NRC FORM 366 (7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | A L B R F 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | | (5)
LICENSEE CODE · LICENSE NUMBER · LICENSE TYPE · CAT

CON'T 01 REPORT SOURCE L (6) | 0 5 0 0 0 2 6 0 (7) | 0 8 1 5 8 0 (8) | 0 9 0 8 8 0 (9)
DOCKET NUMBER · EVENT DATE · REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With the unit at 93 percent power in steady state operation, it was found during TI-
03 36A-that there was an indicated leak on 2B RHR heat exchanger. Redundant systems
04 LPCI, 1 pump in each RHR loop, and cross-connect capabilities were available. There
05 was no danger to the health or safety of the public and no significant resulting chain
06 of events. See Technical Specification 3.5.B.5. Previous occurrence reported on LER
07 BFRO-50-260/80034.
08

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
09	C F (11)	X (12)	Z (13)	H T E X C H (14)	X (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 0	—	0 3 3	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
Z (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	P 1 6 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The probable cause is a leaking gasket on the RHR heat exchanger due to loose
11 flange nuts resulting from thermal cycling and vibration. Corrective action has not
12 been completed. Several possible modifications are being evaluated to correct the
13 loosening of the flange nuts. Interim corrective action consists of increased torque
14 value on nuts.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	E (28)	0 9 3 (29)	NA	B (31)	Routine test

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
20	Y (44)	Press release	

NAME OF PREPARER 8009160 440 PHONE:

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF-17
BF 15.2
1/10/79

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 8033 Technical Specification Involved 3.5.B.5

Reported Under Technical Specification 6.7.2.b(2)

Date of Occurrence 8/15/80 Time of Occurrence 3:35 p.m. Unit 2

Identification and Description of Occurrence:

During routine test TI-36A it was found that there was an indicated leak on RHR heat exchanger 2B.

Conditions Prior to Occurrence:

Unit 1 - 96% steady state power

Unit 2 - 93% steady state power

Unit 3 - 97% steady state power

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

LPCI, 1 pump in each RHR loop, and cross connect capabilities were available.

Apparent Cause of Occurrence:

The heat exchanger has not been disassembled. Probable cause is a leaking flange gasket due to loose flange nuts caused by thermal cycling and vibration.

Analysis of Occurrence:

There was no danger to the health or safety of the public, no damage to plant or equipment, and no resulting significant chain of events.

Corrective Action:

Several modifications are being evaluated to prevent the loosening of the nuts. A DCR has been drafted to install locking tabs for these nuts. RHRSW monitoring will detect any release of activity. Routine inspection will detect heat exchanger problems. Interim corrective action consists of increased torque value on nuts.

Failure Data: LER BFRO-50-260/80034; BFRO-50-259/80043; BFRO- 50-259/78023.

*Retention: Period - Lifetime; Responsibility - Administrative Supervisor

*Revision: