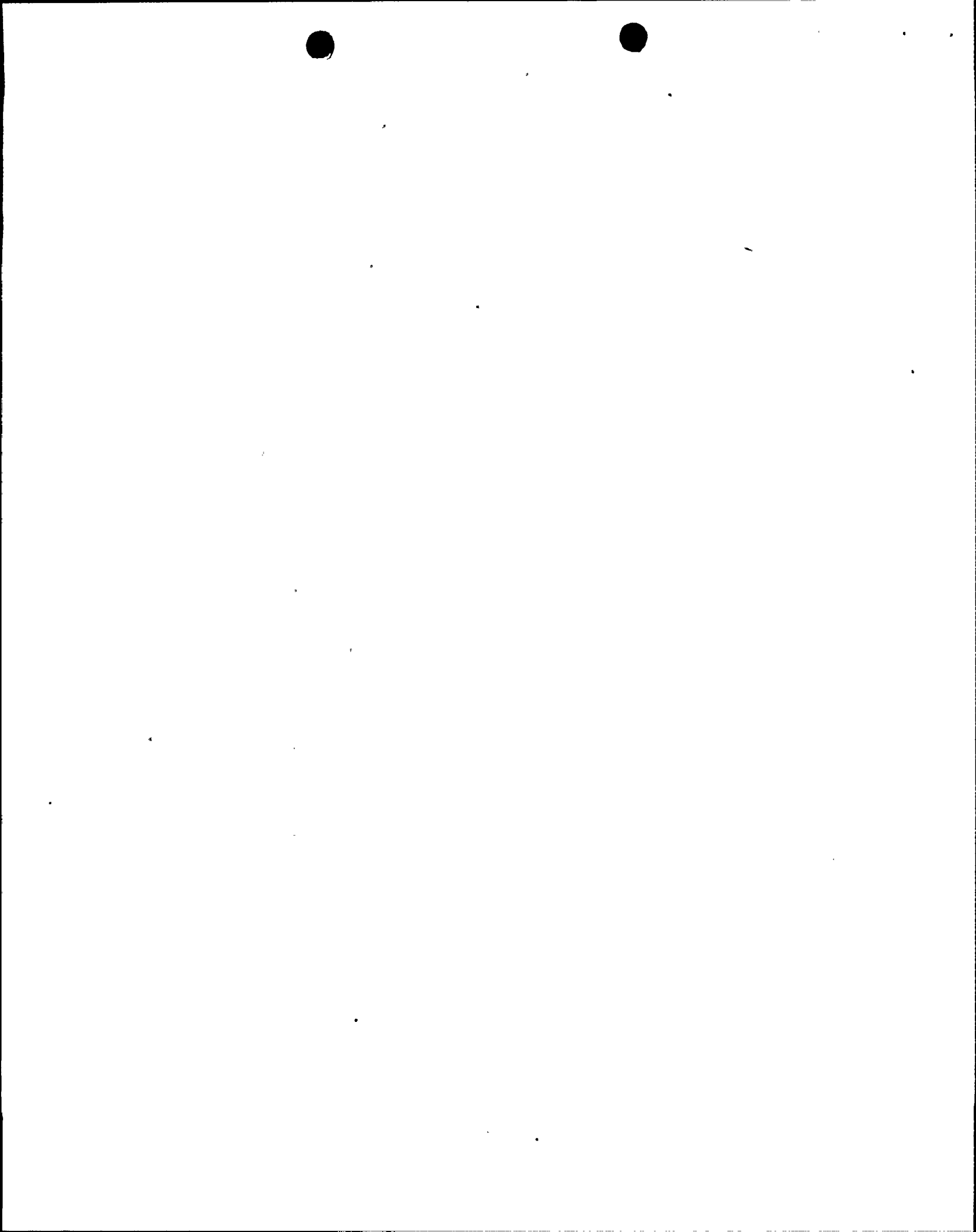
Date	Time	Event
8/01/80	0001	Reactor thermal power at 94%, maximum core flow - rod limited (sequence "B").
	2215	Reduced thermal power from 94% to 51% for removal of "B" recirculation pump MG set from service for brush replacement.
8/02/80	0310	"B" recirculation pump MG set brush replacement completed and pump placed in service, holding for control rod pattern adjustment.
	0700	Control rod pattern adjustment completed, commenced power ascension.
	0730	Commenced PCIOMR from 66% thermal power - control rod sequence "B".
8/03/80	1830	Reactor thermal power at 94%, maximum flow - rod limited.
8/04/80	0900	Commenced power ascension by withdrawing control rods.
	1500	Reactor thermal power at 99%, maximum flow.
8/09/80	0105	Reduced thermal power from 99% to 47% for control rod sequence exchange from "B" to "A".
	0310	Control rod sequence exchange completed, commenced power ascension.
	0555	Reactor thermal power at 60%, holding for TIP run.
	1245	TIP run completed, commenced power ascension.
	1430	Commenced PCIOMR from 77% thermal power - control rod sequence "A".
8/11/80	0310	Reactor thermal power at >99%, steady state.
	0505	Reduced thermal power from >99% to 85% due to "A" reactor feed pump trip on low oil pressure.
	0535	"A" reactor feed pump placed in service, commenced power ascension.
	1207	Commenced PCIOMR from 95% thermal power, control rod sequence "A".
	1905	Reactor thermal power at >99%, steady state.
8/13/80	1500	Reactor thermal power at 99%, maximum flow.
8/16/80	2210	Reduced thermal power from 98% to 63% for control rod pattern adjustment.
8/17/80	0125	Control rod pattern adjustment completed, commenced power ascension.
	0230	Commenced PCIOMR from 78% thermal power - control rod sequence "A".

Significant Operational Events

Unit 3

Date	Time	Event
8/18/80	0700	Reactor thermal power at 99%, maximum flow.
8/22/80	2207	Reduced thermal power from 99% to 65% for removal of "B" condensate booster pump from service for maintenance.
8/23/80	0145	"B" condensate booster pump maintenance completed, commenced power ascension.
	0315	Commenced PCIOMR from 85% thermal power, control rod sequence "A".
	1900	Reactor thermal power at 99%, maximum flow.
8/30/80	0030	Reduced thermal power from 99% to 80% for turbine C. V. tests and SI's.
	0200	Turbine C. V. tests and SI's completed, commenced power ascension.
	0800	Reactor thermal power at >99%, steady state.
8/31/80	2400	Reactor thermal power at >99%, steady state.

(1) Personnel error.



AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-259
 UNIT Browns Ferry - 1
 DATE 9-1-80
 COMPLETED BY Ted Thom
 TELEPHONE 205 729 6846

MONTH August 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>991</u>	17	<u>1023</u>
2	<u>1013</u>	18	<u>1009</u>
3	<u>1011</u>	19	<u>988</u>
4	<u>1013</u>	20	<u>1027</u>
5	<u>1032</u>	21	<u>1030</u>
6	<u>1016</u>	22	<u>1018</u>
7	<u>1019</u>	23	<u>803</u>
8	<u>1010</u>	24	<u>1008</u>
9	<u>977</u>	25	<u>1014</u>
10	<u>716</u>	26	<u>1023</u>
11	<u>971</u>	27	<u>1028</u>
12	<u>1021</u>	28	<u>1032</u>
13	<u>1017</u>	29	<u>1029</u>
14	<u>1026</u>	30	<u>1025</u>
15	<u>1030</u>	31	<u>1035</u>
16	<u>971</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-260UNIT Browns Ferry - 2DATE 9-1-80COMPLETED BY Ted ThomTELEPHONE 205 729 6846MONTH August 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>631</u>	17	<u>788</u>
2	<u>886</u>	18	<u>986</u>
3	<u>1005</u>	19	<u>955</u>
4	<u>990</u>	20	<u>974</u>
5	<u>971</u>	21	<u>969</u>
6	<u>755</u>	22	<u>964</u>
7	<u>969</u>	23	<u>965</u>
8	<u>1026</u>	24	<u>959</u>
9	<u>1016</u>	25	<u>950</u>
10	<u>1019</u>	26	<u>944</u>
11	<u>993</u>	27	<u>815</u>
12	<u>1005</u>	28	<u>972</u>
13	<u>988</u>	29	<u>953</u>
14	<u>997</u>	30	<u>926</u>
15	<u>994</u>	31	<u>957</u>
16	<u>272</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

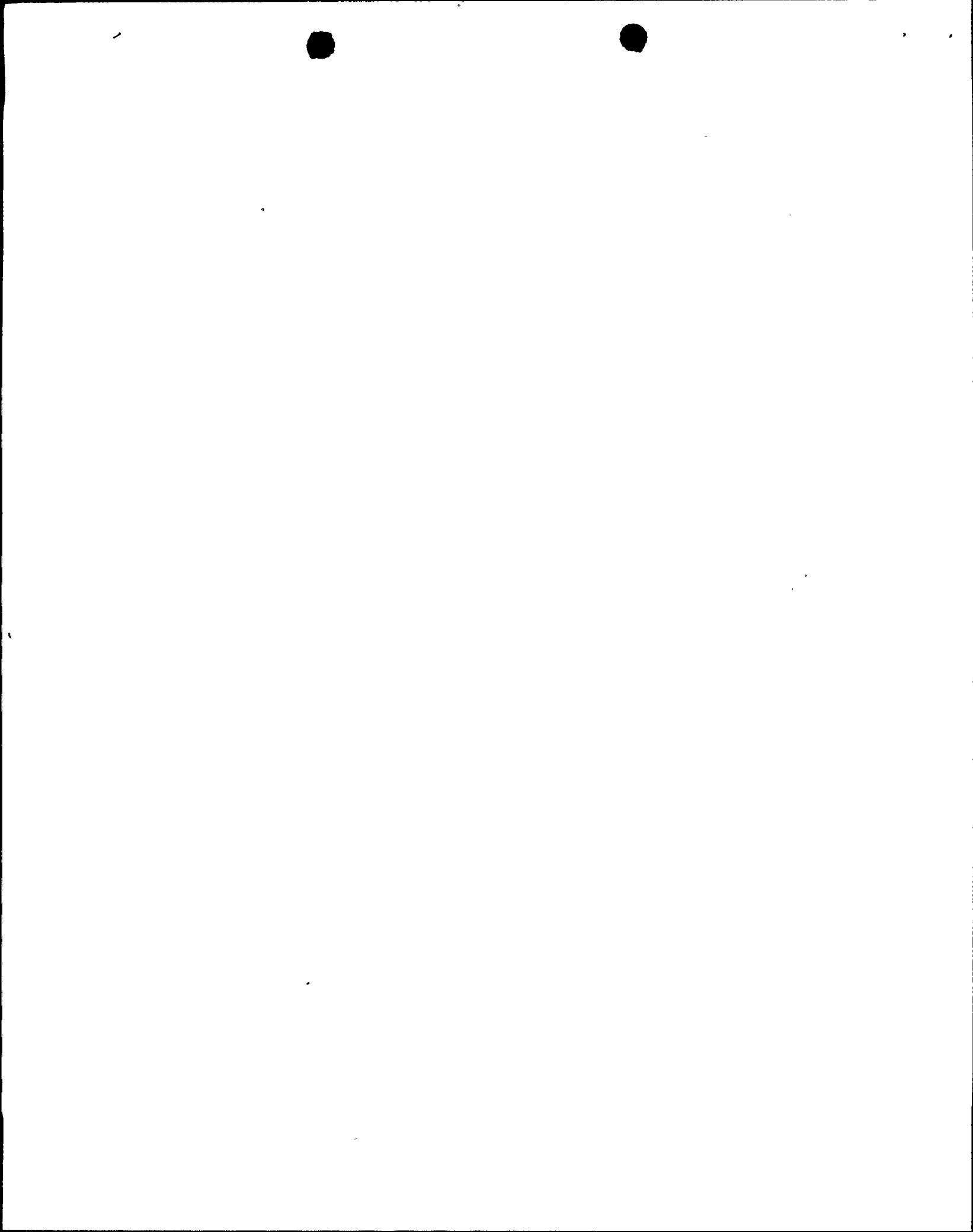
DOCKET NO. 50-296UNIT Browns Ferry - 3DATE 9-1-80COMPLETED BY Ted ThomTELEPHONE 205 729 6846MONTH August 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>955</u>	17	<u>870</u>
2	<u>731</u>	18	<u>1063</u>
3	<u>999</u>	19	<u>1057</u>
4	<u>1056</u>	20	<u>1062</u>
5	<u>1067</u>	21	<u>1067</u>
6	<u>1065</u>	22	<u>1038</u>
7	<u>1061</u>	23	<u>974</u>
8	<u>1040</u>	24	<u>1054</u>
9	<u>706</u>	25	<u>1057</u>
10	<u>975</u>	26	<u>1063</u>
11	<u>1015</u>	27	<u>1066</u>
12	<u>1054</u>	28	<u>1060</u>
13	<u>1023</u>	29	<u>1057</u>
14	<u>1037</u>	30	<u>1045</u>
15	<u>1034</u>	31	<u>1066</u>
16	<u>1003</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)



OPERATING DATA REPORT

DOCKET NO. 50-259
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

OPERATING STATUS

1. Unit Name: Browns Ferry - Unit 1
 2. Reporting Period: August 1980
 3. Licensed Thermal Power (MWt): 3293
 4. Nameplate Rating (Gross MWe): 1152
 5. Design Electrical Rating (Net MWe): 1065
 6. Maximum Dependable Capacity (Gross MWe): 1098.4
 7. Maximum Dependable Capacity (Net MWe): 1065

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
 10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>5,855</u>	<u>53,353</u>
12. Number Of Hours Reactor Was Critical	<u>744</u>	<u>3,653.13</u>	<u>31,930.41</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>356.42</u>	<u>5,046.03</u>
14. Hours Generator On-Line	<u>744</u>	<u>3,524.43</u>	<u>31,137.16</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>2,362,150</u>	<u>10,254,956</u>	<u>84,391,449</u>
17. Gross Electrical Energy Generated (MWH)	<u>766,580</u>	<u>3,297,780</u>	<u>27,816,240</u>
18. Net Electrical Energy Generated (MWH)	<u>741,958</u>	<u>3,194,295</u>	<u>26,996,263</u>
19. Unit Service Factor	<u>100</u>	<u>60.2</u>	<u>58.4</u>
20. Unit Availability Factor	<u>100</u>	<u>60.2</u>	<u>58.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>93.6</u>	<u>51.2</u>	<u>47.5</u>
22. Unit Capacity Factor (Using DER Net)	<u>93.6</u>	<u>51.2</u>	<u>47.5</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>11.2</u>	<u>31.1</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast

Achieved

OPERATING DATA REPORT

DOCKET NO. 50-260
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

OPERATING STATUS

1. Unit Name: Browns Ferry - Unit 2
 2. Reporting Period: August 1980
 3. Licensed Thermal Power (MWt): 3293
 4. Nameplate Rating (Gross MWe): 1152
 5. Design Electrical Rating (Net MWe): 1065
 6. Maximum Dependable Capacity (Gross MWe): 1098.4
 7. Maximum Dependable Capacity (Net MWe): 1065
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
 10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>5,855</u>	<u>48,264</u>
12. Number Of Hours Reactor Was Critical	<u>733.23</u>	<u>5,185.33</u>	<u>29,669.81</u>
13. Reactor Reserve Shutdown Hours	<u>10.77</u>	<u>648.94</u>	<u>12,423.88</u>
14. Hours Generator On-Line	<u>729.73</u>	<u>5,052.32</u>	<u>28,716.63</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>2,169,170</u>	<u>14,585,799</u>	<u>80,740,799</u>
17. Gross Electrical Energy Generated (MWH)	<u>710,000</u>	<u>4,838,890</u>	<u>26,788,958</u>
18. Net Electrical Energy Generated (MWH)	<u>686,119</u>	<u>4,699,430</u>	<u>26,030,899</u>
19. Unit Service Factor	<u>98.1</u>	<u>86.3</u>	<u>59.5</u>
20. Unit Availability Factor	<u>98.1</u>	<u>86.3</u>	<u>59.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>86.6</u>	<u>75.4</u>	<u>50.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>86.6</u>	<u>75.4</u>	<u>50.6</u>
23. Unit Forced Outage Rate	<u>1.9</u>	<u>12.4</u>	<u>33.6</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Refuel outage September 5, 1980, 42 days.</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATING DATA REPORT

DOCKET NO. 50-296
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

OPERATING STATUS

1. Unit Name: Browns Ferry - Unit 3
 2. Reporting Period: August 1980
 3. Licensed Thermal Power (MWt): 3293
 4. Nameplate Rating (Gross MWe): 1152
 5. Design Electrical Rating (Net MWe): 1065
 6. Maximum Dependable Capacity (Gross MWe): 1098.4
 7. Maximum Dependable Capacity (Net MWe): 1065
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
 10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>5,855</u>	<u>30,719</u>
12. Number Of Hours Reactor Was Critical	<u>744</u>	<u>5,100.22</u>	<u>24,007.77</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>101.63</u>	<u>1,759.94</u>
14. Hours Generator On-Line	<u>744</u>	<u>5,003.13</u>	<u>23,440.94</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>2,323,303</u>	<u>15,488,186</u>	<u>68,596,052</u>
17. Gross Electrical Energy Generated (MWH)	<u>779,470</u>	<u>5,217,450</u>	<u>22,620,030</u>
18. Net Electrical Energy Generated (MWH)	<u>754,173</u>	<u>5,073,556</u>	<u>21,961,329</u>
19. Unit Service Factor	<u>100</u>	<u>85.5</u>	<u>76.3</u>
20. Unit Availability Factor	<u>100</u>	<u>85.5</u>	<u>76.3</u>
21. Unit Capacity Factor (Using MDC Net)	<u>95.2</u>	<u>81.4</u>	<u>67.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>95.2</u>	<u>81.4</u>	<u>67.1</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>14.5</u>	<u>10.3</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refuel outage October 1980.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____



UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August

DOCKET NO. 50-259
 UNIT NAME Browns Ferry - 1
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
140	800809	S		B					Derated for recirculation pump MG set brush replacement
141	800818	F		B					Derated for maintenance to control rod 18-39
142	800822	S		H					Derated for control rod pattern adjustment

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-260
 UNIT NAME Browns Ferry - 2
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

REPORT MONTH August

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
152	800804	F		H					Derated due to loss of plant preferred power
153	800805	S		H					Derated for control rod pattern adjustment
154	800816	F	14.27	G	3				Personnel error while performing SI 4.2.A-8

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-296
 UNIT NAME Browns Ferry - 3
 DATE 9-1-80
 COMPLETED BY Don Green
 TELEPHONE 205 729 6846

REPORT MONTH August

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
139	800801	S		B					Derated for "B" recirculation pump brush replacement and control rod pattern adjustment
140	800809	S		H					Derated for control rod sequence exchange from "B" to "A"
141	800816	S		H					Derated for control rod pattern adjustment
142	800822	F		B					Derated for "B" condensate booster pump maintenance

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(9/77)

BROWNS FERRY NUCLEAR PLANT UNIT 1CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of August 1980

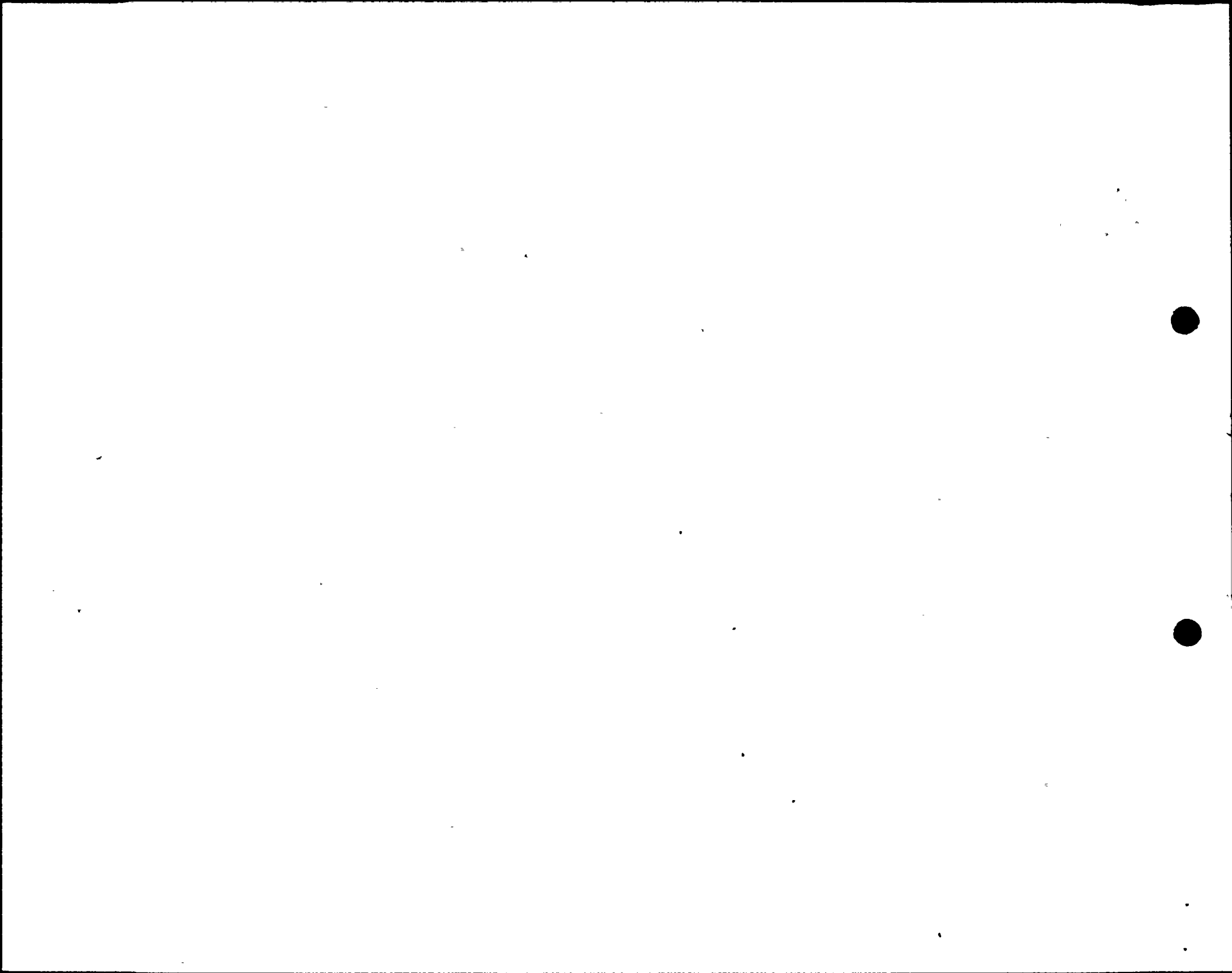
Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
8/10/80	Primary Containment	FCV-64-5	Refueling zone air supply outboard isolation damper getting open-closed indication (red-green light).	None	Limit switch out of adjustment	Incorrect position indication	Adjusted limit switch. Lights and limits operating properly checked okay. TR #178101
8/19/80	CRD	CRD Module 18-39	One of the scram solenoid valves on CRD module 18-39 is intermittently dropping out.	None	Bad Coil	Solenoid valve dropping out.	Replaced coil - checked okay. TR #178146
8/19/80	CRD	CRD	1B CRD pump breaker needs adjusting.	None	Breaker linkage out of adjustment	Breaker was trip-free	Adjusted trip free linkage - works properly now. TR #170696
8/14/80	RHR	1A RHR pump area cooler	1A RHR pump area cooler fan was found tripped on thermal overload.	None	Overload tripping was set too low	1A RHR area cooler inoperable causing 1A RHR pump to be considered inoperable per the Technical Specifications.	Adjusted over to correct setting check okay. TR #129327, 170699, and 181723 LER No. BFRQ-50-2598061

CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of August 19 80

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
8/24/80	Primary Containment	16A-K20 relay	Replaced relay 16A-K20 panel 9-43 drywell FD sump pump breaker interlock with valve..	None	Coil open	Stopped drywell flood drain sump pump	Replaced coil - checked okay. TR #180427



CSSC EQUIPMENT

ELECTRICAL MAINTENANCE SUMMARY

For the Month of August 1980

Date	System	Component	Nature of Maintenance	Effect on Safe Operation of The Reactor	Cause of Malfunction	Results of Malfunction	Action Taken To Preclude Recurrence
8/30/80	RHR	3A RHR Room	3A RHR Room Cooler fan 4x0 ACB thermal overloads tripping	None	Bad overload relay	3A RHR area cooler inoperable causing 3A RHR pump to be considered inoperable per the Technical Specifications.	Installed new relay checked okay. TR #178898 LER No. BFRO-50-296, 8034
8/25/80	RCIC	Magnetic Pickup for speed signal to EGM	Repair connector for magnetic pickup	None	Plug pulled off	No speed signal to EGM which made RCIC inoperable.	Replaced connector. Operations to test checked okay. TR #148297 LER No. BFRO-50-296, 8832

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of August 19 80

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
8/6/80	RBCCW	Pump 1B	Excessive leakage	None	Bad seal	Leakage	Replaced seal TR 181030
8/8/80	Diesel Generator	Diesel Generator D	No. 2 air comp. suction relief lifts continuously	None	Blown head gasket	Blown head gasket	Changed head gasket TR 177910
8/21/80	HPCI	FCV-1-73-6B	Leaking valve	None	Packing leak	Leaking valve	Tightened packing TR 103499
8/19/80	CRD	CRD Module 18-39	Scram pilot valve is exhausting intermittently	None	Pinhole in diaphragm on pressure side of valve	Leaking valve	Replaced both diaphragm solenoid base, subassembly all o-rings TR 153460

CSSC EQUIPMENT

MECHANICAL MAINTENANCE SUMMARY

For the Month of August 19 80

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
8/1/80	CRD	CRD Module 30-51	Leaking valve	None	Scram outlet valve leaking through	Leaking valve	Lengthened valve stem TR 153448
8/12/80	HPCI	Inboard rupture disk	Rupture disk is leaking	None	Rupture disk seal is leaking	HPCI isolated	Replaced disk TR 103659

CSSC EQUIPMENT

----- 3 -----
MECHANICAL MAINTENANCE SUMMARY

For the Month of August 1980

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
8/21/80	High Pressure Fire Protection	Drain valve 3-26-87	Leaking valve	None	Leaking valve	Leaking valve	Tightened valve TR 170192

BROWNS FERRY NUCLEAR PLANT UNIT 1,2 & 3

INSTRUMENT MAINTENANCE SUMMARY

CSSC EQUIPMENT

FOR THE MONTH OF August 19 80

DATE	SYSTEM	COMPONENT	NATURE OF MAINTENANCE	EFFECT ON SAFE OPERATION OF THE REACTOR	CAUSE OF MALFUNCTION	RESULTS OF MALFUNCTION	ACTION TAKEN TO PRECLUDE RECURRENCE
<u>Unit 1</u>							
8-4	Containment Inerting	FT-76-16	Calibrate	None	Zero Drift	Indicated Flow with System Off	None
<u>Unit 2</u>							
8-13	Primary Containment	PdT-64-138	Calibrate	None	Zero Drift	Indication Did Not Agree with Redundant Indicator	None
<u>Unit 3</u>							
8-5	Primary Containment	Pdic-64-2	Repair	None	Pneumatic Motor Leak	Reactor Zone Damper Not Functioning Properly	None

OUTAGE SUMMARY

August 1980

During the past month preparation for the Unit 2 cycle 3 outage have progressed on schedule. Numerous pre-outage work items have been completed. Work continued on the modification program including the Hypochlorite facility, the LLRW Facility, Outage Fabrication Facility, and the moving of the East Portal. Work was started on the generator breaker modification and prefabrication of piping for use during the upcoming outage.

The work schedule for the Unit 2 EOC 3 outage was finalized during this reporting period. Plant sections have provided their anticipated work schedule which has been included in the total outage schedule.