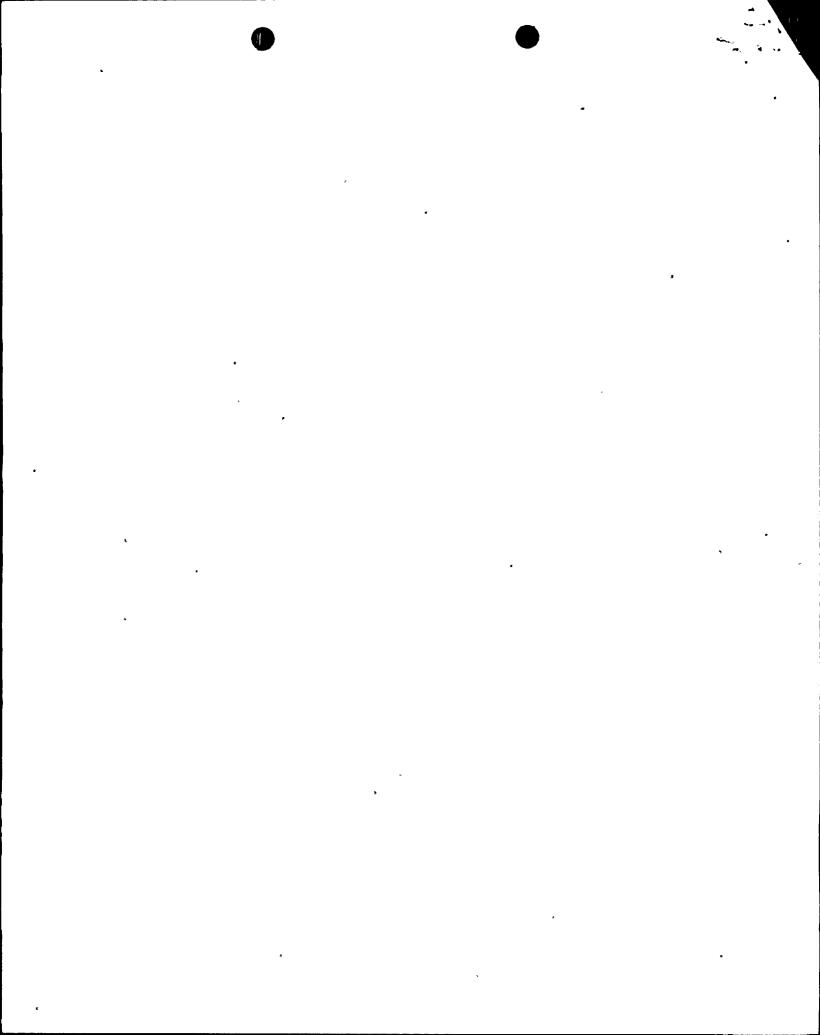
U.S. NUCCEAR REGULATION - - - -LICENSEE EVENT REPORT ٦Ō CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 0 0 0 - |0 3 4 1 1 1 10 1 I B LICENSE NUMBER CON'T REPORT 8 0 8 0 6 3 0 Lx 6 0 15 10 10 10 12 16 10 10 L0 1 6 1 1 0 1 هلمان SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) |During normal operation it was discovered that a spare Dresser Industries safety valve ; 0 2 had a guide with cracks. Preliminary engineering evaluation has shown guide is a non-0 3 pressure boundary component and not necessary to maintain integrity of primary 0 4 coolant system. Evaluation also indicates no interference with valve operation. T.S. 0 5 not applicable. There was no danger to health or safety of public. No previous 0 6 occurrence. 07 0 8 SYSTEM CODE CAUSE COMP. SUBCODE CAUSE VALVE SU9CODE SUBCODE COMPONENT CODE CODE CIC B | (12) B | (13) X |(14 1 E 9 (16) 18 OCCURRENCE SEQUENTIAL REPORT NO. REVISION REPORT LERIRO EVENT YEAR CODE TYPE NO. (17) REPORT 8 0 0 2 5 0 T 10 1 NUMBER 32 ACTION FUTURE TAKEN ACTION METHOD ATTACHMENT SUBMITTED EFFECT ON PLANT PRD-4 PRIME COMP. COMPONENT HOURS (22) FORM SUS. MANUFACTURES SUPPLIER Z (20) Z,](21) 0 0 0 0 Y (23) IX |] Y 2 4](24)](25) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Crack induced during initial pouring of casting and subsequent cooling with service 10 condition caused cracks to become visible. Spare valves on hand will be disassembled 1 1 and inspected for cracks. Valve usage expected to be terminated by spring 1983. 1 2 Dresser safety valve, Model 3707RA-RT21, Size 6 x 10, pressure class 1500 lbs., 1 3 A565 material. 4 8 9 8e FACILITY METHOD OF OTHER STATUS % POWER DISCOVERY DESCRIPTION (32) 0 9 4 29 D (31) NA Another Utility E (28) 80 ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE RELEASED_OF RELEASE 6 NA NA 10 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER 37 Z NA 0 13 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 8 0 0 (40) NA 0 12 89 11 Soss of or DAMAGE TO FACILITY (43) DESCRIPTION Z (42) NA 86 PUBLICITY NRC USE ONLY DESCRIPTION (45) 8007080373 (44) NA 20.0 10 80 5 ş MAME OF PREPARED PHONE --



Tennessee Valley Author Browns Ferry Nuclear Plant

LER SUPPLEMENTAL INFORMATION

BFRO-50- <u>260 / 8025</u> Technical Specification Involved Not applicable
Reported Under Technical Specification 6.7.2.a(9)
Date of Occurrence 6/17/80 Time of Occurrence 12:30 p.m. Unit 2
<u>Identification and Description of Occurrence:</u> This was first discovered at another utility. Subsequently, a spare Dresser ma steam safety valve was inspected at BFNP and found to have cracks on the upper portion of the guides.
Conditions Prior to Occurrence:
Unit 1 - Hot shutdown, preparing to go critical for return to service.
Unit 2 - Normal operation at 1055 MWe.
Unit 3 - Normal operation at 1100 MWe.
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Action specified in the Technical Specification Surveillance Requirements met <u>due to inoperable equipment</u> . Describe.
NA
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Apparent Cause of Occurrence:
Cracks were induced at the time of initial pouring of the casting and subsequen cooling with service conditions of alternate high heating and cooling causing t

## Analysis of Occurrence:

cracks to become more apparent.

Dresser engineering evaluation was that the cracks, due to the method of their forming, would not be liable to propagate. Battelle Memorial Institute has confirmed the initial evaluation of Dresser Engineering Personnel and their metallurgist agrees that no propagation of the crack is indicated by the sample which was examined.

## Corrective Action:

Cracking in this guide (which is not a pressure retaining part) has no effect on the operation, functionability, or design life of the valve. All guides will be inspected for cracks.

## Failure Data:

NA

*Retention: Period - Lifetime; Responsibility - Administrative Supervisor

*Revision:

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