

**NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL**  
(TEMPORARY FORM)

CONTROL NO: 9169

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FROM: Penn. Power & Light Co. Allentown, Pa. 18101 N.W. Curtis		DATE OF DOC 8-25-75	DATE REC'D 8-28-75	LTR XX	TWX	RPT	OTHER
TO: Dr. Walter R. Butler		ORIG 1 signed	CC	OTHER	SENT NRC PDR _____ XX		SENT LOCAL PDR _____ XX
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-387388		

DESCRIPTION: Ltr furnishing addl info re Mark II containments & trans the following:

ENCLOSURES: Mark II Supporting Program for the Susquehanna Plant Units 1 & 2....  
  
(1 cy encl rec'd)

**Do Not Remove**  
**ACKNOWLEDGED**

PLANT NAME: Suseqhehanna Plant Units 1 & 2

**FOR ACTION/INFORMATION**

DHL 9-2-75

BUTLER (L) W/ Copies	SCHWENCER (L) W/ Copies	ZIEMANN (L) W/ Copies	REGAN (E) W/ Copies	REID (L) W/ COPIES
CLARK (L) W/ Copies	STOLZ (L) W/ Copies	DICKER (E) W/ Copies	LEAR (L) W/ Copies	
PARR (L) W/ Copies	VASSALLO (L) W/ Copies	KNIGHTON (E) W/ Copies	SPIES W/ Copies	
KNIEL (L) W/ Copies	PURPLE (L) W/ Copies	YOUNGBLOOD (E) W/ Copies	LPH W/ Copies	

**INTERNAL DISTRIBUTION**

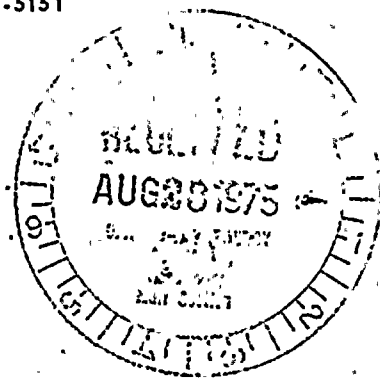
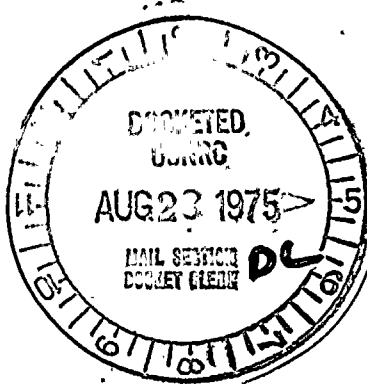
<u>REG FILE (2)</u>	<u>TECH REVIEW</u>	DENTON	<u>LIC ASST</u>	<u>A/T IND.</u>
<u>NRC PDR (2)</u>	SCHROEDER	GRIMES	R. DIGGS (L)	BRAITMAN
<u>OGC, ROOM P-506A</u>	MACCARY	GAMMILL	H. GEARIN (L)	SALTZMAN
<u>GOSSICK/STAFF</u>	KNIGHT	KASTNER	E. GOULBOURNE (L)	MELTZ
<u>BASE</u>	PAWLICKI	BALLARD	P. KREUTZER (E)	
<u>GIAMBUSSO</u>	SHAO	SPANGLER	J. LEE (L)	<u>PLANS</u>
BOYD	STELLO		<u>M. RUSHBROOK (L)</u>	MCDONALD
MOORE (L)	HOUSTON	<u>ENVIRO</u>	S. REED (E)	CHAPMAN
DEYOUNG (L)	NOVAK (3)	MULLER	M. SERVICE (L)	<u>Coupe (Ltr) E. Hughes</u>
SKOVHOLT (L)	CROSS	DICKER	S. SHEPPARD (L)	<u>E. COUPE</u>
GOLLER (L) (Ltr)	IPPOLITO	KNIGHTON	M. SLATER (E)	PETERSON
P. COLLINS	FEDESCO	YOUNGBLOOD	H. SMITH (L)	HARTFIELD (2)
DENISE	J. COLLINS	BEGAN	S. TEETS (L)	KLECKER
REG OPR	LAINAS	<u>PROJECT LDR</u>	G. WILLIAMS (E)	EISENHUT
<u>FILE &amp; REGION (2)</u>	BENAROYA	<u>Bevan</u>	V. WILSON (L)	WIGGINTON
MIPC	VOLLMER	HARLESS	R. INGRAM (L)	<u>VARGA</u>
			M. DUNCAN (E)	

**EXTERNAL DISTRIBUTION**

- |                                  |                                |                        |
|----------------------------------|--------------------------------|------------------------|
| 1 - LOCAL PDR Wilkes Barre, Pa.  | 1 - NATIONAL LABS _____        | 1 - PDR-SAN/LA/NY      |
| 1 - TIC (ABERNATHY) (1)(2)(10)   | 1 - W. PENNINGTON, Rm E-201 GT | 1 - BROOKHAVEN NAT LAB |
| 1 - NSIC (BUCHANAN)              | 1 - CONSULTANTS                | 1 - G. ULRIKSON ORNL   |
| 1 - ASLB                         | NEWMARK/BLUME/AGBABIAN         |                        |
| 1 - Newton Anderson              |                                |                        |
| 1 - ACRS <del>REPORT</del> /SENT |                                |                        |

*[Handwritten Signature]*

August 25, 1975



Dockets No's 50-387

50-388

Dr. Walter R. Butler, Chief  
 Light Water Reactors Branch 1-2  
 Division of Reactor Licensing  
 U.S. Nuclear Regulatory Commission  
 Washington, DC 20555

SUSQUEHANNA SES  
 SUPPORTING PROGRAM FOR MARK II CONTAINMENTS  
 ER 100450 FILE 840-2  
 PLA-80

Dear Dr. Butler:

Attached is a copy of the outline of the BWR Mark II Containment "Supporting Program" presented in a meeting with members of your staff on June 30, 1975. The outline has been expanded to identify the testing and analytical portions of the program and indicates the purpose and schedule for completion of each portion. Full scale in-plant testing of a ramshead device has been added to the program. Analysis of Alternate Load Mitigating Design and "SRL-1" Mitigating Fix Testing have been deleted because no additional load mitigating devices have been identified as necessary. The attached outline is a more detailed schedule for item 5 of the Mark II Containment Program attached to our letter dated June 5, 1975.

The Supporting Program is intended to Confirm the Preliminary Forcing Function Report (PFFR) which is scheduled to be submitted to you in September, 1975. It is intended that the PFFR will be reviewed and approved by the NRC as an acceptable method for handling S/R valve and LOCA hydrodynamic loads for containment design.

Very truly yours,

N. W. Curtis  
 Vice President-Engineering & Construction

WEB:DEB

MARK II SUPPORTING PROGRAM

File

8-25-75

<u>MARK II PROGRAM ACTIVITY</u>	<u>ACTIVITY TYPE</u>	<u>TARGET SCHEDULE</u>
A. LOCA RELATED		
1. 4T Pool Swell Test	Test	1st. Quarter 1976
2. Pool Swell Verlocity Break-through Model	Analysis	2nd. Quarter 1976
3. Impact Tests on Pool Internal Structures	Test	3rd. Quarter 1975
4. Qualify Impact Model	Analysis	4th. Quarter 1975
B. SAFETY/RELIEF VALVE RELATED		
1. Relief Valve Pipe Clearing Model for Quencher	Analysis	4th. Quarter 1976
2. Relief Valve Pipe Clearing Model for Ramshead	Analysis and Test	3rd. Quarter 1975
3. In-Plant Test of S/R Valve Discharge Loads During Consecutive Actuations for Ramshead	Test	3rd. Quarter 1976
4. Relief Valve Consecutuve Actuation Transient Analysis	Analysis	4th. Quarter 1976

## A.1 "4T" POOL SWELL TEST

### Objective

Evaluate the pool swell phenomena for a typical Mark II containment geometry to determine pool swell velocities, water slug thickness, breakthrough elevations and wetwell pressures in a near full scale vertical vent suppression system using one vent at the General Electric Pressure Suppression Test Facility (PSTF). The information will be used to predict the loading on wetwell structural members and the diaphragm floor between the wetwell and drywell.

## A.2 POOL SWELL VELOCITY BREAKTHROUGH MODEL

### Objective

Develop an analytical model for predicting water slug velocity, pool swell breakthrough and froth characteristics. Impact loads on structures above the pool are dependent on these parameters. An analytical model is desired to bridge the gap between test results and actual containment geometry. This activity complements the "4T" test program as its analytical counter-part.

## A.3 IMPACT TESTS ON POOL INTERNAL STRUCTURES

### Objective

Conduct pool swell testing on various structural shapes of pool internals to estimate impact loadings on internal structures. This activity involves one third scale shapes in the Pressure Suppression Test Facility (PSTF). The shapes include circular pipes and I-beams of various sizes plus steel grating. Test results will be used to predict impact loads.

## A.4 QUALIFY IMPACT MODEL

### Objective

Confirm applicability of PSTF data to Mark II geometry and structures. Evaluate impact test data for design application and analytical model verification. If necessary modify current analytical models. Confirm design adequacy of analytical methods with pool swell data. Document analytical methods, assumptions, experimental verification of analytical methods, and experimental basis for specified loads. This activity provides analytical modeling of the impact tests on pool internal structures and components.

### B.1 RELIEF VALVE PIPE CLEARING MODEL FOR QUENCHER

#### Objective

Develop an analytical model of this phenomenon to provide an improved technical basis for design and greater flexibility on design details. Compare this analytical model to current test data. This model will not include prediction of internal pipe pressure effects since they are being developed in activity B.2.

### B.2 RELIEF VALVE PIPE CLEARING MODEL FOR RAMSHEAD

#### Objective

Document the Safety/Relief Valve Pipe Clearing analytical model in a supplement to the Quad Cities Topical Report (NEDO-10859). This activity includes documentation of the Safety/Relief Valve Pipe Clearing Analytical Model for a ramshead including assumptions, justification of analytical methods, and description of the experimental basis for the analytical models. This will include the development of analytical models to predict internal pipe pressures applicable to both the ramshead and the quencher.

### B.3 IN-PLANT TEST OF SAFETY/RELIEF VALVE DISCHARGE LOADS DURING CONSECUTIVE ACTUATIONS

#### Objective

Measure the effect of consecutive SRV discharges on suppression pool pressures and internal pipe pressure. Test data will be used to verify pipe internal pressure and water level for consecutive SRV actuations. Measurements will be made of strains imposed on a pressure suppression containment torus during consecutive valve actuations with varying delay times (i.e., time between valve closure and reopening) in order to determine maximum strain and variation of strain with delay time. Additional data will be obtained to aid in defining the phenomena causing load changes. Internal pipe pressure data and pipe reaction forces will be obtained for verification of analytical model. Pressure measurements at various locations in the pressure suppression pool will be taken. This testing is being conducted by the Mark I Group, however, the Mark II group is participating in order to obtain additional test data on pipe internal pressures, pipe reaction forces and pressure attenuation in the suppression pool.

### B.4 RELIEF VALVE CONSECUTIVE ACTUATION TRANSIENT ANALYSIS

#### Objective

Develop analytical methods for predicting S/R valve discharge pipe pressures for both ramshead and quencher devices and containment loads associated with consecutive S/R valve actuations. This analytical model includes the effects of a vacuum breaker and reflooding due to rapid steam condensation. Both internal pipe pressures and pressures in the suppression pool will be predicted. Current test data will be evaluated and analytical models will be verified. If necessary, analytical models will be modified. Verified analytical models for first and subsequent SRV actuations will be documented. This activity provides the analytical modeling of the testing done in Item B.3 in order to correlate its applicability to the Mark II Containment geometry.